

May 5, 1983

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Mr. Richard P. Crouse  
Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza - Stop 712  
300 Madison Avenue  
Toledo, Ohio 43652

Dear Mr. Crouse:

SUBJECT: AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. NPF-3

The Commission has issued the enclosed Amendment No. 58 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TSs) in response to Items 2 and 3 in your application dated October 14, 1982 (No. 862). Item 1 of your application will be acted upon separately.

This amendment modifies TS Table 3.3-4 to incorporate revised trip setpoints and allowable values for the Borated Water Storage Tank low level interlock and the essential bus feeder breaker trip delay which take into consideration instrument uncertainties. This amendment also revises TS Bases 3/4.7.1.2 to be consistent with the Updated Safety Analysis Report and the results of analyses submitted on May 22, 1981 (No. 717).

In addition to the above changes requested in your application, we have changed Bases 3/4.3.3.7 to correct the description of the operation of the Chlorine Detection Systems. This change has been discussed with members of your staff, and it is acceptable to them.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

~~Original~~

A. W. De Agazio, Project Manager  
Operating Reactors Branch #4  
Division of Licensing

No  
Concurrence  
5/13/83

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Enclosures:

1. Amendment No. 58 to NPF-3
2. Safety Evaluation
3. Notice

|         |          |            |            |          |
|---------|----------|------------|------------|----------|
| OFFICE  | ORB#4:DL | ORB#4:DL   | C-ORB#4:DL | AD/DR:DL |
| SURNAME | RIngram  | ADe Agazio | JSto       | CLatras  |
| DATE    | 3/28/83  | 3/27/83:ch | 3/27/83    | 3/27/83  |



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

May 5, 1983

DISTRIBUTION:  
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Docket No. 50-346

Docketing and Service Section  
Office of the Secretary of the Commission

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment No. 58.

Referenced documents have been provided PDR.

Division of Licensing, ORB#4  
Office of Nuclear Reactor Regulation

Enclosure:  
As Stated

|         |            |  |  |  |  |  |
|---------|------------|--|--|--|--|--|
| OFFICE  | ORB#4:DL   |  |  |  |  |  |
| SURNAME | RIngram;cf |  |  |  |  |  |
| DATE    | 5/11/83    |  |  |  |  |  |

Toledo Edison Company

cc w/enclosure(s):

Mr. Donald H. Hauser, Esq.  
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Nuclear Power Generation Division  
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Commissioners of Ottawa County  
Port Clinton, Ohio 43452

Attorney General  
Department of Attorney General  
30 East Broad Street  
Columbus, Ohio 43215

Harold Kahn, Staff Scientist  
Power Siting Commission  
361 East Broad Street  
Columbus, Ohio 43216

Mr. James G. Keppler, Regional Administrator  
U. S. Nuclear Regulatory Commission, Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Mr. Larry D. Young  
Manager, Nuclear Licensing  
Toledo Edison Company  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
5503 N. State Route 2  
Oak Harbor, Ohio 43449

Regional Radiation Representative  
EPA Region V  
230 South Dearborn Street  
Chicago, Illinois 60604

Ohio Department of Health  
ATTN: Radiological Health  
Program Director  
P. O. Box 118  
Columbus, Ohio 43216



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.58  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated October 14, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. NPF-3 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

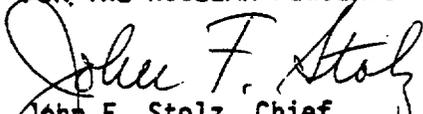
Revise paragraph 2.C.(2) to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.58 , are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 5, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 58

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-13

B 3/4 3-3

B 3/4 7-2

TABLE 3.3-4

SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

| <u>FUNCTIONAL UNIT</u>                                      | <u>TRIP SETPOINT</u>                                    | <u>ALLOWABLE VALUES</u>  |
|---|---|--|
| <b>INSTRUMENT STRINGS</b>                                   |   |  |
| a. Containment Radiation                                    | $< 2 \times$ Background at RATED THERMAL POWER          | $< 2 \times$ Background at RATED THERMAL POWER                       |
| b. Containment Pressure - High                              | $\leq 18.4$ psia  | $\leq 18.52$ psia <sup>*</sup>                                       |
| c. Containment Pressure - High-High                         | $\leq 38.4$ psia  | $\leq 38.52$ psia <sup>*</sup>                                       |
| d. RCS Pressure - Low                                       | $\geq 1620.75$ psig                                     | $\geq 1615.75$ psig <sup>*</sup>                                     |
| e. RCS Pressure - Low-Low                                   | $\geq 420.75$ psig                                      | $\geq 415.75$ psig <sup>*</sup>                                      |
| f. BWST Level   | $\geq 89.5$ and $\leq 100.5$ in. H <sub>2</sub> O       | $\geq 88.3$ and $\leq 101.7$ in. H <sub>2</sub> O <sup>*</sup>       |
| <b>SEQUENCE LOGIC CHANNELS</b>                              |   |  |
| a. Essential Bus Feeder Breaker Trip (90%)                  | $\geq 3744$ volts for $\leq 7.8$ sec                    | $\geq 3558$ volts $\leq 7.8$ sec                                     |
| b. Diesel Generator Start, Load Shed on Essential Bus (59%) | $\geq 2071$ and $\leq 2450$ volts for $0.5 \pm 0.1$ sec | $\geq 2071$ and $\leq 2450$ volts for $0.5 \pm 0.1$ sec <sup>*</sup> |
| <b>INTERLOCK CHANNELS</b>                                   |   |  |
| a. Decay Heat Isolation Valve and Pressurizer Heater        | $< 438$ psig  | $< 443$ psig <sup>*</sup>  |

<sup>\*</sup> Allowable Value for CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION.

\* Referenced to the centerline of DH11 and DH12

TABLE 3.3-5

SAFETY FEATURES SYSTEM RESPONSE TIMES

| <u>INITIATING SIGNAL AND FUNCTION</u>                                   | <u>RESPONSE TIME IN SECONDS</u> |
|---|---------------------------------|
| 1. Manual   |                                 |
| a. Fans   |                                 |
| 1. Emergency Vent Fan   | NA                              |
| 2. Containment Cooler Fan   | NA                              |
| b. HV & AC Isolation Valves   |                                 |
| 1. ECCS Room  | NA                              |
| 2. Emergency Ventilation  | NA                              |
| 3. Containment Air Sample   | NA                              |
| 4. Containment Purge  | NA                              |
| 5. Penetration Room Purge   | NA                              |
| c. Control Room HV & AC Units   | NA                              |
| d. High Pressure Injection  |                                 |
| 1. High Pressure Injection Pumps  | NA                              |
| 2. High Pressure Injection Valves                                       | NA                              |
| e. Component Cooling Water  |                                 |
| 1. Component Cooling Water Pumps  | NA                              |
| 2. Component Cooling Aux. Equip. Inlet Valves                           | NA                              |
| 3. Component Cooling to Air Compressor Valves                           | NA                              |
| f. Service Water System   |                                 |
| 1. Service Water Pumps  | NA                              |
| 2. Service Water From Component Cooling Heat Exchanger Isolation Valves | NA                              |
| g. Containment Spray Isolation Valves                                   | NA                              |
| h. Emergency Diesel Generator   | NA                              |
| i. Containment Isolation Valves   |                                 |
| 1. Vacuum Relief  | NA                              |
| 2. Normal Sump  | NA                              |
| 3. RCS Letdown Delay Coil Outlet  | NA                              |
| 4. RCS Letdown High Temperature   | NA                              |

### 3/4.3 INSTRUMENTATION

#### BASES

#### REMOTE SHUTDOWN INSTRUMENTATION (Continued)

HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost.

#### 3/4.3.3.6 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident.

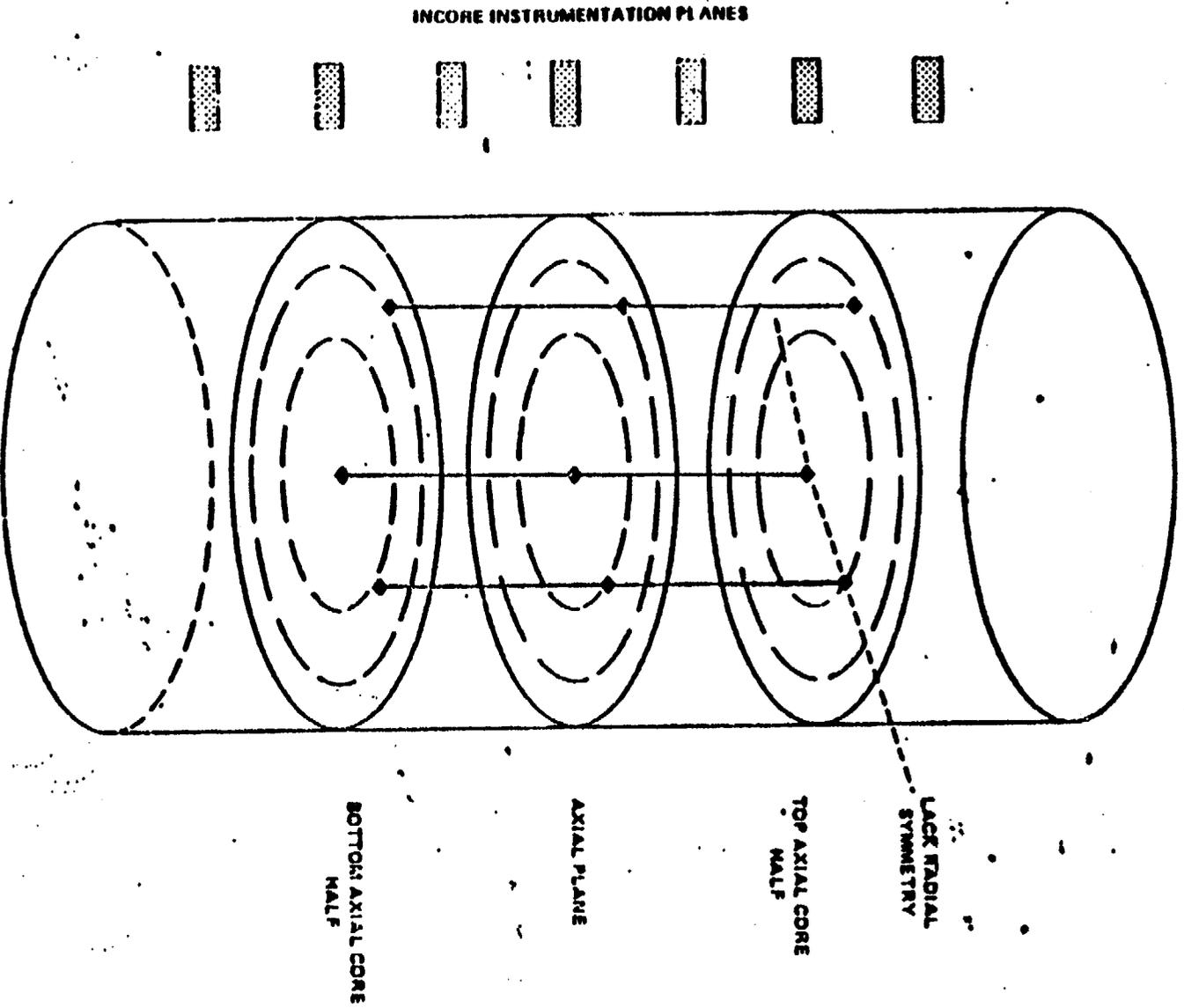
#### 3/4.3.3.7 CHLORINE DETECTION SYSTEMS

The OPERABILITY of the chlorine detection systems ensures that an accidental chlorine release will be detected promptly and the control room will be isolated automatically. The control room ventilation system will be started manually in the recirculation mode to provide the required protection. The chlorine detection systems required by this specification are consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operations Against an Accidental Chlorine Release," February 1975.

#### 3/4.3.3.8 FIRE DETECTION INSTRUMENTATION

Operability of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is restored to OPERABILITY.



Basics Figure 3-1 In-core Instrumentation Specification  
 Acceptable Minimum AXIAL POWER IMBALANCE Arrangement

DAVIS-BESSE, UNIT 1

B 3/4 3-4

### 3/4.7 PLANT SYSTEMS

#### BASES

#### 3/4.7.1 TURBINE CYCLE

#### 3/4.7.1.1 SAFETY VALVES

The OPERABILITY of the main steam line code safety valves ensures that the secondary system pressure will be limited to within 110% its design pressure of 1050 psig during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a turbine trip from 100% RATED THERMAL POWER coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Vessel Code, 1971 Edition. The total relieving capacity for all valves on all of the steam lines is 14,175,000 lbs/hr which is 120 percent of the total secondary steam flow of 11,760,000 lbs/hr at 100% RATED THERMAL POWER. A minimum of 2 OPERABLE safety valves per steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Table 3.7-1.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in secondary system steam flow and THERMAL POWER required by the reduced reactor trip settings of the High Flux channels. The reactor trip setpoint reductions are derived on the following bases:

$$SP = \frac{(X) - (Y)(V)}{Z} \times W$$

where:

SP = reduced Trip Setpoint in percent of RATED THERMAL POWER (Not to Exceed W)

V = maximum number of inoperable safety valves per steam generator

W = High Flux Trip Setpoint for four pump operation as specified in Table 2.2-1

X = Total relieving capacity of all safety valves per steam generator in lbs/hour, 7,087,500 lbs/hour

Y = Maximum relieving capacity of any one safety valve in lbs/hour, 845,759 lbs/hour

Z = Required relieving capacity per steam generator in lbs/hr, 6,585,600 lbs/hr.

## PLANT SYSTEMS

### BASES

#### 3/4.7.1.2 AUXILIARY FEEDWATER SYSTEMS

The OPERABILITY of the auxiliary feedwater systems ensures that the Reactor Coolant System can be cooled down to less than 280°F from normal operating conditions in the event of a total loss of offsite power.

Each steam driven auxiliary feedwater pump is capable of delivering a total feedwater flow of 800 gpm at a pressure of 1050 psig to the entrance of the steam generators. This capacity is sufficient to ensure that adequate feedwater flow is available to remove decay heat and reduce the Reactor Coolant System temperature to less than 280°F where the Decay Heat Removal System may be placed into operation.

#### 3/4.7.1.3 CONDENSATE STORAGE FACILITIES

The OPERABILITY of the condensate storage tank with the and to minimum water volume ensures that sufficient water is available to maintain the RCS at HOT STANDBY conditions for 13 hours with steam discharge to atmosphere and to cooldown the Reactor Coolant System to less than 280°F in the event of a total loss of offsite power or of the main feedwater system. The contained water volume limit includes an allowance for water not usable because of tank discharge line location or other physical characteristics.

#### 3/4.7.1.4 ACTIVITY

The limitations on secondary system specific activity ensure that the resultant offsite radiation dose will be limited to a small fraction of 10 CFR Part 100 limits in the event of a steam line rupture. This dose includes the effects of a coincident 1.0 GPM primary to secondary tube leak in the steam generator of the affected steam line. These values are consistent with the assumptions used in the safety analyses.

#### 3/4.7.1.5 MAIN STEAM LINE ISOLATION VALVES

The OPERABILITY of the main steam line isolation valves ensures that no more than one steam generator will blowdown in the event of a steam line rupture. This restriction is required to 1) minimize the



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. NPF-3

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

Introduction

By letter dated October 14, 1982, Toledo Edison Company (the licensee) requested an amendment to the Appendix A Technical Specifications (TSs) appended to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. The amendment would change the trip setpoints and allowable values for the Borated Water Storage Tank (BWST) low level interlock and the essential bus feeder breaker trip.

The amendment also revises TS Bases 3/4.7.1.2 to be consistent with the Updated Safety Analysis Report and the results of analyses previously submitted and TS Bases 3/4.3.3.7 to correct the description of operation of the Chlorine Detection Systems.

Background and Discussion

The BWST provides borated water to the suction of the decay heat (DH) and containment spray (CS) pumps. Analysis established that a minimum amount (360,000 gallons) of borated water must be available to these pumps. After this water has been used, the suction of the DH and CS pumps is manually transferred from the BWST to the Containment Emergency Sump. The BWST low level interlock prevents a premature manual transfer which could result in delivering less than the required volume of borated water. The higher values of the BWST interlock trip setpoint and allowable value are calculated so that instrument errors and uncertainties will not preclude the required volume of borated water from being delivered. The lower values of the setpoint and allowable values assure that the manual transfer can be made with a sufficient time margin to protect the pumps from cavitation due to a lack of proper net positive suction head. The licensee has recognized that the present TSs do not provide an adequate margin for instrument uncertainties. The requested modifications provide a small additional margin of 2 inches water level to the lower value of the trip setpoint and allowable values to ensure safe plant operation. Once the permissive is reached (between 89.5 and 100.5 inches), the operator is alerted to begin the transfer. The transfer of pump suction from the BWST to the containment emergency sump requires operator action prior to the water level dropping below an indicated level of 49.5 inches. Automatic transfer of pump suction was deleted by Amendment No. 36 dated January 24, 1981.

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In a loss of offsite power situation, the undervoltage relays (set at 90% voltage) on the 4.16 KV safety related buses initiate tripping of the bus feeder breakers. This is to ensure acceptable voltages on the safety related 480V buses for continuous and emergency operation. The accident analyses assume that the maximum allowable operating delay for these relays is 9 seconds. In order to be assured that the 9-second limitation is not exceeded, the trip setpoint is established with instrument uncertainties taken into consideration. The licensee has determined that the combined channel uncertainties amount to 15% of the setting and has proposed this modification to the TSs.

### Evaluation

The licensee has stated that in order to properly account for instrument errors, changes in the setpoints for the BWST low level interlock and essential bus feeder breaker trip delay time should be incorporated in the TSs.

The analysis provided by the licensee shows that the minimum amount of borated water will be available even when instrumentation uncertainties are considered. In addition, with the additional margin for the lower value of the trip setpoint and allowable value, the operator will have sufficient time to commence the switchover so as to protect the CS and DH pumps from cavitation. Therefore, the request to modify the trip setpoint to greater than 89.5 inches from 91.5 inches and to modify the allowable value to greater than 88.3 inches from 90.3 inches is acceptable.

The analysis performed by the licensee to modify the delay time TSs for the essential bus feeder breaker trip provides sufficient assurance that the time delay required by the safety analyses will not be exceeded by instrument uncertainties. The request to change the present value of 7+1.5 seconds to a value of less than 7.8 seconds will provide a 15% margin and is therefore acceptable.

Based upon this evaluation, we find the TS changes proposed for the Davis-Besse BWST level and the essential bus feeder breaker trip to be acceptable.

The Updated Safety Analysis Report, Section 9.2.7, and an analysis submitted by Toledo Edison Company by letter dated May 22, 1981 (No. 717) establish that a flow of 800 gpm is acceptable to accomplish the safety function of the Auxiliary Feedwater System. Therefore, we find the proposed revision to Bases 3/4.7.1.2 acceptable.

On November 30, 1982, the Commission issued Amendment 50 to the operating license. This amendment modified TS Bases 3/4.3.3.7 to correct the description of the control room ventilation system operation in response to high chlorine concentration and manual startup of the control room ventilation system. That amendment, however, incorrectly referred to the control room emergency ventilation system. The correct operation is described in the following paragraphs.

When a high chlorine concentration (> 0.5 ppm) is detected at any of the four chlorine detectors:

1. The control room supply and exhaust fans are stopped,
2. The inlet and outlet dampers are closed, and
3. The control room isolation damper is closed.

Following the automatic isolation of the control room, the control room ventilation system can be started manually by leaving the supply and exhaust dampers closed, opening the isolation dampers, and starting the air-conditioning system in the recirculation mode.

The change to Basés 3/4.3.3.7 corrects the description of operation of the control room ventilation system and does not involve a modification to the system itself. Our finding in Amendment 50 regarding the acceptability of the change stands.

#### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 5, 1983

The following NRC personnel have contributed to this Safety Evaluation:  
Mark Wigdor, ICSB; Albert De Agazio, ORB#4.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-346THE TOLEDO EDISON COMPANYANDTHE CLEVELAND ELECTRIC ILLUMINATING COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 58 to Facility Operating License No. NPF-3, issued to The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees), which revised Technical Specifications (TSs) for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of its date of issuance.

This amendment modifies TS Table 3.3-4 to incorporate revised trip setpoints and allowable values for the Borated Water Storage Tank low level interlock and the essential bus feeder breaker trip delay which take into consideration instrument uncertainties. This amendment also revises: (1) TS Bases 3/4.7.1.2 to be consistent with the Updated Safety Analysis Report and the results of analyses and (2) TS Bases 3/4.3.3.7 to correct the description of the operation of the Chlorine Detection Systems.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations

in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 14, 1982, (2) Amendment No. 58 to License No. NPF-3, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and at the William Carlson Library, University of Toledo, 2801 Bancroft Avenue, Toledo, Ohio 43606.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 5th day of May 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stoiz, Chief  
Operating Reactors Branch #4  
Division of Licensing