

April 3, 1985

DMB 016

Docket No. 50-346

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Mr. Richard P. Crouse
Vice President, Nuclear
Toledo Edison Company
Edison Plaza - Stop 712
300 Madison Avenue
Toledo, Ohio 43652

Dear Mr. Crouse:

SUBJECT: AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-3;
RCS HIGH POINT VENTS

The Commission has issued Amendment No. 85 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TSs) in response your application dated November 20, 1984 (No. 1102). This application supersedes a portion of Item 1 of your application dated May 5, 1982 (No. 803) which had not been processed previously. The remainder of your May 5, 1982 application has been processed as a separate licensing action.

This amendment adds Limiting Condition for Operation, Section 3.4.11, and Surveillance Requirements, Section 4.4.11, which relate to the reactor coolant system hot leg high point vents and pressurizer vent. In addition, Bases Section 3/4.4.11 for high point vents has been added.

A copy of the Safety Evaluation supporting this amendment is enclosed. The Notice of Issuance will be included in the Commission's Monthly Notice in the Federal Register.

Sincerely,

"ORIGINAL SIGNED BY:"

Albert W. De Agazio, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No.85 to NPF-3
2. Safety Evaluation

cc w/enclosures:

See next page

ORB#4:DL
RIngram
3/4/85

ORB#4:DL
ADe Agazio;cf
3/5/85

ORB#4:DL
JStoltz
3/5/85

OELD
M. KARMA
3/25/85

AD:DL
GLainas
3/29/85

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Toledo Edison Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated November 20, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

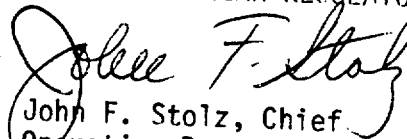
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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 3, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 4-32
B 3/4 4-13

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. Each internals vent valve shall be demonstrated OPERABLE at least once per 18 months during shutdown,* by:
 - 1. Verifying through visual inspection that the valve body and valve disc exhibit no abnormal degradation,
 - 2. Verifying the valve is not stuck in an open position, and
 - 3. Verifying through manual actuation that the valve is fully open when a force of ≤ 400 lbs. is applied vertically upward.

*Performance of this Surveillance Requirement may be deferred until May 30, 1980 to coincide with the first refueling outage.

REACTOR COOLANT SYSTEM
REACTOR COOLANT SYSTEM VENTS
LIMITING CONDITION FOR OPERATION

3.4.11 The following reactor coolant system vent paths shall be operable:

- a. Reactor Coolant System Loop 1 with vent path through valves RC 4608A and RC 4608B.
- b. Reactor Coolant System Loop 2 with vent path through valves RC 4610A and RC 4610B.
- c. Pressurizer; with vent path through EITHER valves RC11 and RC2 (PORV) OR valves RC 239A and RC 200.

APPLICABILITY: Modes 1, 2 and 3

ACTION:

- a. With one of the above vent paths inoperable, restore the inoperable vent path to OPERABLE status within 30 days, or, be in HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 30 hours.
- b. With two of the above vent paths inoperable, restore at least one of the inoperable vent paths to OPERABLE status within 72 hours or be in HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 30 hours.
- c. With three of the above vent paths inoperable, restore at least two of the inoperable vent paths to OPERABLE status within 72 hours or be in HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 30 hours.
- d. The provisions of specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.11 Each reactor coolant system vent path shall be demonstrated OPERABLE at least once per 18 months by:

1. Verifying all manual isolation valves in each vent path are locked in the open position, and
2. Cycling each valve in the vent path through at least one complete cycle of full travel from the control room during COLD SHUTDOWN or REFUELING, and
3. Verifying flow through the reactor coolant vent system vent paths during COLD SHUTDOWN or REFUELING.

REACTOR COOLANT SYSTEM

BASES

3/4.4.10 STRUCTURAL INTEGRITY

The inspection programs for ASME Code Class 1, 2 and 3 components, except steam generator tubes, ensure that the structural integrity of these components will be maintained at an acceptable level throughout the life of the plant. To the extent applicable, the inspection program for these components is in compliance with Section XI of the ASME Boiler and Pressure Vessel Code.

The internals vent valves are provided to relieve the pressure generated by steaming in the core following a LOCA so that the core remains sufficiently covered. Inspection and manual actuation of the internals vent valves 1) ensure OPERABILITY, 2) ensure that the valves are not stuck open during normal operation, and 3) demonstrates that the valves are fully open at the forces equivalent to the differential pressures assumed in the safety analysis.

3/4.4.11 HIGH POINT VENTS

The Reactor Coolant System high point vents are installed per NUREG-0737 item II.B.1 requirements. The operability of the system ensures capability of venting steam or noncondensable gas bubbles in the reactor cooling system to restore natural circulation following a small break loss of coolant accident.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

INTRODUCTION AND BACKGROUND

In November 1980, the NRC staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which includes all TMI Action Plan items approved by the Commission for implementation at nuclear power plants. NUREG-0737 identifies those items for which Technical Specifications (TSs) are required. A number of items which require TSs were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of TSs for all of these items in Generic Letter 83-37. Generic Letter 83-37 was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's TSs to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of TSs were found.

By letter dated November 20, 1984, Toledo Edison Company (the licensee) responded to Generic Letter 83-37 by submitting a TS change request for Davis-Besse Unit 1. This evaluation covers TMI Action Plan Item II.B.1 - Reactor Coolant System (RCS) Vents.

EVALUATION

Our guidance for RCS Vents identified the need for at least one operable vent path at the reactor vessel head, the pressurizer steam space, and RCS high points for Babcock and Wilcox reactors. Generic Letter 83-37 also provided limiting conditions for operation and the surveillance requirements for the RCS vents.

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The licensee has been granted a temporary exemption from the requirement of the reactor vessel head vent specified in 10 CFR 50.44(c)(3)(iii). For the remaining vent paths, the licensee has proposed TSs that are consistent with our guidance contained in Generic Letter 83-37. Therefore, we conclude that the proposed TSs are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a change in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 3, 1985

The following NRC personnel contributed to this Safety Evaluation:
C. Patel.