February 25, 1988

Docket No. 50-346
Serial No. DB-88-005

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Mr. Donald C. Shelton
Vice President, Nuclear
Toledo Edison Company
Edison Plaza - Stop 712
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Toledo, Ohio 43652

Dear Mr. Shelton:

SUBJECT: AMENDMENT NO. 106 TO FACILITY OPERATING LICENSE NO. NPF-3: FIRE PROTECTION (TAC NO. 65361)

The Commission has issued Amendment No. 106 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TS's) in response to your application dated December 7, 1987 (No. 1446).

This amendment revises the TS Sections 3/4.7.10, 6.4, and 6.9, and Bases section 3/4.7.10 to update the TS's to reflect current plant design, testing, and compensatory measures.

Copies of the Safety Evaluation and of the notice of issuance are enclosed.

Sincerely,

/s/

Albert W. De Agazio, Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV
V & Special Projects

Enclosures:
1. Amendment No. 106 to License No. NPF-3
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures: See next page

Office:	LA/PDIII-3	PM/PDIII-3	PD/PDIII-3	OGC-WF1
Surname:	PKreutzer	ADeAgazio/tg	KPerkins	
Date:	2/18/88	2/18/88	2/19/88	2/24/88

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(exact copy)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated December 7, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 106 , are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than March 26, 1988 .

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects - III, IV, V and Special Projects

Attachment:
Changes to the Technical Specifications

Date of Issuance: February 25, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. Corresponding overleaf pages are provided to maintain document completeness.

REMOVE	INSERT
VII	VII
XII	XII
3/4 7-47	3/4 7-47
-	3/4 7-48
B 3/4 7-6	B 3/4 7-6
6-5	6-5
6-18	6-18

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

BASES

INDEX

BASES

PLANT SYSTEMS

3/4.7.10 FIRE BARRIERS

LIMITING CONDITION FOR OPERATION

3.7.10 All fire barriers separating portions of redundant safe shutdown systems required in the event of a fire shall be OPERABLE.

APPLICABILITY: At all times.

ACTION:

a. With one or more of the above fire barriers inoperable, within 1 hour, either:

1. Establish a continuous fire watch on at least one side of the affected fire barrier, or

2. Verify the OPERABILITY of the fire detectors on at least one side of the affected fire barrier and establish an hourly fire watch patrol.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.10 Each of the above required fire barriers, including sealing devices, shall be verified OPERABLE by:

a. Performing a visual inspection of the exposed surfaces of each fire-rated wall*, floor and ceiling, electrical raceway fire enclosure and structural steel fire-proofing at least once per 18 months.

b. Performing a visual inspection of each fire door, fire damper and associated hardware at least once per 18 months.

*Barrier 102 West/210 East and a portion of barriers 206 East/210 West and 205 North/206 South behind the filter bank are not subject to the requirements for visual inspection due to ALARA considerations.

c. Performing a visual inspection of at least ten percent of each type of sealed penetration at least once per 18 months. If the penetration(s) is determined to be inoperable, declare the affected penetration(s) inoperable and perform a visual inspection of an additional ten percent of the degraded type of sealed penetration. This inspection process shall continue until a ten percent sample with no visually apparent adverse changes in appearance or changes from the as-built condition are found or until all required sealed penetrations of the degraded type have been inspected. Samples shall be selected such that each penetration seal will be inspected at least once per 15 years.

d. Verifying at least once per 24 hours each fire door (i) that is unlocked is closed or (ii) that is equipped with an automatic hold-open and release mechanism is free from obstructions.

e. Verifying at least once per 7 days each locked fire door is closed and locked.

f. Performing a functional test that verifies the operation of automatic hold-open and release mechanisms upon full openings, and latch and closing mechanisms upon full and partial openings, at least once per 18 months.

3/4.7.8 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from by product, source, and special nuclear material souces will not exceed allowable intake values.

3/4.7.9 FIRE SUPPRESSION SYSTEMS

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinquish fires occuring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, spray and/or sprinklers, and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the inoperable equipment is restored to service.

In the event the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

3/4.7.10 FIRE BARRIERS

The OPERABILITY of the fire barrier ensures that fires will be confined or adequately retarded from spreading to adjacent fire areas or to portions of redundant safe shutdown systems required in the event of a fire within the fire area. This design feature minimizes the possibility of a single fire rapidly involving several fire areas of the facility prior to detection and extinguishment. The fire barriers are passive elements in the facility fire protection program.

Fire barriers, including cable penetration barriers, fire doors and dampers, are considered OPERABLE when the visually observed condition is the same as the as-designed condition. The as-designed condition of each fire barrier is based on a tested configuration or a configuration analyzed to withstand the fire hazards associated with the fire area.

6.3 FACILITY STAFF QUALIFICATION

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Chemistry and Health Physics General Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Director.

6.5 REVIEW AND AUDIT

6.5.1 STATION REVIEW BOARD (SRB)

FUNCTION

6.5.1.1 The Station Review Board (SRB) shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Review Board shall be composed of the:

Chairman:	Station Review Board Chairman*
Member:	Assistant Plant Manager, Operations
Member:	Assistant Plant Manager, Maintenance
Member:	Technical Support Manager
Member:	Chemistry and Health Physics General Superintendent
Member:	Operations Engineering Supervisor (Plant)
Member:	An Engineering Director or Performance Engineering Manager
Member:	Operations Quality Assurance Manager
Member:	Operations Superintendent (Plant)

* Designated in writing by the Plant Manager. The Chairman will be drawn from SRB members.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRB activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SRB shall meet at least once per calendar month and as convened by the SRB Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SRB shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Review Board shall be responsible for:

a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.