

February 25, 1988

Docket No. 50-346
Serial No. DB-88-005

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Mr. Donald C. Shelton
Vice President, Nuclear
Toledo Edison Company
Edison Plaza - Stop 712
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Dear Mr. Shelton:

SUBJECT: AMENDMENT NO. 106 TO FACILITY OPERATING LICENSE NO. NPF-3:
FIRE PROTECTION (TAC NO. 65361)

The Commission has issued Amendment No. 106 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TS's) in response to your application dated December 7, 1987 (No. 1446).

This amendment revises the TS Sections 3/4.7.10, 6.4, and 6.9, and Bases section 3/4.7.10 to update the TS's to reflect current plant design, testing, and compensatory measures.

Copies of the Safety Evaluation and of the notice of issuance are enclosed.

Sincerely,

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Albert W. De Agazio, Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV
V & Special Projects

Enclosures:

1. Amendment No.106to
License No. NPF-3
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures: See next page

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Date: *2/11/88*

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2/18/88

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2/14/88

OGC-WF1
[Signature]
2/24/88

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Mr. Donald C. Shelton
Toledo Edison Company

Davis-Besse Nuclear Power Station
Unit No. 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated December 7, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 106, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than March 26, 1988 .

FOR THE NUCLEAR REGULATORY COMMISSION



Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 25, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. Corresponding overleaf pages are provided to maintain document completeness.

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PLANT SYSTEMS

3/4.7.10 FIRE BARRIERS

LIMITING CONDITION FOR OPERATION

3.7.10 All fire barriers separating portions of redundant safe shutdown systems required in the event of a fire shall be OPERABLE.

APPLICABILITY: At all times.

ACTION:

- a. With one or more of the above fire barriers inoperable, within 1 hour, either:
 - 1. Establish a continuous fire watch on at least one side of the affected fire barrier, or
 - 2. Verify the OPERABILITY of the fire detectors on at least one side of the affected fire barrier and establish an hourly fire watch patrol.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

- 4.7.10 Each of the above required fire barriers, including sealing devices, shall be verified OPERABLE by:
- a. Performing a visual inspection of the exposed surfaces of each fire-rated wall*, floor and ceiling, electrical raceway fire enclosure and structural steel fire-proofing at least once per 18 months.
 - b. Performing a visual inspection of each fire door, fire damper and associated hardware at least once per 18 months.

*Barrier 102 West/210 East and a portion of barriers 206 East/210 West and 205 North/206 South behind the filter bank are not subject to the requirements for visual inspection due to ALARA considerations.

SURVEILLANCE REQUIREMENTS

- c. Performing a visual inspection of at least ten percent of each type of sealed penetration at least once per 18 months. If the penetration(s) is determined to be inoperable, declare the affected penetration(s) inoperable and perform a visual inspection of an additional ten percent of the degraded type of sealed penetration. This inspection process shall continue until a ten percent sample with no visually apparent adverse changes in appearance or changes from the as-built condition are found or until all required sealed penetrations of the degraded type have been inspected. Samples shall be selected such that each penetration seal will be inspected at least once per 15 years.
- d. Verifying at least once per 24 hours each fire door (i) that is unlocked is closed or (ii) that is equipped with an automatic hold-open and release mechanism is free from obstructions.
- e. Verifying at least once per 7 days each locked fire door is closed and locked.
- f. Performing a functional test that verifies the operation of automatic hold-open and release mechanisms upon full openings, and latch and closing mechanisms upon full and partial openings, at least once per 18 months.

PLANT SYSTEMS

BASES

3/4.7.8 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from by product, source, and special nuclear material sources will not exceed allowable intake values.

3/4.7.9 FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the inoperable equipment is restored to service.

In the event the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

3/4.7.10 FIRE BARRIERS

The OPERABILITY of the fire barrier ensures that fires will be confined or adequately retarded from spreading to adjacent fire areas or to portions of redundant safe shutdown systems required in the event of a fire within the fire area. This design feature minimizes the possibility of a single fire rapidly involving several fire areas of the facility prior to detection and extinguishment. The fire barriers are passive elements in the facility fire protection program.

Fire barriers, including cable penetration barriers, fire doors and dampers, are considered OPERABLE when the visually observed condition is the same as the as-designed condition. The as-designed condition of each fire barrier is based on a tested configuration or a configuration analyzed to withstand the fire hazards associated with the fire area.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATION

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Chemistry and Health Physics General Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Director.

6.5 REVIEW AND AUDIT

6.5.1 STATION REVIEW BOARD (SRB)

FUNCTION

6.5.1.1 The Station Review Board (SRB) shall function to advise the Plant Manager on all matters related to nuclear safety.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.1.2 The Station Review Board shall be composed of the:

Chairman:	Station Review Board Chairman*
Member:	Assistant Plant Manager, Operations
Member:	Assistant Plant Manager, Maintenance
Member:	Technical Support Manager
Member:	Chemistry and Health Physics General Superintendent
Member:	Operations Engineering Supervisor (Plant)
Member:	An Engineering Director or Performance Engineering Manager
Member:	Operations Quality Assurance Manager
Member:	Operations Superintendent (Plant)

* Designated in writing by the Plant Manager. The Chairman will be drawn from SRB members.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SRB shall meet at least once per calendar month and as convened by the SRB Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SRB shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Review Board shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.

