

DECEMBER 23 1981

Docket No. 50-346

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DMB 016

Mr. Richard P. Crouse
 Vice President, Nuclear
 Toledo Edison Company
 Edison Plaza
 300 Madison Avenue
 Toledo, Ohio 43652

Dear Mr. Crouse:

The Commission has issued the enclosed Amendment No. 42 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 5, 1981.

This amendment modifies the TSs concerning regulating rod groups position limits, axial power shaping rod position limits and surveillance requirement, and axial power imbalance limits and corrective action requirement.

The TS modifications concerning regulating rod groups position limits, axial power shaping rod position limits and axial power imbalance limits will accommodate extension of the present cycle length to 306 + 10-30 Effective Full Power Days.

The modification to the axial power shaping rod position surveillance requirement makes this surveillance requirement similar to the regulating rod position surveillance requirement. This modification was also requested by attachment 4 of your application dated December 26, 1980, but which has not previously been approved.

The modification to the axial power imbalance corrective action requirement relaxes the requirement to bring the unit to hot-standby if axial power imbalance exceeds the limits for more than 15 minutes. The modified TS permits continued operation at reduced power to restore operation within limits. This modification was also requested by attachment 4 of your application dated July 10, 1981, but which has previously not been approved.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Handwritten signature
 JOHN F. STOLZ
 John F. Stolz, Chief
 Operating Reactors Branch #4
 Division of Licensing

Handwritten note:
 No verbal objection
 to FR notice and amend
 OELD
 TREBY
 12/23/81

- Enclosures:
 1. Amendment No. 42
 2. Safety Evaluation
 3. Notice



OFFICE	cc w/enclosures: See next page	ORB#4:DL RIngram;cf 12/27/81	ORB#4:DL ADe Agazio 12/27/81	ORB#4:DL JStolz 12/27/81	AD, PR:DL Novak 12/27/81
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OFFICIAL RECORD COPY

Toledo Edison Company

cc w/enclosure(s):

Mr. Donald H. Hauser, Esq.
The Cleveland Electric
Illuminating Company
P. O. Box 5000
Cleveland, Ohio 44101

Gerald Charnoff, Esq.
Shaw, Pittman, Potts
and Trowbridge
1800 M Street, N.W.
Washington, D. C. 20036

Paul M. Smart, Esq.
Fuller & Henry
300 Madison Avenue
P. O. Box 2088
Toledo, Ohio 43603
Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

President, Board of County
Commissioners of Ottawa County
Port Clinton, Ohio 43452

Attorney General
Department of Attorney General
30 East Broad Street
Columbus, Ohio 43215

Harold Kahn, Staff Scientist
Power Siting Commission
361 East Broad Street
Columbus, Ohio 43216

Mr. Ted Myers
Manager, Nuclear Licensing
Toledo Edison Company
Edison Plaza
300 Madison Avenue
Toledo, Ohio 43652

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
5503 N. State Route 2
Oak Harbor, Ohio 43449

Mrs. Julia Baldwin, Librarian
Government Documents Collection
William Carlson Library
University of Toledo
2801 W. Bancroft Avenue
Toledo, Ohio 43606

Regional Radiation Representative
EPA Region V
230 South Dearborn Street
Chicago, Illinois 60604

cc w/enclosure(s) and incoming dtd.:
11/5/81

Ohio Department of Health
ATTN: Radiological Health
Program Director
P. O. Box 118
Columbus, Ohio 43216



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated November 5, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. NPF-3 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

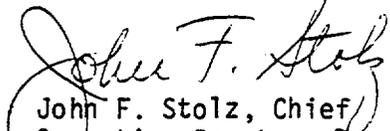
Revise paragraph 2.C.(2) to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 23, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 1-26
3/4 1-28a
3/4 1-28b (new page)
3/4 1-29a
3/4 1-29b
3/4 1-34
3/4 1-36
3/4 1-38
3/4 1-39 (new page)
3/4 2-1
3/4 2-2a
3/4 2-2b (new page)
3/4 2-3a
3/4 2-3b (new page)

REACTIVITY CONTROL SYSTEMS

SAFETY ROD INSERTION LIMIT

LIMITING CONDITION FOR OPERATION

3.1.3.5 All safety rods shall be fully withdrawn.

APPLICABILITY: 1* and 2*#.

ACTION:

With a maximum of one safety rod not fully withdrawn, except for surveillance testing pursuant to Specification 4.1.3.1.2, within one hour either:

- a. Fully withdraw the rod or
- b. Declare the rod to be inoperable and apply Specification 3.1.3.1.

SURVEILLANCE REQUIREMENTS

4.1.3.5 Each safety rod shall be determined to be fully withdrawn:

- a. Within 15 minutes prior to withdrawal of any regulating rod during an approach to reactor criticality.
- b. At least once per 12 hours thereafter.

*See Special Test Exception 3.10.1 and 3.10.2.

#With $K_{eff} \geq 1.0$.

REACTIVITY CONTROL SYSTEMS

REGULATING ROD INSERTION LIMITS

LIMITING CONDITION FOR OPERATION

3.1.3.6 The regulating rod groups shall be limited in physical insertion as shown on Figures 3.1-2a, 2b and 2c and 3.1-3a, 3b and 3c, with a rod group overlap of $25 \pm 5\%$ between sequential withdrawn groups 5, 6, and 7.

APPLICABILITY: MODES 1* and 2*#.

ACTION:

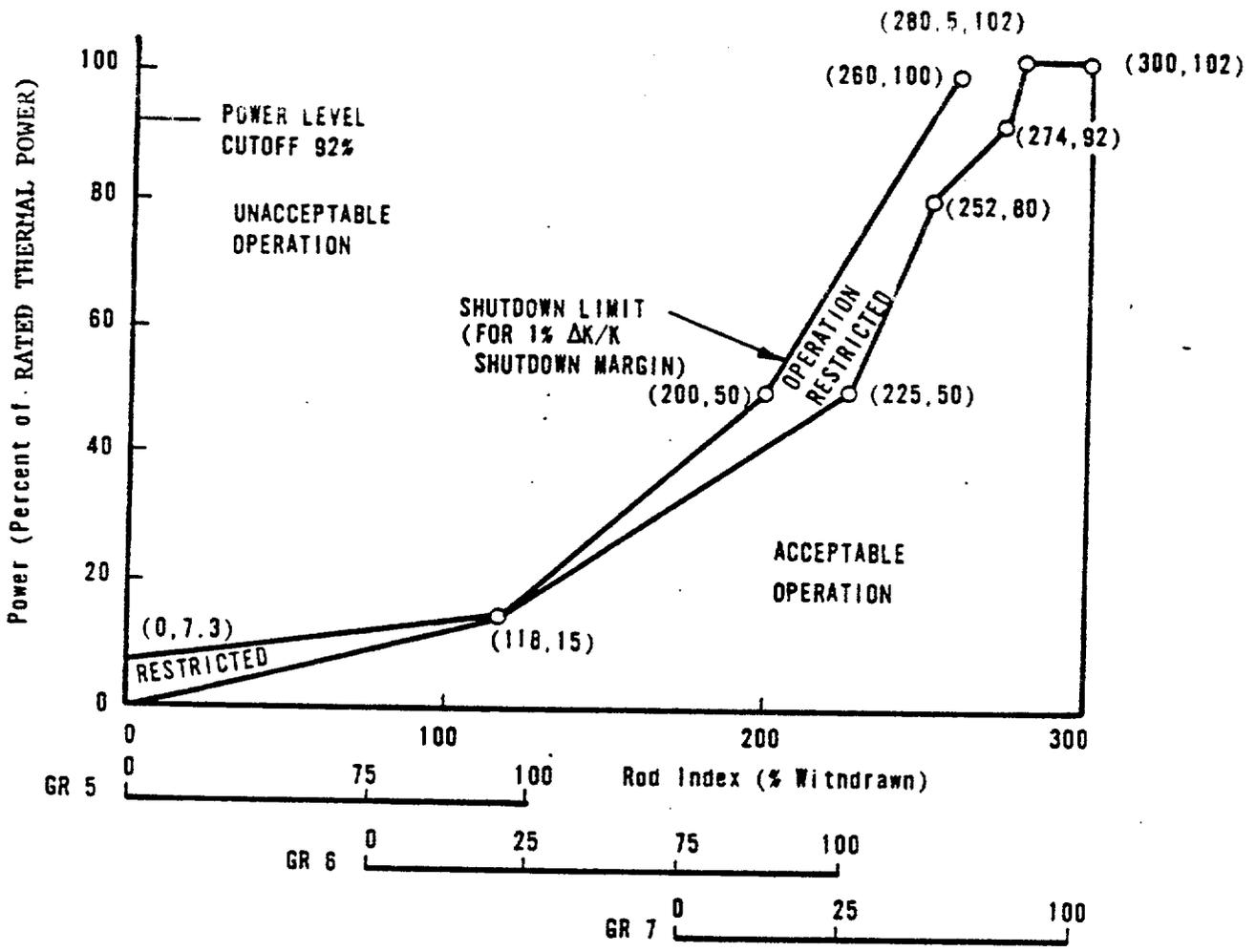
With the regulating rod groups inserted beyond the above insertion limits (in a region other than acceptable operation), or with any group sequence or overlap outside the specified limits, except for surveillance testing pursuant to Specification 4.1.3.1.2, either:

- a. Restore the regulating groups to within the limits within 2 hours, or
- b. Reduce THERMAL POWER to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the rod group position using the above figures within 2 hours, or
- c. Be in at least HOT STANDBY within 6 hours.

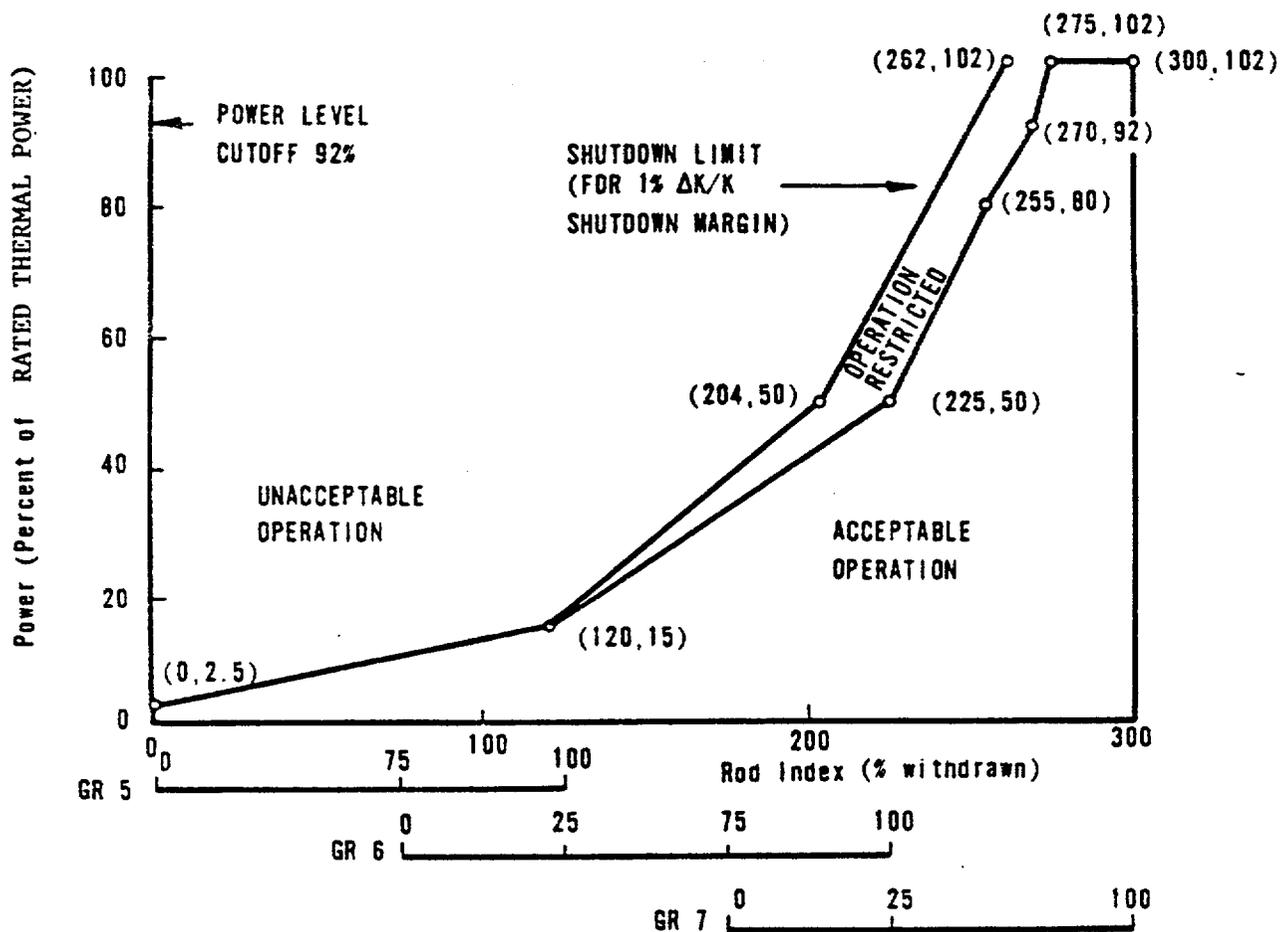
NOTE: If in unacceptable region, also see Section 3/4.1.1.1.

*See Special Test Exceptions 3.10.1 and 3.10.2.

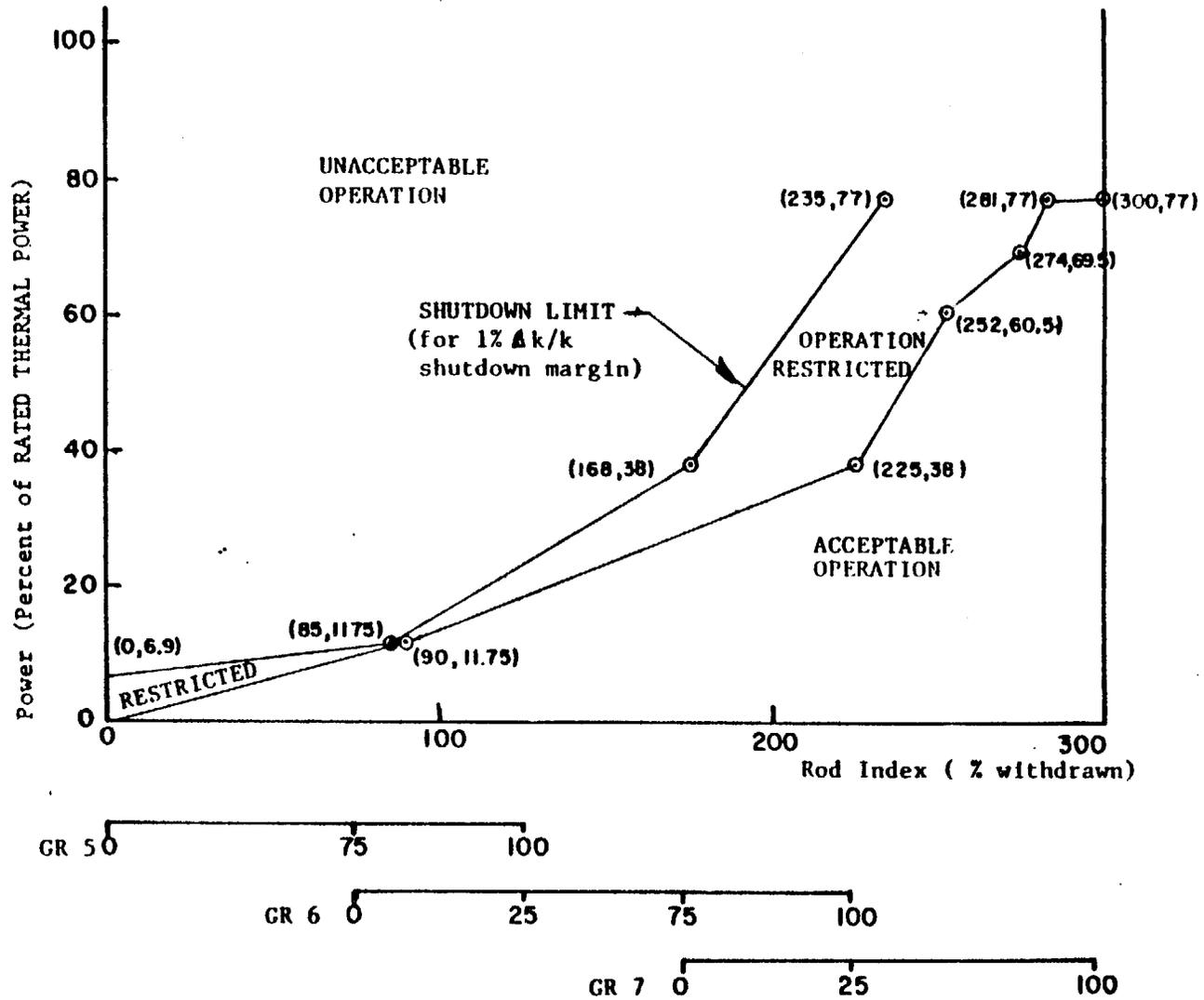
#With $k_{eff} \geq 1.0$.



