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Docket No. 50-346

Mr. Richard P. Crouse  
Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

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U.S. NRC SERVICES  
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RECEIVED DISTRIBUTION SERVICES UNIT

Dear Mr. Crouse:

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1, in response to your submittal of October 7, 1980. The amendment modifies the Technical Specifications to reflect modifications of the Fire Detection Instrumentation recently completed at the facility. These Technical Specification revisions delete the old Fire Detection System and incorporate the new Fire Detection System in the Limiting Conditions for Operation and the Surveillance Requirements.

Your submittal was in partial response to our letter dated September 23, 1980, in which we transmitted a request that you propose necessary revisions to your Technical Specifications to reflect plant modifications that were being completed in accordance with our Fire Protection Safety Evaluation Report (SER) dated July 26, 1979. We have reviewed your proposed Technical Specification changes and conclude that the revisions are in accordance with our Fire Protection SER and with your Fire Hazards Analysis Report. These modifications represent an increased effectiveness of your Fire Detection System. On this basis, we find the proposed Technical Specification changes are acceptable.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, and negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

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Mr. Richard P. Crouse-2

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by this action, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

Original signed by  
Robert W. Reid

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 34 to NPF-3
2. Notice

cc w/enclosures:  
See next page

*Concern in Fed. Reg. Notice  
and amendment, subject  
to revision of notice per  
change noted thereon.  
RWR*

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD-OR:DL	OELD	
SURNAME	RIngram	BGarnier/cab	RReid	TNovak	M. Rothschild	T.V. Wambach
DATE	10/31/80	10/31/80	10/31/80	10/31/80	10/07/80	11/14/80



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

November 18, 1980

Docket No. 50-346

Mr. Richard P. Crouse  
Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

Dear Mr. Crouse:

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1, in response to your submittal of October 7, 1980. The amendment modifies the Technical Specifications to reflect modifications of the Fire Detection Instrumentation recently completed at the facility. These Technical Specification revisions delete the old Fire Detection System and incorporate the new Fire Detection System in the Limiting Conditions for Operation and the Surveillance Requirements.

Your submittal was in partial response to our letter dated September 23, 1980, in which we transmitted a request that you propose necessary revisions to your Technical Specifications to reflect plant modifications that were being completed in accordance with our Fire Protection Safety Evaluation Report (SER) dated July 26, 1979. We have reviewed your proposed Technical Specification changes and conclude that the revisions are in accordance with our Fire Protection SER and with your Fire Hazards Analysis Report. These modifications represent an increased effectiveness of your Fire Detection System. On this basis, we find the proposed Technical Specification changes are acceptable.

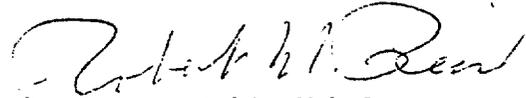
We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Mr. Richard P. Crouse-2

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by this action, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the Notice of Issuance is also enclosed.

Sincerely,



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosures:

1. Amendment No. 34 to NPF-3
2. Notice

cc w/enclosures:  
See next page

Toledo Edison Company

cc w/enclosure(s):

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Nuclear Power Generation Division  
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Bethesda, Maryland 20014

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Port Clinton, Ohio 43452

President, Board of County  
Commissioners of Ottawa County  
Port Clinton, Ohio 43452

Attorney General  
Department of Attorney General  
30 East Broad Street  
Columbus, Ohio 43215

Harold Kahn, Staff Scientist  
Power Siting Commission  
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Columbus, Ohio 43216

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Industrial Commission  
State of Ohio  
2323 West 5th Avenue  
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Mr. Ted Myers  
Manager, Nuclear Licensing  
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Edison Plaza  
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U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
5503 N. State Route 2  
Oak Harbor, Ohio 43449

Director, Criteria and Standards  
Division  
Office of Radiation Programs (ANR-460)  
U. S. Environmental Protection Agency  
Washington, D. C. 20460

U. S. Environmental Protection Agency  
Federal Activities Branch  
Region V Office  
ATTN: EIS COORDINATOR  
230 South Dearborn Street  
Chicago, Illinois 60604

cc w/enclosure(s) and incoming dtd.:  
October 7, 1980

Ohio Department of Health  
ATTN: Director of Health  
450 East Town Street  
Columbus, Ohio 43216



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated October 7, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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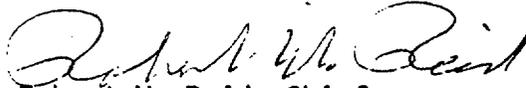
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 34, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 13, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-52

3/4 3-53

3/4 3-54

3/4 3-55

3/4 3-56

## INSTRUMENTATION

### CHLORINE DETECTION SYSTEMS

#### LIMITING CONDITION FOR OPERATION

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3.3.3.7 The chlorine detection system, with the alarm/trip setpoint adjusted to actuate at a chlorine concentration of  $\leq 5$  ppm, shall be OPERABLE with at least two OPERABLE chlorine detectors located in the Reactor Control Room ventilation air intake.

