

May 31, 1988

Docket No. 50-346

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Dear Mr. Shelton:

On March 9, 1988, the Commission issued Amendment No. 109 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1, in response to your application dated January 22, 1988, as clarified by later correspondence. The amendment consisted of changes to the Technical Specifications to revise the responsibilities and authority of the Station Review Board and added a section on Technical Review and Control.

Technical Specification page 6-6 issued with Amendment No. 109 had back-up page 6-5 which was not changed by Amendment No. 109. However, page 6-5 had recently been changed by Amendment No. 106. Inadvertently, the change made by Amendment 106 was overlooked, and the page was issued without incorporating the Amendment 106 revisions.

Enclosed is another copy of page 6-6, as revised by Amendment No. 109, with the corrected page 6-5 as revised by Amendment No. 106 printed on the back.

Thank you for calling this matter to our attention. Please accept our apologies for any inconvenience this administrative error may have caused you.

Sincerely,

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Albert W. De Agazio, Sr. Project Manager  
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Enclosure:  
TS Page 6-6

cc: See next page

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Davis-Besse Nuclear Power Station  
Unit No. 1

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## ADMINISTRATIVE CONTROLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Chemistry and Health Physics General Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Director.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION REVIEW BOARD (SRB)

##### FUNCTION

6.5.1.1 The Station Review Board (SRB) shall function to advise the Plant Manager on all matters related to nuclear safety.

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## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.1.2 The Station Review Board shall be composed of the:

Chairman:	Station Review Board Chairman*
Member:	Assistant Plant Manager, Operations
Member:	Assistant Plant Manager, Maintenance
Member:	Technical Support Manager
Member:	Chemistry and Health Physics General Superintendent
Member:	Operations Engineering Supervisor (Plant)
Member:	An Engineering Director or Performance Engineering Manager
Member:	Operations Quality Assurance Manager
Member:	Operations Superintendent (Plant)

\* Designated in writing by the Plant Manager. The Chairman will be drawn from SRB members.

### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SRB Chairman; however, no more than two alternates shall participate as voting members in SRB activities at any one time.

### MEETING FREQUENCY

6.5.1.4 The SRB shall meet at least once per calendar month and as convened by the SRB Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SRB shall consist of the Chairman or his designated alternate and four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The Station Review Board shall be responsible for:

- a. Review of plant administrative procedures and changes thereto.
- b. Review of the safety evaluation for 1) procedures, 2) changes to procedures, equipment or systems and 3) tests or experiments completed under the provisions of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question.
- c. Review of proposed procedures and changes to procedures and equipment determined to involve an unreviewed safety question as defined in 10 CFR 50.59.