September 27, 1988

Docket No. 50-346
Serial No. DB-88-046

Mr. Donald C. Shelton
Vice President, Nuclear
Toledo Edison Company
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Dear Mr. Shelton:

SUBJECT: AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. NPF-3: STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM PRESSURE SWITCHES (TAC 66728)

The Commission has issued the enclosed Amendment No. 121 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TS's) in response to your application dated January 28, 1988 (No. 1463).

This amendment revises the TS's relating to the number of pressure switches used to initiate the Steam and Feedwater Rupture Control System (SFRCS). The amendment, specifically, revises Technical Specification 3.3.2.2, Table 3.3-11, "Steam and Feedwater Rupture Control System," by reducing the number of pressure switches used to initiate the SFRCS from 16 to 8. Currently, two pressure switches per steam line provide input to each logic channel. Actuation of either of these pressure switches will trip that logic channel. There are two redundant logic channels for each independent SFRCS actuation channel. The amendment reduces the number of pressure switches so that only one pressure switch per steam line provides input to each logic channel. The TS requirements for the total number of channels, channels to trip, minimum channels operable, and required action remain unchanged.

The purpose of the low main-steam-line pressure switch is to provide a means of detecting a rupture in the main steam or feedwater piping. When SFRCS is actuated in response to a low steam-line-pressure signal, the station turbine is tripped and main steam and main feedwater for both steam generators are isolated. The auxiliary feedwater train for the affected steam generator is realigned to the unaffected steam generator. The reduction from 16 to 8 pressure switches reduces the probability of spurious SFRCS actuation due to unnecessary redundancy, therefore reducing the number of significant transients.

The staff has reviewed the proposed change and finds it acceptable on the basis that it is desirable to minimize the number of pressure switches while maintaining a sufficient number to meet the single-failure criterion as specified in IEEE Std. 279-1971.

D/01

C/P1

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant has been prepared and published in the Federal Register (July 26, 1988, (53 FR 28082). Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of this amendment will not have a significant effect on the quality of the human environment.

The staff has concluded, based on the considerations discussed above, that : (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the notice of issuance is enclosed. The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

/s/

Albert W. De Agazio, Sr.Project Manager
Project Directorate III-3 Division of
Reactor Projects - III, IV, V & Special
Projects

Enclosures: 1. Amendment No.121to
License No. NPF-3
2. Federal Register notice

cc w/enclosures: See next page

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Date:	9/13/88	9/13/88	9/14/88	9/22/88

Mr. Donald C. Shelton
Toledo Edison Company

Davis-Besse Nuclear Power Station
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISION COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated January 28, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

(a) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 121 , are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than November 10, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects - III, IV, V, & Special Projects

Attachment: Changes to the Technical Specifications

Date of Issuance: September 27, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove	Insert
3/4 3-24	3/4 3-24

INSTRUMENTATION

STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.2.2 The Steam and Feedwater Rupture Control System (SFRCS) instrumentation channels shown in Table 3.3-11 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3-12 and with RESPONSE TIMES as shown in Table 3.3-13.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

a. With a SFRCS instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3-12, declare the channel inoperable and apply the applicable ACTION requirement of Table 3.3-11, until the channel is restored to OPERABLE status with the trip setpoint adjusted consistent with the Trip Setpoint value.

b. With a SFRCS instrumentation channel inoperable, take the action shown in Table 3.3-11.

SURVEILLANCE REQUIREMENTS

4.3.2.2.1 Each SFRCS instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST during the MODES and at the frequencies shown in Table 4.3-11.

4.3.2.2.2 The logic for the bypasses shall be demonstrated OPERABLE during the at power CHANNEL FUNCTIONAL TEST of channels affected by bypass operation. The total bypass function shall be demonstrated OPERABLE at least once per 18 months during CHANNEL CALIBRATION testing of each channel affected by bypass operation.

4.3.2.2.3 The STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM RESPONSE TIME of each SFRCS function shall be demonstrated to be within the limit at least once per 18 months. Each test shall include at least one channel per function such that all channels are tested at least once every N times 18 months where N is the total number of redundant channels in a specific SFRCS function as shown in the "Total No. of Channels" Column of Table 3.3-11.

TABLE 3.3-11

STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM INSTRUMENTATION

FUNCTIONAL UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	ACTION
1. Main Steam Pressure Low Instrument Channels*	2	1	2	13#
a. PS 3689B Steam Line 1 Channel 1				
b. PS 3689D Steam Line 2 Channel 1				
c. PS 3689F Steam Line 1 Channel 1				
d. PS 3689H Steam Line 2 Channel 1				
e. PS 3687A Steam Line 2 Channel 2				
f. PS 3687C Steam Line 1 Channel 2				
g. PS 3687E Steam Line 2 Channel 2				
h. PS 3687G Steam Line 1 Channel 2				

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U. S. NUCLEAR REGULATORY COMMISSION

TOLEDO EDISON COMPANY, ET AL.

DOCKET NO. 50-346

NOTICE OF ISSUANCE OF AMENDMENT TO

FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 121 to Facility Operating License No. NPF-3, issued to The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensee), which revised the Technical Specifications for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment was effective as of the date of its issuance.

The amendment revised the Technical Specifications by reducing the number of pressure switches used to initiate the Steam and Feedwater Rupture Control System from 16 to 8.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER

on May 24, 1988 (53 FR 18630). No request for hearing or petition for leave to intervene was filed following this notice.

For further details with respect to this action see (1) the application for amendment dated January 28, 1988 (2) Amendment No. 121 to License No. NPF-3, (3) the Commission's related Safety Evaluation dated September 27, 1988 and (4) the Environmental Assessment dated July 19, 1988 (53 FR 28082). All of these items are available for public inspection at the Commission's Public Document Room, 2120 L Street, N.W., Washington, D.C., and at the University of Toledo Library, Documents Department, 2801 Bancroft Avenue, Toledo, Ohio 43606.

A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Projects - III, IV, V and Special Projects.

Dated at Rockville, Maryland this 27th day of September 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

Albert W. De Agazio, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects