

August 22, 1985

DMB 016

Docket No. 50-346

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Mr. Joe Williams, Jr.  
Senior Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza - Stop 712  
300 Madison Avenue  
Toledo, Ohio 43652

Dear Mr. Williams:

SUBJECT: AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. NPF-3;  
REPORTING OF CHALLENGES TO PORV AND SAFETY VALVES

The Commission has issued Amendment No. 87 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of a change to the Appendix A Technical Specifications (TSs) in response to your application dated September 17, 1984 (No. 1082).

This amendment adds to Specification 6.9.1.6 the requirement to report in the monthly operating report challenges to the pressurizer power operated relief valve (PORV) and the pressurizer code safety valves.

On May 7, 1980, the NRC sent a letter to all operating reactor licensees informing them of five additional TMI-2 related requirements including a requirement that Item II.K.3.3 be implemented by January 1, 1981. No model Technical Specifications were included. In its response to this letter dated June 26, 1980 (No. 624), Toledo Edison committed to report challenges of reactor coolant safety and relief valves in its monthly operating reports. Failures of such valves are reported in accordance with Section 6.9.1.8.a of the Technical Specifications.

On September 20, 1982, the Commission issued Generic Letter No. 82-16 to all pressurized water power reactor licensees. This letter discussed a number of items from NUREG-0737 which were required to be implemented by December 31, 1981, and which would require Technical Specifications to assure that facility operation is maintained within the limits determined acceptable following implementation at each facility. Generic Letter 82-16 also included guidance on the scope of a specification which would be acceptable to the NRC staff. Examples in Standard Technical Specification format were included. One of the items covered in Generic Letter 82-16 was Item II.K.3.3.

Toledo Edison responded (November 15, 1982 (No. 873)) by reiterating its previous commitment and stating its belief that technical specifications were not required. The NRC staff did not agree with this position and in the Safety Evaluation supporting Amendment No. 73 issued July 25, 1984, stated that if Toledo Edison chose to report safety valve (SV) to relief valve challenges in the monthly report, Section 6.9.1.6 of the Technical Specifications should be revised to include this requirement.

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Mr. Joe Williams, Jr.

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We have reviewed your proposed change to Technical Specification Section 6.9.1.6 and find that it is consistent with guidance contained in Generic Letter 82-16 and our Safety Evaluation supporting Amendment No. 73 and is, therefore, acceptable.

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

The Notice of Issuance will be included in the Commission's Biweekly Notice in the Federal Register.

Sincerely,

**"ORIGINAL SIGNED COPY"**

Albert W. De Agazio, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Amendment No. 87 to NPF-3

cc w/enclosure:  
See next page

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A De Agazio;cf  
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JStalz  
8/9/85

OELD  
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8/20/85

Mr. J. Williams  
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Davis-Besse Nuclear Power Station  
Unit No. 1

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated September 17, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

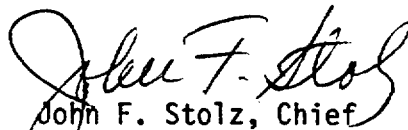
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Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 22, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 87

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

6-15

## ADMINISTRATIVE CONTROLS

power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

### ANNUAL OPERATING REPORT<sup>1/</sup>

6.9.1.4 Annual reports covering the activities of the unit during the previous calendar year shall be submitted prior to March 31 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,<sup>2/</sup> e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The complete results of steam generator tube inservice inspections (Specification 4.4.5.5.b).

### MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics, shutdown experience and challenges to the Pressurizer Power Operated Relief Valve (PORV) and the Pressurizer Code Safety Valves shall be submitted on a monthly basis to the Director, Office of Management and Program Analysis, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, to arrive no later than the 15th of each month following the calendar month covered by the report.

<sup>1/</sup> A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

<sup>2/</sup> This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

## ADMINISTRATIVE CONTROLS

### REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

### PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.
- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
- c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.
- d. Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation greater than or equal to 1%  $\Delta k/k$ ; a calculated reactivity balance indicating a SHUTDOWN MARGIN less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 0.5%  $\Delta k/k$ ; or occurrence of any unplanned criticality.