

November 28, 1984

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Docket No. 50-346

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Toledo Edison Company
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Toledo, Ohio 43652

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Dear Mr. Crouse:

SUBJECT: AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. NPF-3;
CONTAINMENT AIR RECIRCULATION FAN STATUS INSTRUMENTATION

The Commission has issued the enclosed Amendment No. 78 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of changes to the Appendix A Technical Specifications (TSs) in response to your application dated February 28, 1984 (No. 1028).

This amendment deletes from Specification 3.3.3.6, Table 3.3-10, the requirement for a minimum of one operable channel of instrumentation to provide containment air recirculation fan status, and deletes from Specification 4.3.3.6, Table 4.3-10, the associated surveillance requirement.

Amendment No. 66, issued, January 20, 1984, deleted Limiting Condition for Operation 3.6.4.2 and Surveillance Requirement 4.6.4.2 relating to the Containment Recirculation System. The Containment Recirculation System consists of ducts and fans and is designed to mix the atmosphere in the containment building to prevent localized buildups of post-accident generated hydrogen to a combustible concentration. The Safety Evaluation supporting Amendment No. 66 concluded that such mixing is also provided by natural correction currents and the combined action of the containment spray and the fan coolers.

With the requirement for containment Recirculation System operability deleted from the Technical Specifications, all other references to the operability or surveillance of the system or components also should have been deleted. However, the requirement contained in specifications 3.3.3.6 and 4.3.3.6 (Tables 3.3-6 and 4.3-6) were overlooked.

Based on our review of your application and the considerations discussed above and in the Safety Evaluation supporting Amendment No. 66, we find your proposed Technical Specification changes acceptable.

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PDR ADOCK 05000346
PDR

This amendment relates to a change in a requirement with respect to the use and surveillance of a facility component within the restricted area as defined by 10 CFR 20. We have determined that the amendment involves no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to health and safety of the public.

The Notice of Issuance will be included in the Commission's Monthly Notice.

Sincerely,

***ORIGINAL SIGNED BY
JOHN F. STOLZ***

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosure:
Amendment No. 78

cc w/enclosure:
See next page

ORB#4:DL
RIngram
11/14/84

ORB#4:DL *Call*
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11/15/84

ORB#4:DL
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[Handwritten signature]

Toledo Edison Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated February 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:


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P PDR

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 78, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 28, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 78

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-48

3/4 3-50

TABLE 3.3-10
POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. SG Outlet Steam Pressure	1/Steam Generator
2. RC Loop Outlet Temperature	2/Loop
3. RC Loop Pressure	2/Loop
4. Pressurizer Level	2
5. SG Startup Range Level	2/Steam Generator
6. Auxiliary Feedwater Status	1/AFW System
7. Containment Vessel Hydrogen	2
8. Containment Vessel Post-Accident Radiation	2
9. Containment Vessel Isolation Status	1/Valve
10. SFAS Status	1/Channel
11. Safety Features Equipment Status	1/System
12. RPS Status	1/Channel
13. SFACS Status	1/Channel
14. High Pressure Injection Flow	1/Channel

TABLE 3.3-10 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>
15. Low Pressure Injection (LPI) Flow	1/Channel
16. HPI System Pump and Valve Status	1/System
17. LPI System Pump and Valve Status	1/System
18. Containment Spray Pump and Valve Status	1/System
19. Core Flood Valve Status	1/System
20. BWST Valve Status	1/System
21. Containment Emergency Sump Valve Status	1/Valve
22. Deleted	
23. Containment Air Cooling Fan Status	1/Fan
24. EVS Fan and Damper Status	1/System
25. Auxiliary Feedwater Flow Rate	2/Steam Generator
26. RC System Subcooling Margin Monitor	1
27. PORV Position Indicator	1
28. PORV Block Valve Position Indicator	1
29. Safety Valve Position Indicator	1/Valve
30. BWST Level	3

TABLE 4.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. SG Outlet Steam Pressure	M	R*
2. RC Loop Outlet Temperature	M	R
3. RC Loop Pressure	M	R
4. Pressurizer Level	M	R
5. SG Startup Range Level	M	R
6. Auxillary Feedwater Status	M	NA
7. Containment Vessel Hydrogen	M	R
8. Containment Vessel Post-Accident Radiation	M	R
9. Containment Vessel Isolation Status	M	NA
10. SFAS Status	M	NA
11. Safety Features Equipment Status	M	NA
12. RPS Status	M	NA
13. SFRCS Status	M	NA
14. High Pressure Injection Flow	M	R

*18 month surveillance test due May 17, 1983, may be delayed until 2400 hours September 17, 1983.

TABLE 4.3-10 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
15. Low Pressure Injection (DIIR) Flow	H	R
16. HPI System Pump and Valve Status	H	NA
17. LPI System Pump and Valve Status	H	NA
18. Containment Spray Pump and Valve Status	H	NA
19. Core Flood Valve Status	H	NA
20. BWST Valve Status	H	NA
21. Containment Emergency Sump Valve Status	H	NA
22. Deleted		
23. Containment Air Cooling Fan Status	H	NA
24. FVS Fan and Damper Status	H	NA
25. Auxiliary Feedwater Flow Rate	H	R
26. RC System Subcooling Margin Monitor	H	R
27. PORV Position Indicator	H	R
28. PORV Block Valve Position Indicator	H	R
29. Pressurizer Safety Valve Position Indicator	H	R
30. BWST Level	S	R

DAVIS BESSIE, UNIT 1

3/4 3-50

Amendment No. 26, 27, 78