APPLICABILITY: 1, 2, 3 and 4

#### ACTION:

- a. With one chlorine detector or the chlorine detection system inoperable, within 1 hour initiate and maintain operation of the control room ventilation system in the recirculation mode of operation; restore the inoperable detection system or detector to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

---

4.3.3.7 The chlorine detection system shall be demonstrated OPERABLE by performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days, and a CHANNEL CALIBRATION at least once per 18 months.

## INSTRUMENTATION

### FIRE DETECTION INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

---

3.3.3.8 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-14 shall be OPERABLE.

APPLICABILITY: Whenever equipment in that fire detection zone is required to be OPERABLE.

ACTION: With the number of OPERABLE fire detection instrument(s) less than the minimum number OPERABLE requirement of Table 3.3-14:

- a. Within 1 hour establish a fire watch patrol to inspect the accessible zone(s) with the inoperable instrument(s) at least once per hour, and
- b. Restore the inoperable instrument(s) to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the instrument(s) to OPERABLE status.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

---

4.3.3.8.1 Each of the above required accessible fire detection instruments shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST. Each of the above required inaccessible fire detection instrument shall be demonstrated OPERABLE at least once per 18 months by performance of a CHANNEL FUNCTIONAL TEST.

4.3.3.8.2 The NFPA Code 72D Class A supervised circuits supervision associated with the detector alarms of each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.

4.3.3.8.3 The non-supervised circuits between the local panels in Specification 4.3.3.8.3 and the control room shall be demonstrated OPERABLE at least once per 31 days.

FIRE DETECTION INSTRUMENTS

<u>INSTRUMENT LOCATION</u>	<u>MINIMUM INSTRUMENTS OPERABLE</u>		
	<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
<u>1. Containment</u>			
#a. FDZ-RCP 1 Elev. 603'	0	0	1 *
#b. FDZ-RCP 2 Elev. 603'	0	0	1 *
#c. FDZ-RCP 3 Elev. 603'	0	0	1 *
#d. FDZ-RCP 4 Elev. 603'	0	0	1 *
#e. FDZ-PZR Elev. 603'	0	0	1 *
#f. FDZ 214 - Core Flooding Tank 1-1 Area Elev. 565'	0	0	3 *
#g. FDZ 215 - Cmtt. Letdown Cooler Area Elev. 565'	0	0	2 *
#h. FDZ 220 - Incore Instrument Trench Area Elev. 565'	0	0	4 *
#i. FDZ 317 - Hatch Area - Elev. 565'	0	0	20 *
#j. FDZ 410 - East Passage - Elev. 603'/657'	0	0	9 *
<u>2. Containment Annulus</u>			
#a. FDZ-A208 Elev. 590'	0	0	10
#b. FDZ-236H Elev. 774'	0	0	3
#c. FDZ-236L Elev. 590'	0	0	9
<u>3. Auxiliary Building</u>			
a. FDZ 402 - #1 Electrical Penetration Rm. Elev. 603'	0	0	12
b. FDZ 405 - Auxiliary Building Storage Rm. Elev. 603'	0	0	1
c. FDZ 427 - #2 Electrical Penetration Rm. Elev. 603'	0	0	7
d. FDZ 303 - #3 Mechanical Penetration Rm. Elev. 585'	0	0	12
e. FDZ 304 - Corridor to Mech. Pent Rms 3&4 Elev. 585'	0	0	4
f. FDZ 310 - Passage to BA Mix Tank Elev. 585'	0	0	8
g. FDZ 312 - Spent Fuel Pool Pump Rm. Elev. 585'	0	0	4
h. FDZ 314 - #4 Mech. Pent. Room Elev. 585'	0	0	17
i. FDZ 300 - Fuel Handling Area Elev. 585'	0	0	5

#Fire Detectors in high radiation areas which are NOT accessible.

<u>INSTRUMENT LOCATION</u>	<u>F</u>	<u>INSTRUMENTS OPERABLE</u>	
		<u>FLAME</u>	<u>SMOKE</u>
<b>3. <u>Auxiliary Building (Continued)</u></b>			
j. FDZ 209 - Corridor to #1 Mech. Pent. Rm. Elev. 565'	0	0	3
k. FDZ 227 - Boric Acid Evap Passageway Elev. 565'	0	0	6
l. FDZ 208 - #1 Mechanical Penetration Rm. Elev. 565'	0	0	10
m. FDZ 231 - Clean Waste Booster Pump Rm. Elev. 565'	0	0	1
n. FDZ 232 - Primary & Deborating Demin Vlv Rm. - Elev. 565'	0	0	1
o. FDZ 234 - Boric Acid Evaporator Rm 1-2 Elev. 565'	0	0	1
p. FDZ 235 - Boric Acid Evaporator Rm 1-1 Elev. 565'	0	0	1
q. FDZ 236 - #2 Mechanical Penetration Rm. Elev. 565'	0	0	4
r. FDZ 240 - Boric Acid Addition Tank Rm. Elev. 565'	0	0	5
s. FDZ 241 - Passage to B.A. Addition Tk Rm. Elev. 565'	0	0	2
t. FDZ 101 - Equipment and Pipe Tunnel Elev. 545'	0	0	1
u. FDZ 105 - ECCS Pump Room 1-1 Elev. 545'	0	0	4
v. FDZ 110 - Corridor to Central Area of Aux Bldg. - Elev. 545'	0	0	5
w. FDZ 113 - Decay Heat Exchanger Pit Elev. 545'	0	0	1
x. FDZ 115 - ECCS Pump Room 1-2 Elev. 545'	0	0	2
y. FDZ 124 - Clean Waste Receiver Tank Rm. 1-1 - Elev. 545'	0	0	4
<b>4. <u>Auxiliary Building Fan Rooms</u></b>			
a. FDZ 500 - Radwaste & Fuel Handling Area and Air Supply Area - Elev. 623'	0	0	20
b. FDZ 501 - Radwaste Exhaust Equipment and Main Station Exhaust Fan Room Elev. 623'	0	0	22
c. FDZ 515 - Purge and Exhaust Equipment Rm. Elev. 623'	0	0	22
d. FDZ 516 - Non-rad Air and Exhaust Equip. Rm. - Elev. 623'	0	0	5

