March 14, 1988

Docket No. 50-346
Serial No. DB-87-017

Mr. Donald C. Shelton
Vice President, Nuclear
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Dear Mr. Shelton:

SUBJECT: AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. NPF-3: REMOTE SHUTDOWN INSTRUMENTATION--EXTENSION OF RANGE OF RCS PRESSURE INSTRUMENT (TAC NO. 66006)

The Commission has issued Amendment No. 110 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TSs) in response to your application dated September 29, 1987 (No. 1413).

This amendment revises the TSs to permit an extension of the range of the Reactor Coolant System (RCS) Pressure instrument on the Auxiliary Shutdown Panel to 3000 psig from 2500 psig. The change revises Specification 3.3.3.5, Table 3.3-9, Item 3 to have a range of 0 psig to 3000 psig.

General Design Criterion 13 (Appendix A to 10 CFR 50) includes a requirement that instrumentation be provided to monitor variables and systems over their anticipated ranges for accident conditions as appropriate to ensure adequate safety. The currently installed RCS pressure instrument has a range of 0 psig to 2500 psig as presently required by TS Section 3.3.3.5. However, the Updated Safety Analysis Report (USAR) documents transients which have calculated peak pressures which exceed 2500 psig.

Regulatory Guide (R.G.) 1.97 describes a method acceptable to the NRC for complying with the Commission's regulations (cf. General Design Criteria 13, 19, and 64) to provide instrumentation to monitor plant variables and systems during and following an accident. Table 3 in R.G. 1.97 recommends that RCS pressure instrumentation should have a minimum range of 0 psig to 3000 psig (for PWR designs other than CE) which should be adequate to indicate peak calculated accident pressures. To conform to the guidance of R.G. 1.97, the upper range of the RCS pressure instrument must be extended, and the TSs must be amended to permit the change.

The RCS pressure instrumentation covered by Specification 3.3.3.5 does not have any influence on automatic plant control or automatic safety action. Instead, the instrumentation only provides control room and auxiliary shutdown panel indication; however, the operators may rely upon the pressure indication to make decisions and take actions. Even though the expanded range of the

instrument will lower the instrument's resolution and accuracy, Toledo Edison Company has committed to review those procedures which require operator action based upon values from this instrument and to revise such values as necessary to ensure that the consequences of the actions are not affected by the decreased resolution and accuracy and that there will be no impact on safe operation of the plant.

We have reviewed your September 29, 1987 application for amendment. We find that your proposed change is consistent with the recommendations contained in R.G. 1.97, and, therefore, are acceptable.

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Albert W. De Agazio, Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosure:
Amendment No.110 to
License No. NPF-3

cc w/enclosure:
See next page

SICB

Do not issue until Notice period expires

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Date:	2/18/88	2/9/88	2/22/88	2/25/88

Mr. Donald C. Shelton
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Davis-Besse Nuclear Power Station
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated September 29, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 110, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than the end of the fifth refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 14, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 110

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove	Insert
3/4 3-44	3/4 3-44

REMOTE SHUTDOWN INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.5 The remote shutdown monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE with readouts displayed external to the control room.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

a. With the number of OPERABLE remote shutdown monitoring channels less than required by Table 3.3-9, either restore the inoperable channel to OPERABLE status within 30 days, or be in HOT SHUTDOWN within the next 12 hours.

b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.5 Each remote shutdown monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-6.

TABLE 3.3-9

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

INSTRUMENT	READOUT LOCATION	MEASUREMENT RANGE	MINIMUM CHANNELS OPERABLE
1. Reactor Trip Breaker Indication	(a) 480v F&DC CH. 2 Switchgear Room	OPEN-CLOSE	(a) 1 (Trip Breaker A)
	(b) 480v E&DC CH. 1 Switchgear Room		(b) 1 (Trip Breaker B)
	(c) 480v F&DC CH. 2 Switchgear Room		(c) 1 (Trip Breaker C)
	(d) CRDC Cabinet Room		(d) 1 (Trip Breaker D)
2. Reactor Coolant Temperature - Hot Leg	Aux. Shutdown Panel	520-620 °F	1
3. Reactor Coolant System Pressure	Aux. Shutdown Panel	0-3000 psig	1
4. Pressurizer Level	Aux. Shutdown Panel	0-320 inches	1
5. Steam Generator Outlet Steam Pressure	Aux. Shutdown Panel	0-1200 psig	1/steam generator
6. Steam Generator Level Startup Range	Aux. Shutdown Panel	0-250 inches	1/steam generator
7. Control Rod Position Limit Switches	Control Rod Drive Control Cabinets, System Logic Cabinet #4	0, 25, 50, 75 and 100%	1/rod