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DMB-016

Docket No. 50-346

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Mr. Richard P. Crouse  
Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza - Stop 712  
300 Madison Avenue  
Toledo, Ohio 43652

Dear Mr. Crouse:

SUBJECT: AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. NPF-3;  
POST-ACCIDENT MONITORING AND SAMPLING

The Commission has issued Amendment No. 84 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TSs) in response to a portion of Item 1 of your application dated May 5, 1982 (No. 803) and your application dated March 22, 1984, in its entirety. Item 2 of your May 5, 1982, application was incorporated into the license by Amendment 48; the remainder of Item 1 has been revised by your application of November 20, 1984 (No. 1102) and will be completed as a separate licensing action.

This amendment modifies Tables 3.3-10 and 4.3-10 relating to post-accident monitoring instrumentation by adding incore thermocouples, reactor coolant hot leg level, containment water level, and containment pressure to the list of post-accident instrumentation that must be operable and are subject to surveillance requirements. The amendment also adds TS Section 6.8.4.c which requires the establishment of a post-accident sampling program.

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P PDR

Mr. Richard P. Crouse

-2-

A copy of the Safety Evaluation supporting this amendment is enclosed. The Notice of Issuance will be included in the Commission's Monthly Notice in the Federal Register.

Sincerely,

~~Original signed by~~

Albert W. De Agazio, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 84 to NPF-3
2. Safety Evaluation

cc w/enclosures:  
See next page

ORB#4:DL  
RIngram  
2/28/85

ORB#4:DL *ana*  
A De Agazio;cr  
3/5/85

ORB#4:DL  
JStolz  
2/27/85

*See notice  
filed*  
2/8/85

AD:OR:DL  
GL:mas  
3/1/85

Toledo Edison Company

cc w/enclosure(s):

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated May 5, 1982, and March 22, 1984, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

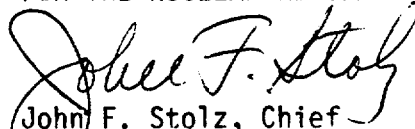
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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84 , are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 13, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

Pages

3/4 3-48a (New page)  
3/4 3-50a (New page)  
6-14a

TABLE 3.3-10 (Continued)  
POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>
31. Containment Normal Sump Level	1
32. Containment Wide Range Level	1
33. Containment Wide Range Pressure	1
34. Incore Thermocouples	2 per core quadrant
35. Reactor Coolant Hot Leg Level (Wide Range)	1

TABLE 4.3-10 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
31. Containment Normal Sump Level	M	R
32. Containment Wide Range Level	M	R
33. Containment Wide Range Pressure	M	R
34. Incore Thermocouples	M	R
35. Reactor Coolant Hot Leg Level (Wide Range)	M	R

DAVIS-BESSE, UNIT 1

3/4 3-50a

Amendment No. 84



## ADMINISTRATIVE CONTROLS

### 6.8.4 (Cont.)

#### c. Post-Accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for sampling and analysis,
- (iii) Provisions for maintenance of sampling and analysis equipment.

### 6.9 REPORTING REQUIREMENTS

#### ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the appropriate Regional Office unless otherwise noted.

#### STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the licensee involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

INTRODUCTION AND BACKGROUND

In November 1980, the NRC staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements", which includes all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications (TSs) were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of TSs for all of these items in Generic Letter 83-37. Generic Letter 83-37 was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's TSs to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of TSs were found.

By letters dated May 5, 1982, and March 22, 1984, Toledo Edison Company (the licensee) submitted proposed TSs which address the request made in Generic Letter 83-37. This evaluation covers the following TMI Action Plan items:

1. Post-Accident Sampling (II.B.3)
2. Containment Pressure Monitor (II.F.1.4)
3. Containment Water Level Monitor (II.F.1.5)
4. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

EVALUATION

1. Post-Accident Sampling (II.B.3)

The guidance provided by Generic Letter 83-37 requested that an administrative program be established, implemented and maintained to ensure that the licensee has the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions. The Post-Accident Sampling System is not required to be operable at all times. Administrative procedures are to be established for returning inoperable instruments to operable status as soon as practicable.

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The licensee has provided a proposed revision to the TSs which is consistent with the guidelines provided in our Generic Letter 83-37. We conclude that the licensee has an acceptable TS for the Post-Accident Sampling System.

2. Containment Pressure Monitor (II.F.1.4)

The Davis-Besse containment has been provided with two supplementary channels for monitoring containment pressure following an accident. The licensee has proposed TSs that are consistent with the guidelines contained in Generic Letter 83-37. We conclude that the proposed TSs for containment pressure monitor are acceptable.

3. Containment Water Level Monitor (II.F.1.5)

Narrow range and wide range containment water level monitors provide the capability required by TMI Action Plan Item II.F.1.5. The TSs for Davis-Besse contain limiting conditions for operation and surveillance requirements that are consistent with the guidance contained in Generic Letter 83-37. We conclude that the proposed TSs for containment water level monitors are acceptable.

4. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

Generic Letter 83-37 provided guidance on TSs for the subcooling margin monitors, a reactor coolant inventory tracking system and core exit thermocouples. The licensee had existing subcooling margin monitor TSs in place and submitted proposed TSs for the core exit thermocouples and inventory tracking system. We have reviewed the proposed TSs and conclude that they are acceptable as they meet the intent of our guidance contained in Generic Letter 83-37.

ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 13, 1985

The following NRC personnel contributed to this Safety Evaluation:  
Chandu Patel