<u>INSTRUMENT LOCATION</u>		<u>MINIMUM INSTRUMENTS OPERABLE</u>		
		<u>HEAT</u>	<u>FLAME</u>	<u>SMOKE</u>
5.	<u>Control Room Complex</u>			
	a. FDZ 505 - Main Control Board Cabinets Elev. 623'	0	0	3
	b. FDZ 505 - Control Cabinet Room Elev. 623'	0	0	5
	c. FDZ 505 - Computer Room - Elev. 623'	0	0	1
6.	<u>Cable Spreading Room</u>			
	a. FDZ 422A - Elev. 613'	0	0	5
7.	<u>A/C Equipment Room</u>			
	a. FDZ 603 - Elev. 643'	0	0	11
8.	<u>Diesel Generator Rooms</u>			
	**a. FDZ 318 - Diesel Generator Rm. 1-1 Elev. 585'	0	0	5
	**b. FDZ 319 - Diesel Generator Rm. 1-2 Elev. 585'	0	0	4
	c. FDZ 321A - Diesel Generator Day Tank Room 1-1 - Elev. 5	0	0	1
	d. FDZ 320A - Diesel Generator Day Tank Room 1-2 - Elev. 5	0	0	1
9.	<u>Battery Rooms</u>			
	a. FDZ 428A - Battery Room B - Elev. 603'	0	0	2
	b. FDZ 429B - Battery Room A - Elev. 603'	0	0	2
10.	<u>Component Cooling Water Pump Room</u>			
	a. FDZ 328 - Elev. 585'	0	0	9
11.	<u>Feed Pump Rooms</u>			
	a. FDZ 237 - Auxiliary Feed Pump 1-1 Elev. 565'	0	0	3
	b. FDZ 238 - Auxiliary Feed Pump 1-2 Elev. 565'	0	0	3

MINIMUM INSTRUMENTS OPERABLE

INSTRUMENT LOCATION

HEAT                      FLAME                      SMOKE

12. Switchgear Rooms

a. FDZ 324 - CD High Voltage Switchgear Elev. 585'	0	0	3
b. FDZ 325 - A High Voltage Switchgear Elev. 585'	0	0	8
c. FDZ 323 - B High Voltage Switchgear Elev. 585'	0	0	11
d. FDZ 428 - F High Voltage Switchgear Elev. 603'	0	0	12
e. FDZ 429 - E High Voltage Switchgear Elev. 603'	0	0	6

13. Intake Structure

a. FDZ 052 - Diesel Fire Pump Room Elev. 576'	0	0	1
b. FDZ 052- Service Water Pump Room Elev. 576'	0	0	3
c. FDZ 053 - Service Water Vlv. Room Elev. 565'	0	0	6

\*The fire detection instruments located within the Containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

\*\*These detectors automatically actuate fire suppression systems.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-346THE TOLEDO EDISON COMPANYANDTHE CLEVELAND ELECTRIC ILLUMINATING COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 34 to Facility Operating License No. NPF-3, issued to The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees), which revised the Technical Specifications for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications to reflect modifications of the Fire Detection Instrumentation recently completed at the facility. These Technical Specification revisions delete the old Fire Detection System and incorporate the new Fire Detection System in the Limiting Conditions for Operation and the Surveillance Requirements.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR § 51.5(d)(4) an environmental impact statement or negative declaration and environ-

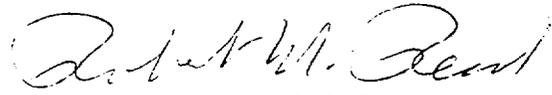
DEC 9 1980

mental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 7, 1980, (2) Amendment No. 34 to License No. NPF-3, and (3) the Commission's letter dated November 18, 1980. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Ida Rupp Public Library, 310 Madison Street, Port Clinton, Ohio. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 18th day of November 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing