

December 27, 1988

Docket No. 50-346  
Serial No. DB-88-008

Mr. Donald C. Shelton  
Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza - Stop 712  
300 Madison Avenue  
Toledo, Ohio 43652

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Dear Mr. Shelton:

SUBJECT: AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. NPF-3:  
DELETE CERTAIN CONTAINMENT ISOLATION VALVE CLOSURE TIMES  
(TAC NO. 57877)

The Commission has issued the enclosed Amendment No. 127 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TS's) in response to your application dated April 22, 1985 (Serial No. 1135) supplemented by letter dated July 31, 1986 (Serial No. 1265).

This amendment involves changes to TS Section 3/4.6.3, Containment Isolation Valves, Table 3.6-2, Containment Isolation Valves. This amendment revises the TS's to delete the isolation times specified in Table 3.6-2 for valves HP 2A, HP 2B, HP 2C, HP 2D, CS 1530, and CS 1531. Additionally, the amendment corrects typographical errors relating to valves CF 1542, CF 1544, and CF 1545, and deletes redundant listings for valves DH 87 and DH 88.

Toledo Edison Company asserts that the proposed deletion of the closure stroke times for the valves identified above is appropriate because these valves have no specific safety requirement for closure response time. These valves are in the high-pressure safety injection lines and containment spray lines; they receive a signal to open automatically to perform their safety functions of providing reactor coolant inventory and containment cooling in the event of a loss-of coolant accident. Closure of any of these valves is by manual operator action only, and, therefore, the closure time is not critical. Deletion of the stroke time requirement from Table 3.6-2, however, does not delete the requirement for the closure capability of these valves. This requirement is retained in Table 3.6-2.

The staff has reviewed the proposed deletions of response times for these valves and finds that the proposed deletions do not affect the reliability or operability of safety systems as required for safe plant operation, shutdown, or accident mitigation. Therefore, the proposed deletions are acceptable. The staff also has reviewed the proposed typographical corrections and finds the corrections acceptable.

8901040309 881227  
PDR ADDCK 05000346  
P PNU

DF01  
1/1  
CF  
J.A.

This amendment involves changes to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Albert W. De Agazio, Sr. Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III, IV,  
V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.127 to  
License No. NPF-3

cc w/enclosures:  
See next page

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Surname: PKreutzer  
Date: 12/13/88

PM/PDIII-3  
ADeAgazio/mr  
12/14/88

PD/PDIII-3  
JHannon  
12/18/88

OGC-WF1  
12/20/88

Mr. Donald C. Shelton  
Toledo Edison Company

Davis-Besse Nuclear Power Station  
Unit No. 1

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated April 22, 1985, supplemented July 31, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

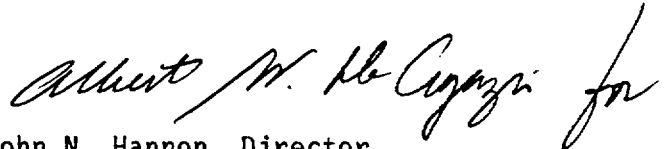
8901040314 881227  
PDR ADDCK 05000346  
P PNU

(a) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than February 10, 1989.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Albert W. McCarty for".

John N. Hannon, Director  
Project Directorate III-3  
Division of Reactor Projects - III, IV,  
V, & Special Projects  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: December 27, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 127

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove

3/4 6-17  
3/4 6-18  
3/4 6-19  
3/4 6-20  
3/4 6-21  
3/4 6-22

Insert

3/4 6-17  
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3/4 6-21  
3/4 6-22

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES (Continued)

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME</u> (seconds)
16	RC1719A	Containment Vessel Vent Header	10
16	RC1719B	Containment Vessel Vent Header	10
18 #	SS598	Steam Generator Secondary Water Sample Line	10
21	DW6831A	Demineralized Water Supply Line	10
21	DW6831B	Demineralized Water Supply Line	10
30 #	DH9A	Containment Sump Emergency Recirc Line	71
31 #	DH9B	Containment Sump Emergency Recirc Line	71
32	RC1773A	RCS Drain to RC Drain Tank	10
32	RC1773B	RCS Drain to RC Drain Tank	10

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES (Continued)

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (seconds)</u>
41	RC232	Pressurizer Quench Tank Circulating Inlet Line	10
42A	SA2010	Service Air Supply Line	10
42B	CV5010E	Containment Vessel Air Sample Return	15
43A	IA2011	Instrument Air Supply Line	10
43B	CV5011E	Containment Vessel Air Sample Return	15
44A	CF1541	Core Flood Tank Fill and N2 Supply Line	10
44B	NN236	Pressurizer Quench Tank N2 Supply Line	10
47A	CF1545	Core Flood Tank Sample Line	10
47B	CF1542	Core Flood Tank Vent Line	10
48	RC229A	Pressurizer Quench Tank Circulating Outlet Line	10
48	RC229B	Pressurizer Quench Tank Circulating Outlet Line	10
51	CV5037	Hydrogen Purge System Exhaust Line	60
51	CV5038	Hydrogen Purge System Exhaust Line	60
52	MU66A	Reactor Coolant Pump Seal Supply	12
53	MU66B	Reactor Coolant Pump Seal Supply	12
54	MU66C	Reactor Coolant Pump Seal Supply	12
55	MU66D	Reactor Coolant Pump Seal Supply	12
56	MU38	Reactor Coolant Pump Seal Return	12
56	MU59A	Reactor Coolant Pump Seal Return	30
56	MU59B	Reactor Coolant Pump Seal Return	30
56	MU59C	Reactor Coolant Pump Seal Return	30
56	MU59D	Reactor Coolant Pump Seal Return	30



TABLE 3.6-2

CONTAINMENT ISOLATION VALVES (Continued)

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (seconds)</u>
67	CV5090	Hydrogen Dilution System Supply	60
68A	SS235A	Pressurizer Quench Tank Sample	30
68A	SS235B	Pressurizer Quench Tank Sample	30
68B	CV5010B	Containment Air Sample	15
68B	CV5011B	Containment Air Sample	15
69	CV5065	Hydrogen Dilution System Supply	60
71B	CV5010A	Containment Air Sample	15
71B	CV5011A	Containment Air Sample	15
71C	CF1544	Core Flood Tank N2 Fill	10
73B	CV5010C	Containment Air Sample	15
73B	CV5011C	Containment Air Sample	15
74B	CV5010D	Containment Air Sample	15
74B	CV5011D	Containment Air Sample	15

## B. CONTAINMENT PURGE AND EXHAUST ISOLATION

33 ##	CV5005	Containment Vessel Purge Inlet Line	10
33 ##	CV5006	Containment Vessel Purge Inlet Line	10
34 ##	CV5007	Containment Vessel Purge Outlet Line	10
34 ##	CV5008	Containment Vessel Purge Outlet Line	10

## C. OTHER

5 #	SW1366	Containment Air Cooling Units SW Inlet Line	N/A
6 #	SW1368	Containment Air Cooling Units SW Inlet Line	N/A
7 #	SW1367	Containment Air Cooling Units SW Inlet Line	N/A
9 #	SW1356	Containment Air Cooling Units SW Outlet Line	N/A

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES (Continued)

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (seconds)</u>
10 #	SW1358	Containment Air Cooling Units SW Outlet Line	N/A
11 #	SW1357	Containment Air Cooling Units SW Outlet Line	N/A
17	CV343	Containment Vessel Leak Test Inlet Line	N/A
17	Flange	Containment Vessel Leak Test Inlet Line (Inside Containment)	N/A
19	MU6422	Normal RCS Makeup Line	N/A
19 #	HP57	High Pressure Injection Line	N/A
19 #	HP2A	High Pressure Injection Line	N/A
20 #	HP56	High Pressure Injection Line	N/A
20 #	HP2B	High Pressure Injection Line	N/A
22 #	HP49	High Pressure Injection Line	N/A
22 #	HP2D	High Pressure Injection Line	N/A
23 #	SF1	Fuel Transfer Tube	N/A
23	Flange	Fuel Transfer Tube	N/A
24 #	SF2	Fuel Transfer Tube	N/A
24	Flange	Fuel Transfer Tube	N/A
*25	CS33	Containment Spray Line	N/A
*25	CS17	Containment Spray Line	N/A
25	SA536	Containment Spray Line	N/A
25	SA532	Containment Spray Line	N/A
25	CS1531	Containment Spray Line	N/A
26	CS1530	Containment Spray Line	N/A
*26	CS36	Containment Spray Line	N/A
*26	CS18	Containment Spray Line	N/A
26	SA535	Containment Spray Line	N/A
26	SA533	Containment Spray Line	N/A
27 #	DH1A	Low Pressure Injection Line	N/A
27 #	DH76	Low Pressure Injection Line	N/A
28 #	DH1B	Low Pressure Injection Line	N/A
28 #	DH77	Low Pressure Injection Line	N/A