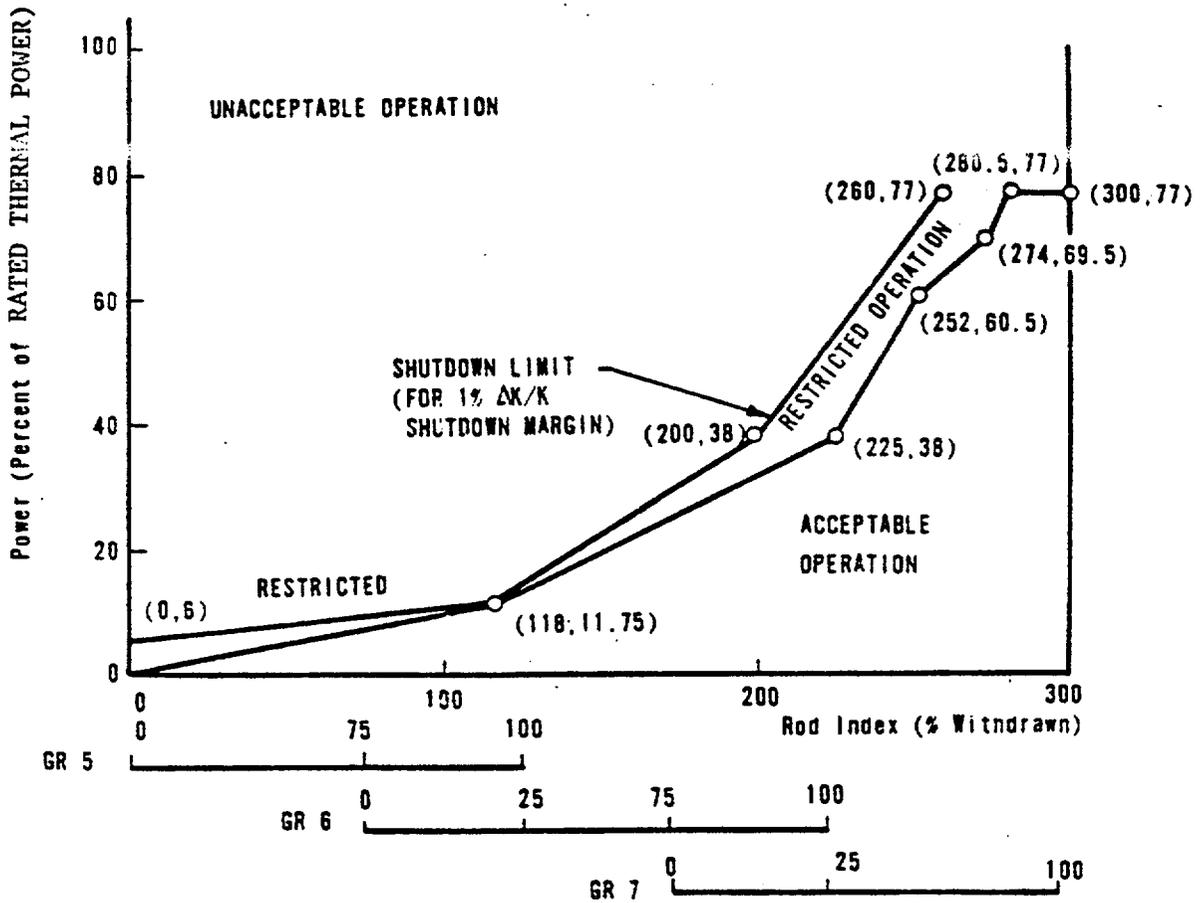
TECHNICAL SPECIFICATION FIGURE 3.1-2b. Regulating Group Position Limits. 150 ±10 to 250 ±10 EFPD, Four RCPs - Davis-Besse 1, Cycle 2



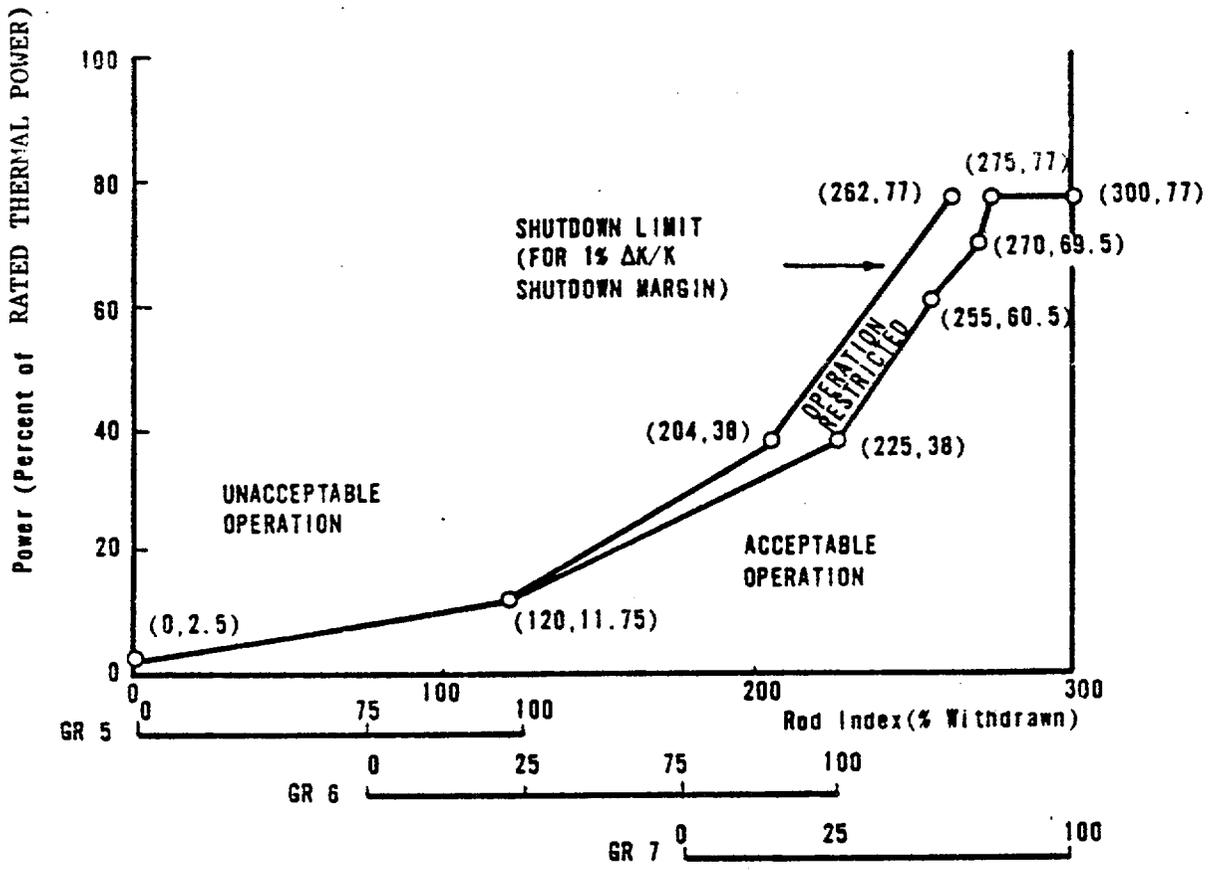
TECHNICAL SPECIFICATION FIGURE 3.1-2c. Regulating Group Position Limits, After 250 ± 10 EFPD, Four RCPs - Davis-Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.1-3a Regulating Group Position Limits, 0 to 150 ± 10 EFPD, Three RCPs - Davis Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.1-3b. Regulating Group Position Limits, 150 ±10 to 250 ±10 EFPD, Three RCPs - Davis-Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.1-3c. Regulating Group Position Limits, After 250 ±10 EFPD, Three RCPs - Davis-Besse 1, Cycle 2

REACTIVITY CONTROL SYSTEMS

XENON REACTIVITY

LIMITING CONDITION FOR OPERATION

3.1.3.8 THERMAL POWER shall not be increased above the power level cutoff specified in Figure 3.1-2 unless one of the following conditions is satisfied:

- a. Xenon reactivity is within 10 percent of the equilibrium value for RATED THERMAL POWER and is approaching stability, or
- b. THERMAL POWER has been within a range of 87 to 92 percent of RATED THERMAL POWER for a period exceeding 2 hours in the soluble poison control mode, excluding xenon free start-ups.

APPLICABILITY: MODE 1.

ACTION:

With the requirements of the above specification not satisfied, reduce THERMAL POWER to less than or equal to the power level cutoff within 15 minutes.

SURVEILLANCE REQUIREMENTS

4.1.3.8 Xenon reactivity shall be determined to be within 10% of the equilibrium value for RATED THERMAL POWER and to be approaching stability or it shall be determined that the THERMAL POWER has been in the range of 87 to 92% of RATED THERMAL POWER for \geq 2 hours, prior to increasing THERMAL POWER above the power level cutoff.

REACTIVITY CONTROL SYSTEMS

AXIAL POWER SHAPING ROD INSERTION LIMITS

LIMITING CONDITION FOR OPERATION

3.1.3.9 The axial power shaping rod group shall be limited in physical insertion as shown on Figures 3.1-5a, -5b, -5c, -5d and 5e.

APPLICABILITY: MODES 1 and 2*.

ACTION:

With the axial power shaping rod group outside the above insertion limits, either:

- a. Restore the axial power shaping rod group to within the limits within 2 hours, or
- b. Reduce THERMAL POWER to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the rod group position using the above figures within 2 hours, or
- c. Be in at least HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.9 The position of the axial power shaping rod group shall be determined to be within the insertion limits at least once every 12 hours except when the axial power shaping rod insertion limit alarm is inoperable, then verify the group to be within the insertion limit at least once every 4 hours.

*With $k_{eff} \geq 1.0$.

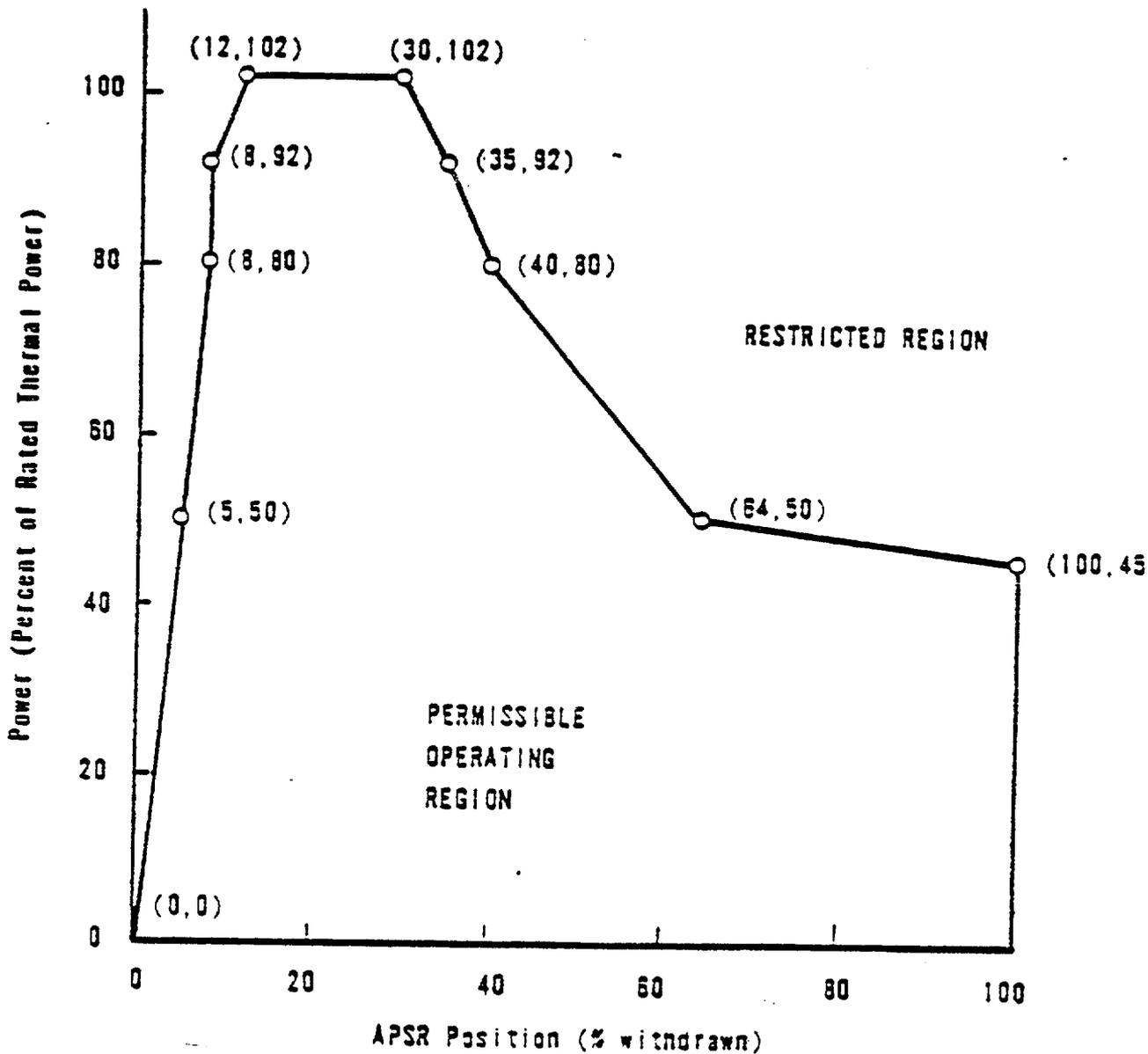
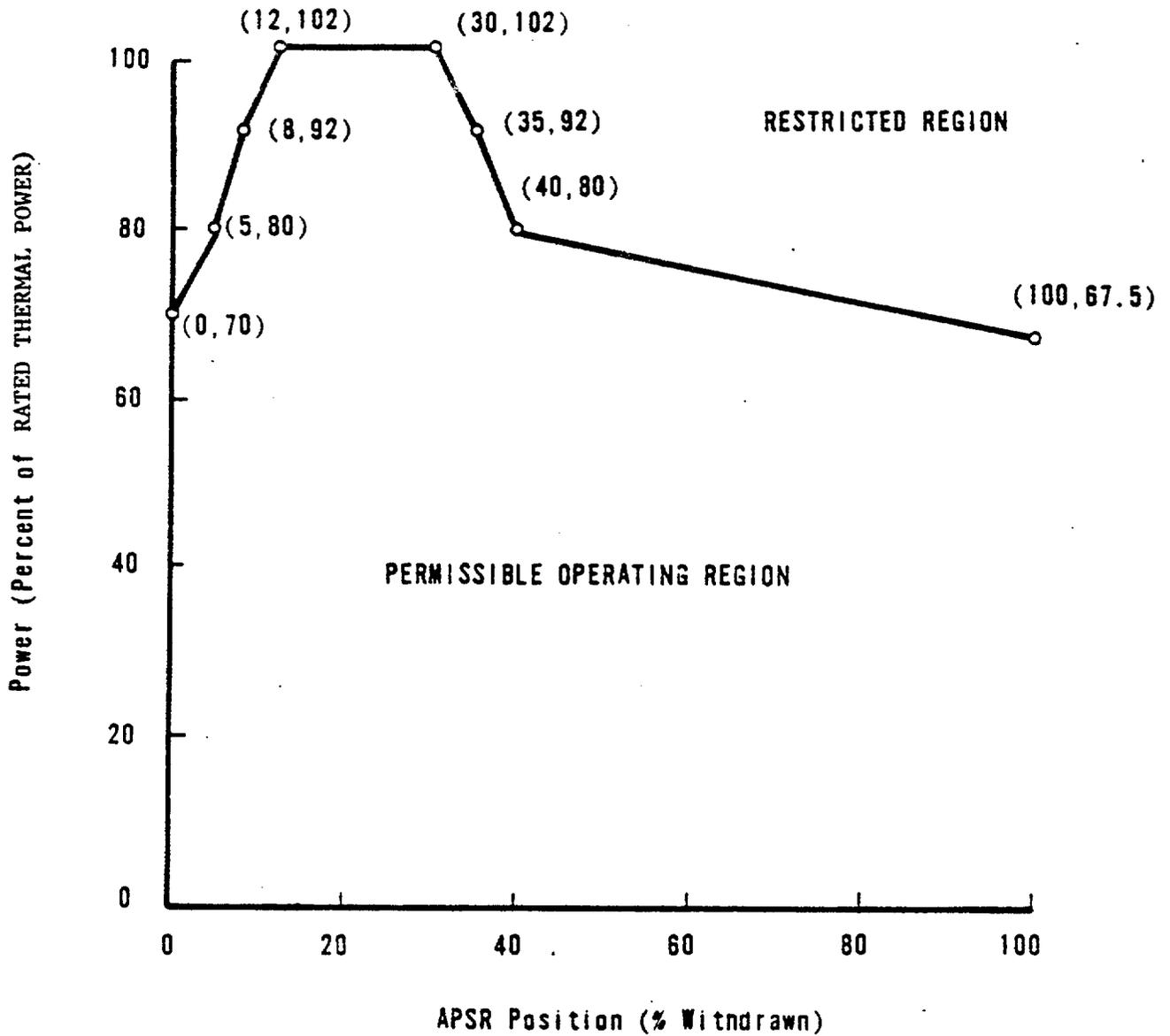


FIGURE 3.1-5a
 APSR Position Limits, 0 to 150 \pm 10 EFWD,
 Four RCPs - Davis-Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.1-5b. APSR Position Limit, 150 ± 10 to 250 ± 10 EFPD, Four RCPs - Davis-Besse 1, Cycle 2

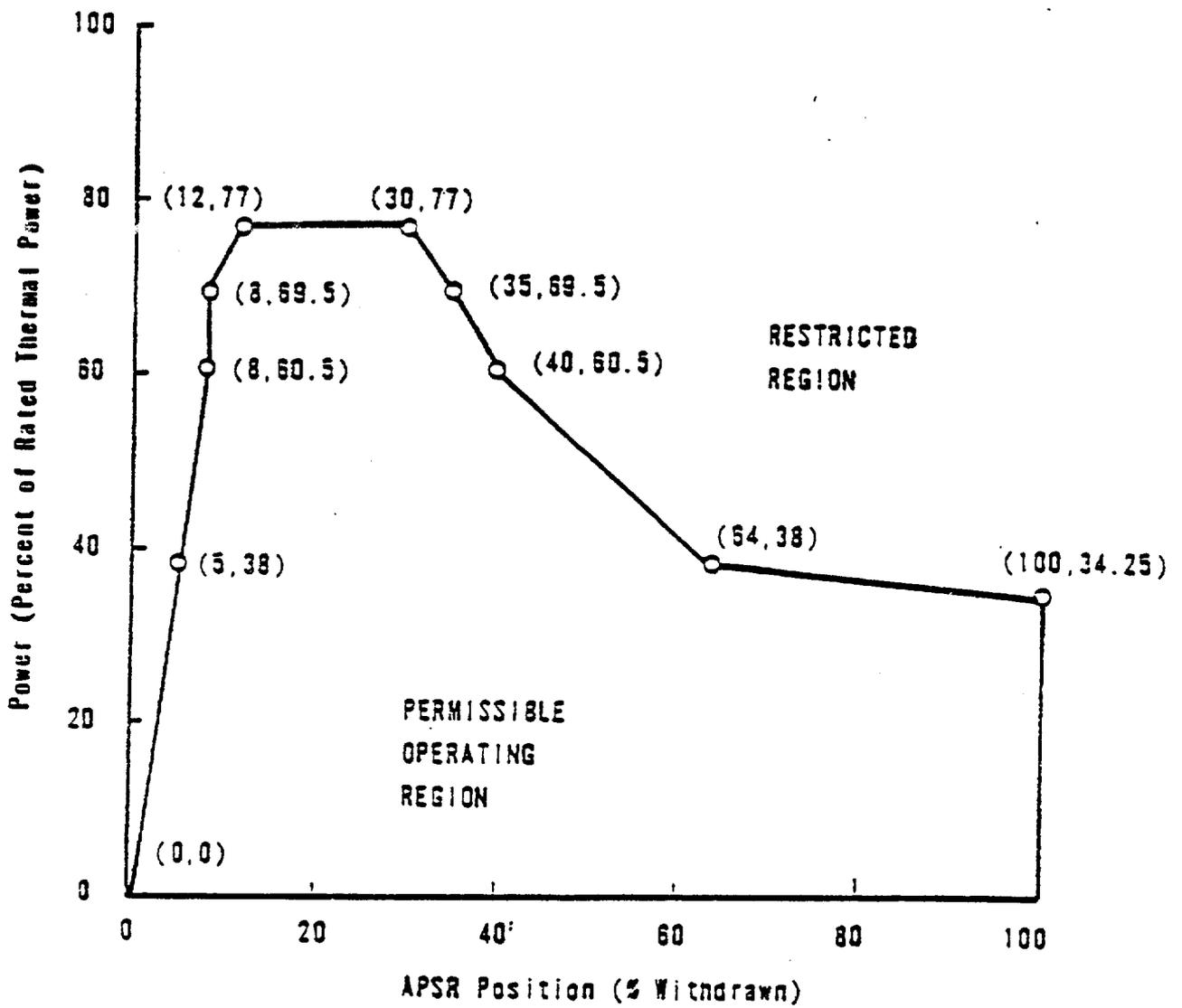
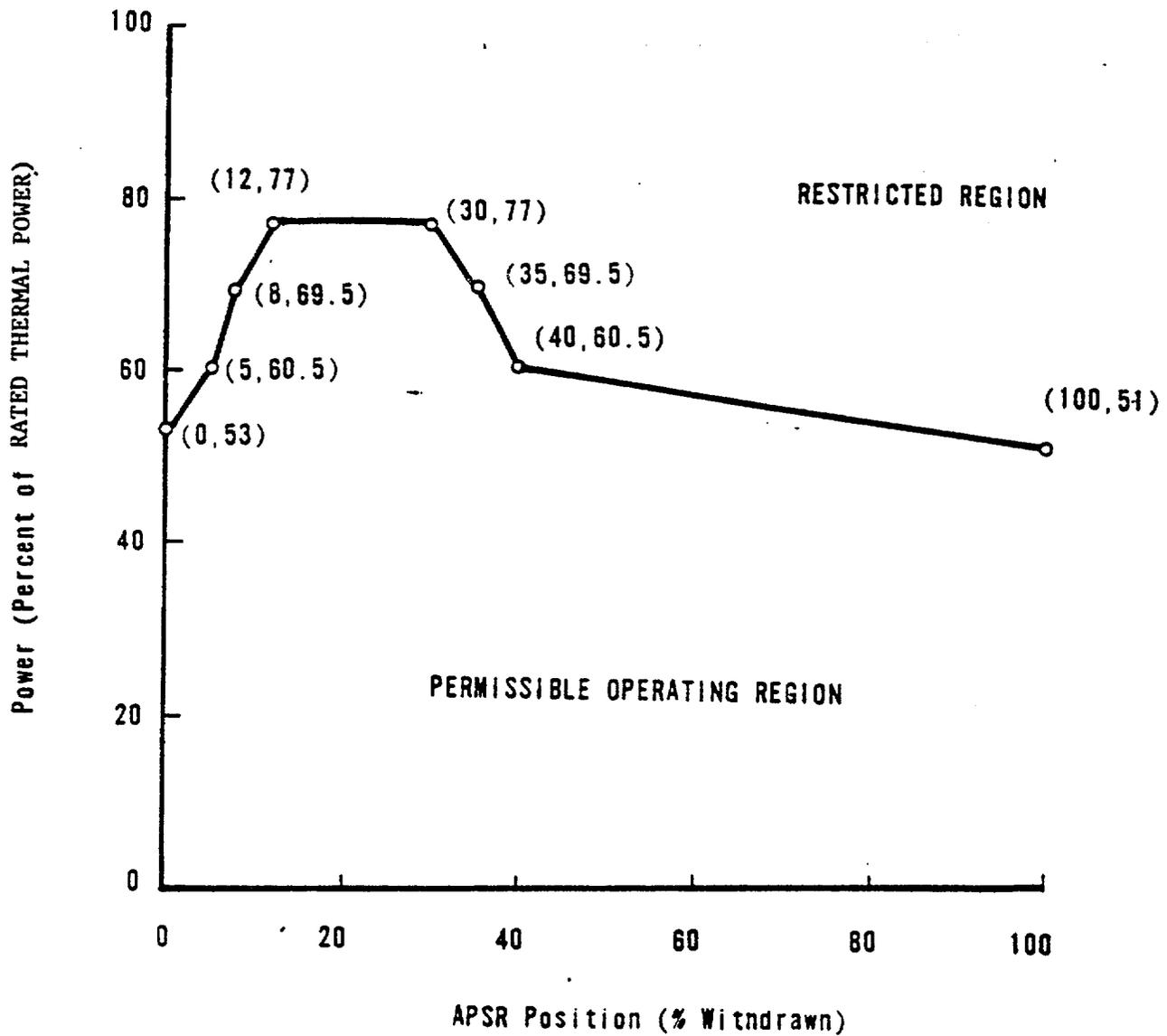
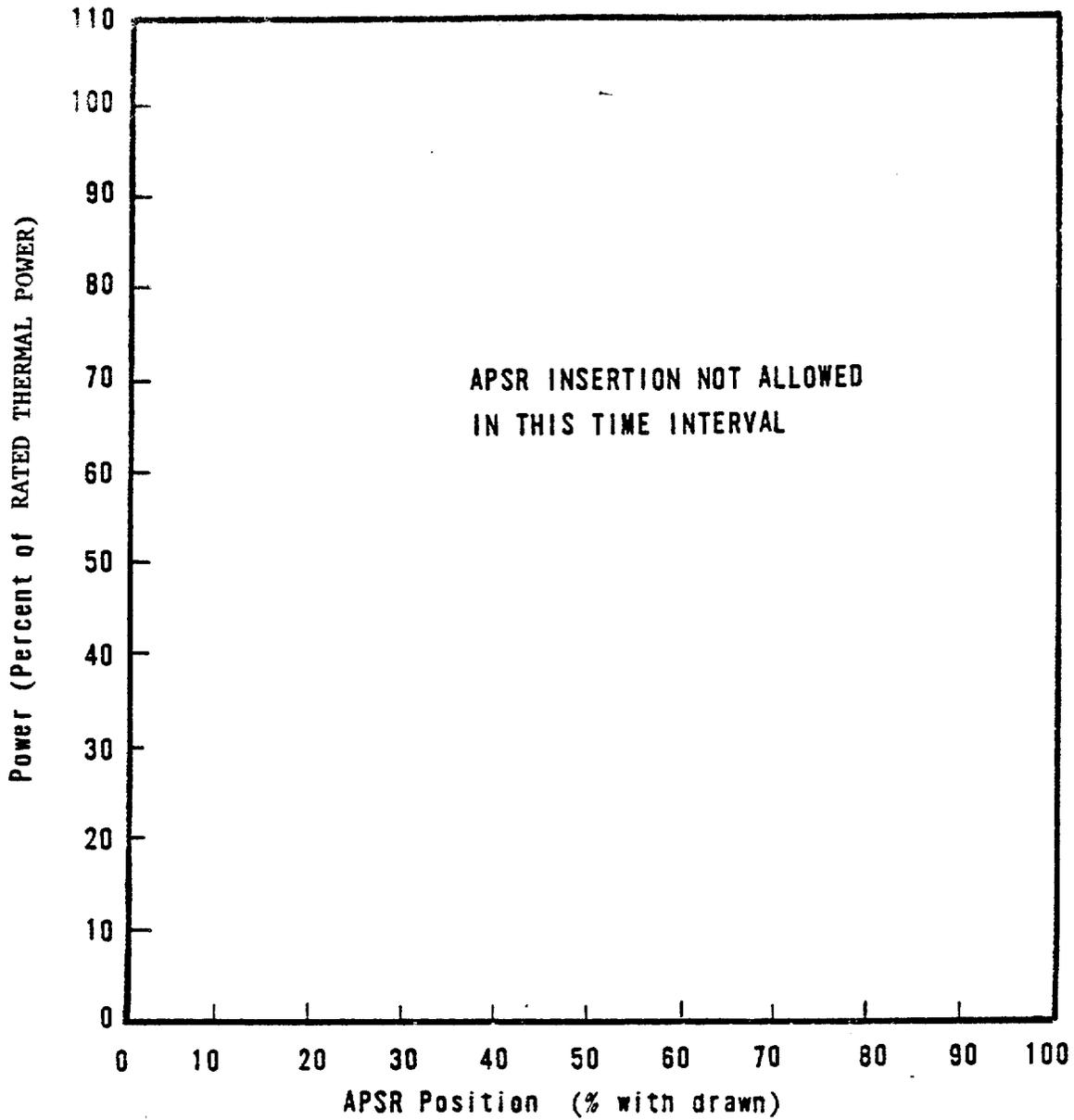


FIGURE 3.1-5 c

APSR Position Limits, 0 to 150 ±10 $\frac{1}{100}$ FPD,
 Three RCPs - Davis-Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.1-5d. APSR Position Limits, 150 ±10 to 250 ±10 EFPD, Three RCPs - Davis-Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.1-5e. APSR Position Limits, After 250 \pm 10 EFPD, Three or Four RCPs - Davis-Besse 1, Cycle 2

3/4.2. POWER DISTRIBUTION LIMITS

AXIAL POWER IMBALANCE

LIMITING CONDITION FOR OPERATION

3.2.1 AXIAL POWER IMBALANCE shall be maintained within the limits shown on Figures 3.2-1a, -1b, -1c and 3.2-2a, -2b, -2c.

APPLICABILITY: MODE 1 above 40% of RATED THERMAL POWER.*

ACTION:

With AXIAL POWER IMBALANCE exceeding the limits specified above, either:

- a. Restore the AXIAL POWER IMBALANCE to within its limits within 15 minutes, or
- b. Within one hour reduce power until imbalance limits are met or to 40% of Rated Thermal Power or less.

SURVEILLANCE REQUIREMENTS

4.2.1 The AXIAL POWER IMBALANCE shall be determined to be within limits at least once every 12 hours when above 40% of RATED THERMAL POWER except when the AXIAL POWER IMBALANCE alarm is inoperable, then calculate the AXIAL POWER IMBALANCE at least once per hour.

*See Special Test Exception 3.10.1.

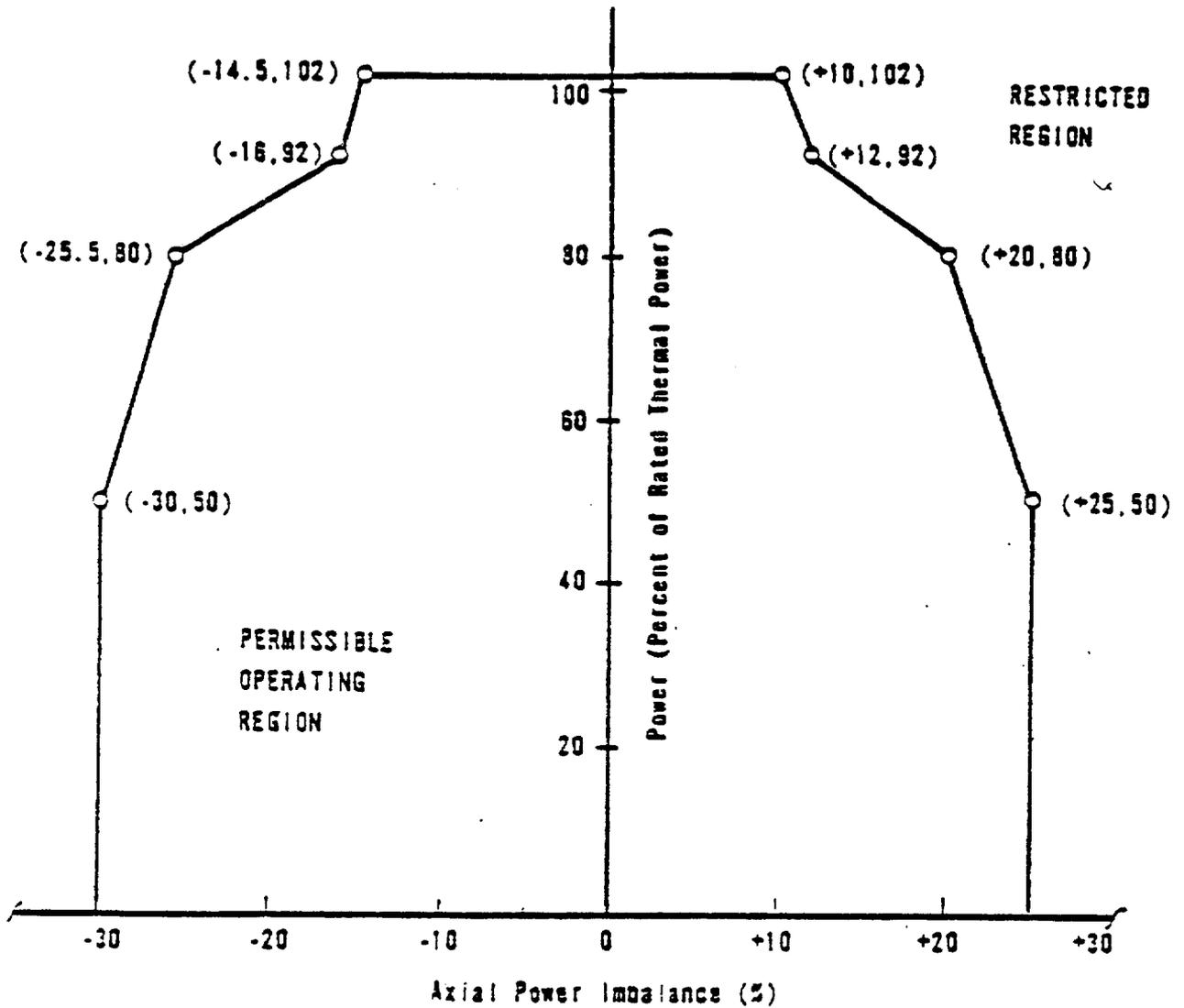
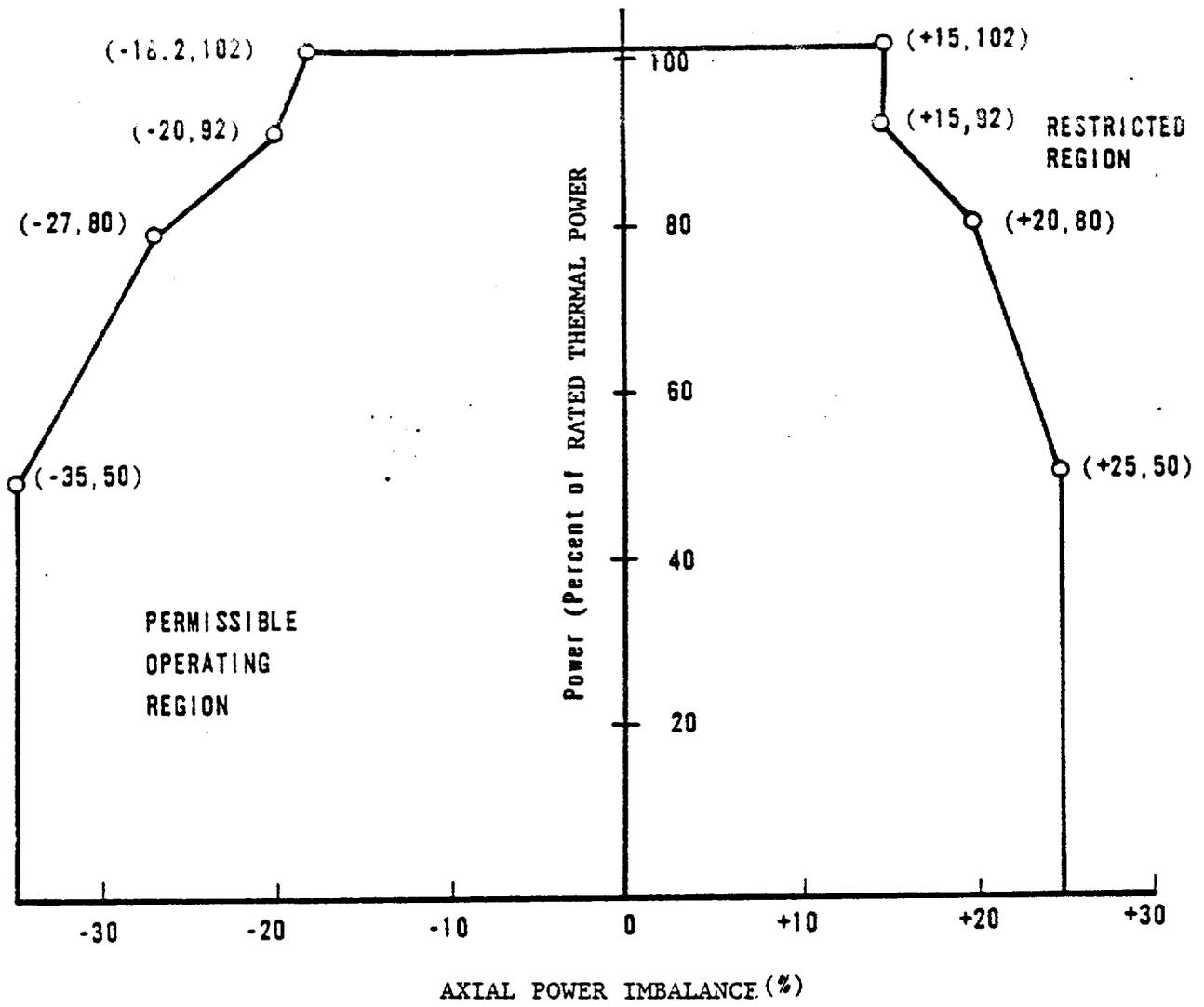
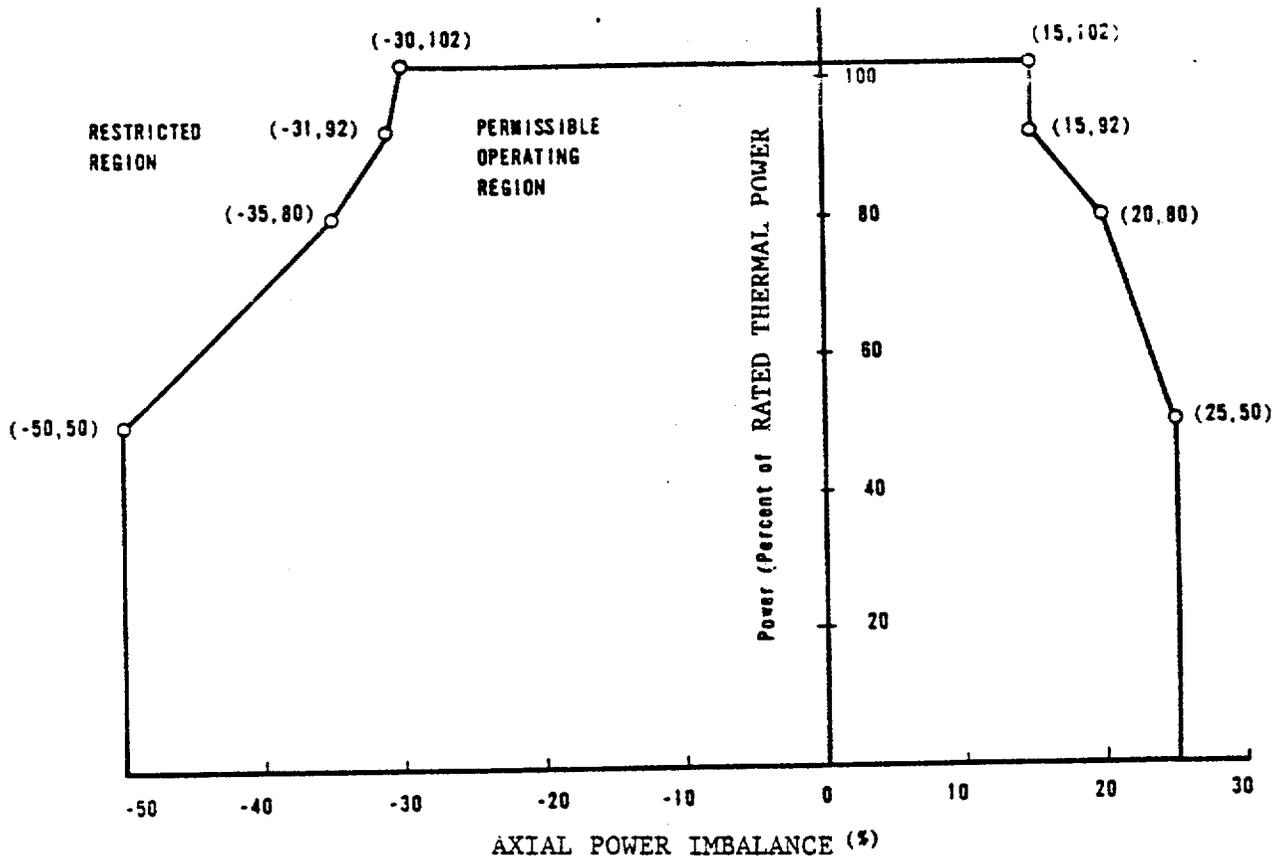


