

DMB-016

October 26, 1983

Socket No. 50-346

Mr. Richard P. Crouse  
Vice President - Nuclear  
Toledo Edison Company  
Edison Plaza - Stop 712  
300 Madison Avenue  
Toledo, Ohio 43652

Dear Mr. Crouse:

SUBJECT: AMENDMENT 63 TO FACILITY OPERATING LICENSE NO. NPF-3;  
AFW TURBINE SPEED SWITCH INTERLOCK SURVEILLANCE

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. This amendment consists of a change to the Appendix A Technical Specification (TSs) in response to Item 1 of your application dated July 10, 1981 (No. 731). Items 3, 4, 5, and 6 are under review and will be the subject of separate licensing actions. Item 2 was withdrawn by your letter dated February 12, 1982. Supplemental information related to Item 1 was submitted with your letter of May 17, 1983 (No. 942); this information was considered in evaluating the requested TS modification.

This amendment deletes from Specification 4.7.1.2.d the requirement for periodic channel functional tests and channel calibrations of the auxiliary feed pump turbine speed switches. With the deletion of this surveillance requirement, the speed switches and interlocks to valves HV 360 and HV 388 can be removed.

The purpose of the speed switch interlocks with the auxiliary feedwater (AFW) pump discharge valves (HV 360 and HV 388) is to prevent steam generator overfill. Under low turbine speed conditions which correspond to a low steam generator pressure of 90 psi, the speed switches prevent the insertion of water into the steam generator above the 3 ft. steam generator level by closing the AFW pump discharge valves. This prevents overcooling of the reactor coolant system. Under normal operating conditions, including startup and shutdown, the AFW system is not used and therefore the speed switches provide no benefit to safety under these conditions. During cooldown to cold shutdown conditions when the AFW system has been used, the Decay Heat Removal System is initiated, so that the AFW system is no longer required, at conditions corresponding to a steam generator pressure of approximately 220 psi--significantly higher than the 90 psi at which the turbine speed switches would be required to operate. In addition, we have not identified any event which together with a single failure would result in a pressure of 90 psi or less in both steam generators and require the use of the AFW system.

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SURNAME							
DATE							

Mr. Richard Crouse

- 2 -

Instrumentation in the control room provides the operator with the water levels within the steam generators, steam generator steam pressure, and AFW turbine speed. A computer alarm is provided for low turbine speed.

The only manual action which would be required upon low steam generator pressure without the speed switches is closing of the AFW pump discharge valve for each of the two AFW pumps. This action can be performed from within the control room. You stated that the minimum time from full power operation until operator action would be required to shut the AFW discharge valves is two hours and thirty minutes. This appears to be adequate time for the operator to perform whatever action necessary as the result of the initial event prior to the operator's attention being required by the AFW system to close the pump discharge valves.

Based on our review of your application and the considerations discussed above, we find your proposed TS change acceptable.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

The evaluation of this amendment request was provided by John N. Ridgely of the NRR staff. Notice of Issuance will be included in the Commission's next Monthly Notice.

Original signed by

John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Amendment No. 63 to NPF-3

cc: See next page \*See other white for concurrences

*JFB*  
No legal objections *10/25*

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD:OR:DL	OEL: <i>Mull</i>
SURNAME	RIngram*	AD: <i>Adler</i> To: ps JStolz*	GLainas*		<i>m. Rothchild</i>
DATE	9/19/83	9/27/83	9/21/83	9/22/83	9/30/83

Mr. Richard Crouse

- 2 -

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John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Amendment No. to NPF-3

cc:  
See next page

OFFICE	ORB#4:DL	ORB#4:DL	ORB#4:DL	AD:OR:DL	OELD		
SURNAME	R. Ingram	A De Agazio	J Stolz	GL Athas			
DATE	9/19/83	9/19/83	9/20/83	9/21/83	9/ /83		

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 26, 1983

Docket No. 50-346

Mr. Richard P. Crouse  
Vice President - Nuclear  
Toledo Edison Company  
Edison Plaza - Stop 712  
300 Madison Avenue  
Toledo, Ohio 43652

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SUBJECT: AMENDMENT 63 TO FACILITY OPERATING LICENSE NO. NPF-3;  
AFW TURBINE SPEED SWITCH INTERLOCK SURVEILLANCE

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Instrumentation in the control room provides the operator with the water levels within the steam generators, steam generator steam pressure, and AFW turbine speed. A computer alarm is provided for low turbine speed.

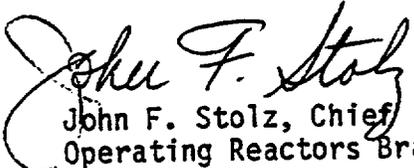
The only manual action which would be required upon low steam generator pressure without the speed switches is closing of the AFW pump discharge valve for each of the two AFW pumps. This action can be performed from within the control room. You stated that the minimum time from full power operation until operator action would be required to shut the AFW discharge valves is two hours and thirty minutes. This appears to be adequate time for the operator to perform whatever action necessary as the result of the initial event prior to the operator's attention being required by the AFW system to close the pump discharge valves.

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We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

The evaluation of this amendment request was provided by John N. Ridgely, of the NRR staff. Notice of Issuance will be included in the Commission's next Monthly Notice.

  
John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Amendment No. 63 to NPF-3

cc:  
See next page

Toledo Edison Company

cc w/enclosure(s):

Mr. Donald H. Hauser, Esq.  
The Cleveland Electric  
Illuminating Company  
P. O. Box 5000  
Cleveland, Ohio 44101

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
5503 N. State Route 2  
Oak Harbor, Ohio 43449

Gerald Charnoff, Esq.  
Shaw, Pittman, Potts  
and Trowbridge  
1800 M Street, N.W.  
Washington, D. C. 20036

Paul M. Smart, Esq.  
Fuller & Henry  
300 Madison Avenue  
P. O. Box 2088  
Toledo, Ohio 43603

Regional Radiation Representative  
EPA Region V  
230 South Dearborn Street  
Chicago, Illinois 60604

Mr. Robert B. Borsum  
Babcock & Wilcox  
Nuclear Power Generation Division  
7910 Woodmont Avenue, Suite 220  
Bethesda, Maryland 20814

Ohio Department of Health  
ATTN: Radiological Health  
Program Director  
P. O. Box 118  
Columbus, Ohio 43216

President, Board of County  
Commissioners of Ottawa County  
Port Clinton, Ohio 43452

Attorney General  
Department of Attorney General  
30 East Broad Street  
Columbus, Ohio 43215

James W. Harris, Director (Addressee Only)  
Division of Power Generation  
Ohio Department of Industrial Relations  
2323 West 5th Avenue  
P. O. Box 825  
Columbus, Ohio 43216

Harold Kahn, Staff Scientist  
Power-Siting Commission  
361 East Broad Street  
Columbus, Ohio 43216

Mr. James G. Keppler, Regional Administrator  
U. S. Nuclear Regulatory Commission, Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

8311160406 831026  
PDR ADOCK 05000346  
P PDR

Mr. Larry D. Young  
Manager, Nuclear Licensing  
Toledo Edison Company  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555



THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.63  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated July 10, 1981, as supplemented May 17, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-3 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

Revise paragraph 2.C.(2) to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license-amendment becomes effective upon removal of the auxiliary feedwater turbine speed switch interlocks to valves HV 360 and HV 388.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 26, 1983

ATTACHMENT TO LICENSE AMENDMENT NO 63

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

Pages

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PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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- c. The Auxiliary Feed Pump Turbine Steam Generator Level Control System shall be demonstrated OPERABLE by performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days, and a CHANNEL CALIBRATION at least once per 18 months.
  
- d. The Auxiliary Feed Pump Suction Pressure Interlocks, and Auxiliary Feed Pump Turbine Inlet Steam Pressure Interlocks shall be demonstrated OPERABLE by performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days, and a CHANNEL CALIBRATION at least once per 18 months.

## PLANT SYSTEMS

### CONDENSATE STORAGE TANK

#### LIMITING CONDITION FOR OPERATION

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3.7.1.3 The condensate storage facilities (condensate storage tank and deaerator storage tank) shall be OPERABLE with a minimum contained volume of 250,000 gallons of water.

APPLICABILITY: MODES 1, 2 and 3.

#### ACTION:

With the condensate storage facilities inoperable, within 4 hours either:

- a. Restore the condensate storage facilities to OPERABLE status or be in HOT SHUTDOWN within the next 12 hours, or
- b. Demonstrate the OPERABILITY of the service water system as a backup supply to the auxiliary feedwater system and restore the condensate storage facilities to OPERABLE status within 7 days or be in HOT SHUTDOWN within the next 12 hours.

#### SURVEILLANCE REQUIREMENTS.

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4.7.1.3.1 The condensate storage facilities shall be demonstrated OPERABLE at least once per 12 hours by verifying the contained water volume to be within its limits when the facilities are the supply source for the auxiliary feedwater pumps.

4.7.1.3.2 The service water system shall be demonstrated OPERABLE at least once per 12 hours by verifying that at least one service water loop is operating and that the service water loop-auxiliary feedwater system isolation valves are either open or OPERABLE whenever the service water system is the supply source for the auxiliary feedwater pumps.