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES (Continued)

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (seconds)</u>
*29 #	DH11	Decay Heat Pump Suction Line	N/A
*29	DH23	Decay Heat Pump Suction Line	N/A
29 #	PSV4849	Decay Heat Pump Suction Line	N/A
35 #	AF599	Auxiliary Feedwater Line	N/A
36 #	AF608	Auxiliary Feedwater Line	N/A
37 #	FW601	Main Feedwater Line	N/A
38 #	FW612	Main Feedwater Line	N/A
**39 #	MS100	Main Steam Line	N/A
**39 #	ICS11A	Main Steam Line	N/A
39 #	MS375	Main Steam Line	N/A
39 #	MS100-1	Main Steam Line	N/A
*39 #	MS107	Main Steam Line	N/A
*39 #	MS107A	Main Steam Line	N/A
*40 #	MS106	Main Steam Line	N/A
*40 #	MS106A	Main Steam Line	N/A
**40 #	MS101	Main Steam Line	N/A
**40 #	ICS11B	Main Steam Line	N/A
40 #	MS394	Main Steam Line	N/A
40 #	MS101-1	Main Steam Line	N/A
41	RC113	Pressurizer Quench Tank Inlet Line	N/A
42A	SA502	Service Air Supply Line	N/A
42B	CV124	Containment Vessel Air Sample Return	N/A
43A	IA501	Service Air Supply Line	N/A
43B	CV125	Containment Vessel Air Sample Return	N/A
44A	CF15	Core Flood Tank Fill and Nitrogen Supply Line	N/A
44B	NN58	Pressurizer Quench Tank Inlet Line	N/A
*47A	CF2A	Core Flood Tank Sample Line	N/A
*47A	CF2B	Core Flood Tank Sample Line	N/A
*47B	CF5A	Core Flood Tank Vent Line	N/A
*47B	CF5B	Core Flood Tank Vent Line	N/A

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES (Continued)

<u>PENETRATION NUMBER</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (seconds)</u>
49	DH87	Refueling Canal Fill Line	N/A
49	DH88	Refueling Canal Fill Line	N/A
50 #	HP48	High Pressure Injection Line	N/A
50 #	HP2C	High Pressure Injection Line	N/A
50	MU6421	RCS Makeup Line	N/A
52	MU242	RCP Seal Water Supply	N/A
53	MU243	RCP Seal Water Supply	N/A
54	MU244	RCP Seal Water Supply	N/A
55	MU245	RCP Seal Water Supply	N/A
59	Flange	Secondary Side Cleaning (Inside Containment)	N/A
59	Flange	Secondary Side Cleaning (Outside Containment)	N/A
57 #	MS603	Steam Generator Blowdown Line	N/A
60 #	MS611	Steam Generator Blowdown Line	N/A
67	CV209	Hydrogen Dilution System Supply	N/A
69	CV210	Hydrogen Dilution System Supply	N/A
71A #	CV2000B	Containment Pressure Sensor	N/A
71C	CF16	Core Flood Tank Nitrogen Fill Line	N/A
72A #	CV2001B	Containment Pressure Sensor	N/A
72C #	CV624B	Containment Pressure Differential Transmitter	N/A
73A #	CV2002B	Containment Pressure Sensor	N/A
73C #	CV645B	Containment Pressure Differential Transmitter	N/A
74A #	CV2003B	Containment Pressure Sensor	N/A
*74C	DH2735	Pressurizer Auxiliary Spray	N/A
*74C	DH2736	Pressurizer Auxiliary Spray	N/A

\*May be opened on an intermittent basis under administrative control.

#Not subject to Type C leakage tests.

\*\*Surveillance testing not required prior to entering MODE 4 but shall be performed prior to entering Mode 3.

##Provisions of Specification 3.0.4 are not applicable provided the valve is in the closed positions and deactivated.