ADMINISTRATIVE CONTROLS

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- b. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. Seismic event analysis, Specification 4.3.3.3.2.
- e. Fire Detection Instrumentation, Specification 3.3.3.8.
- f. Fire Suppression Systems, Specifications 3.7.9.1 and 3.7.9.2.
- g. Dose or dose commitment exceedences to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released to UNRESTRICTED AREAS (Specification 3.11.1.2).
- h. The discharge of radioactive liquid waste without treatment and in excess of the limits in Specification 3.11.1.3.
- i. The calculated air dose from radioactive gases exceeding the limits in Specification 3.11.2.2.
- j. The calculated dose from the release of iodine-131, tritium, and radionuclides in particulate form with half-lives greater than 8 days, in gaseous effluents exceeding the limits in Specification 3.11.2.3.
- k. The discharge of radioactive gaseous waste without treatment and in excess of the limits in Specification 3.11.2.4.
- l. The calculated doses from the release of radioactive materials in liquid or gaseous effluents exceeding the limits in Specification 3.11.4.
- m. The level of radioactivity as the result of plant effluents in an environmental sampling medium exceeding the reporting levels of Table 3.12-2 (Specification 3.12.1).

ADMINISTRATIVE CONTROLS

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE EVENTS.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATING TO AMENDMENT NO. 106 TO FACILITY OPERATING LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

1.0 INTRODUCTION

On May 20, 1987, Toledo Edison Company submitted an application for amendment of the Davis-Besse Nuclear Power Station Appendix A Technical Specifications (TS's) relating to the Davis-Besse fire protection program. The staff reviewed that proposed amendment but expressed certain concerns and raised several questions regarding Toledo Edison Company's approach to a number of issues. A significant issue concerned Toledo Edison Company's proposal to remove from the TS-required surveillance those fire protection features that are necessary to conform with Toledo Edison Company's commitments to comply with the guidelines contained in Appendix A to Branch Technical Position APCSB 9.5-1. Toledo Edison Company submitted a revised (and reduced scope) application dated December 7, 1987. This revised application concerns fire barriers, fire brigade training and related issues.

When Toledo Edison Company submitted the revised application, the commitment was made to implement the recommendations contained in Generic Letter 86-10 concerning removal of fire protection matters from the technical specifications. This amendment is, therefore, one step toward eventual conformance with G.L. 86-10. The staff's evaluation of this amendment is as follows.

2.0 DISCUSSION

Toledo Edison Company has characterized the changes to the technical specifications as being either administrative in nature, including more stringent requirements, or reflecting actual plant design.

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Examples of the administrative changes include reformatting the Action Statements and Surveillance Requirements and clarification of specific equipment addressed by the proposed TS's. Examples of the more stringent requirements include the addition of Action Statements and Surveillance Requirements. Examples of changes that reflect the actual plant design and fire protection program include defining inaccessible equipment and revising the scope of the TS's to address safe shutdown equipment required in the event of a fire.

3.0 EVALUATION

The staff has reviewed specifically all proposed changes. Those changes that are administrative in nature will resolve certain ambiguities that are present in the existing technical specification and are, therefore, acceptable. The remaining changes, which are of a technical nature, are consistent with provisions of the standard technical specifications or past precedent. These changes conform with the guidance issued in Appendix A to BTP APCSB and are, therefore, acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

An Environmental Assessment and Finding of No Significant Impact has been issued for this Amendment (53 FR 5061, February 19, 1988).

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Kubicki

Dated: February 25, 1988

U.S. NUCLEAR REGULATORY COMMISSIONTOLEDO EDISON COMPANYTHE CLEVELAND ELECTRIC ILLUMINATING COMPANYDOCKET NO. 50-346NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 106 to Facility Operating License No. NPF-3, issued to Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees), which revised the Technical Specifications (TS's) for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility), located in Ottawa County, Ohio. The amendment is effective as of the date of issuance and is to be implemented not later than

This amendment revises the TS Sections 3/4.7.10, 6.4, and 6.9, and Bases section 3/4.7.10 to update the TS's to reflect current plant design, testing, and compensatory measures relating to the fire protection features and programs at the facility.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings, as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Prior Hearing in connection with this action was published in the Federal Register on December 16, 1987 (52 FR 47811). No request for a hearing or petition to intervene was filed following this notice.

Also in connection with this action, the Commission prepared an Environmental Assessment and Finding of No Significant Impact which was published in the Federal Register on February 19, 1988 at 53 FR 5061.

For further details with respect to this action, see (1) the application for amendment dated December 7, 1987, (2) Amendment No. 106, to license No. NPF-3, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H St. N. W., Washington, D. C. and at the University of Toledo Library, Documents Department, 2801 Bancroft Avenue, Toledo, Ohio. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Projects - III, IV, V, and Special Projects.

Dated at Bethesda, Maryland, this 25th day of February 1988.

FOR THE NUCLEAR REGULATORY COMMISSION



Albert W. De Agazio, Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV,
V & Special Projects