FIGURE 3.2-1a

Axial Power Imbalance Limits, 0 to 150 \pm 10 \bar{M} FPD,
Four RCPs - Davis-Besse 1, Cycle 2

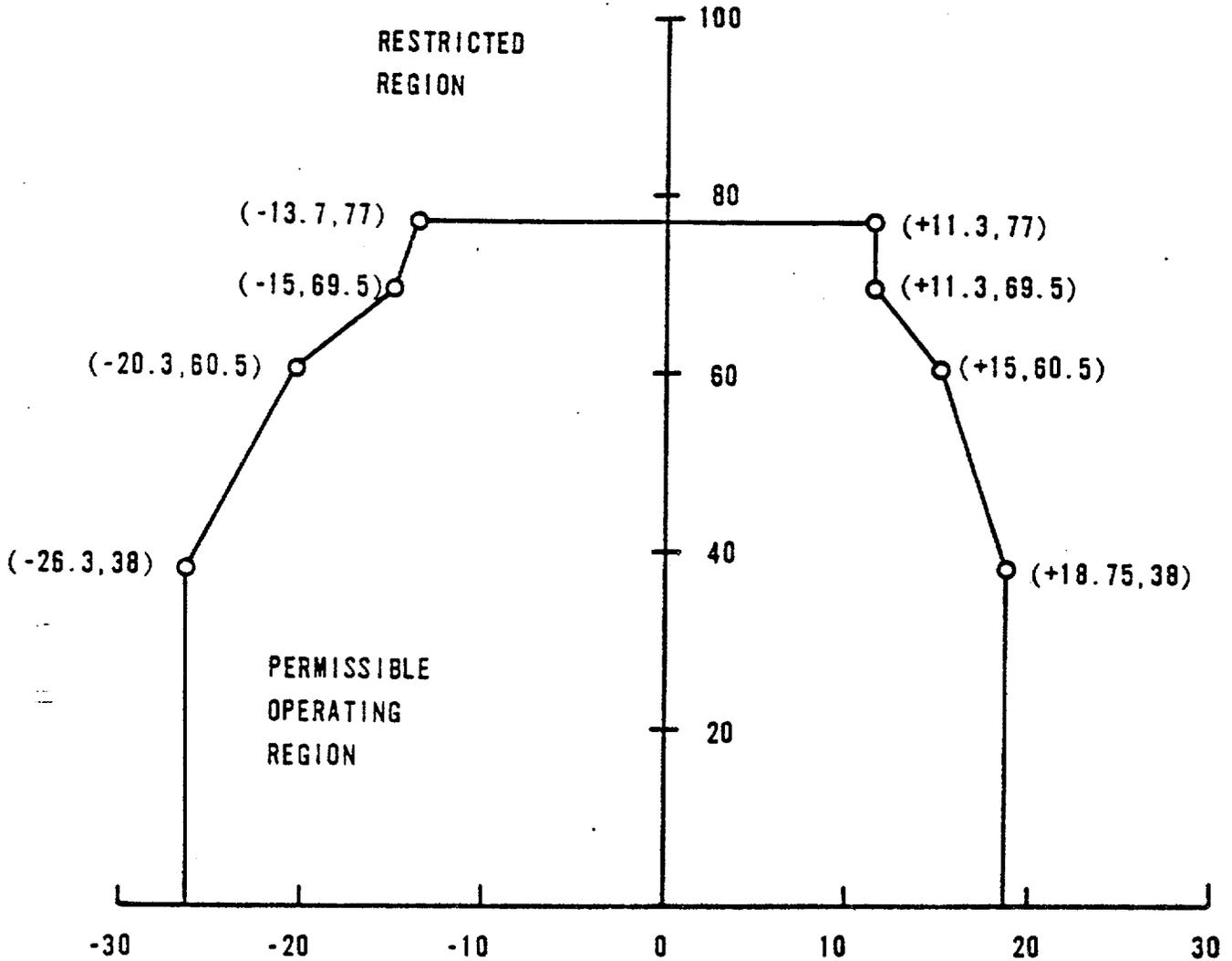


TECHNICAL SPECIFICATION FIGURE 3.2-1b. AXIAL POWER IMBALANCE Limits, 150 ±10 to 250 ±10 EFPD, Four RCPs - Davis-Besse 1, Cycle 2



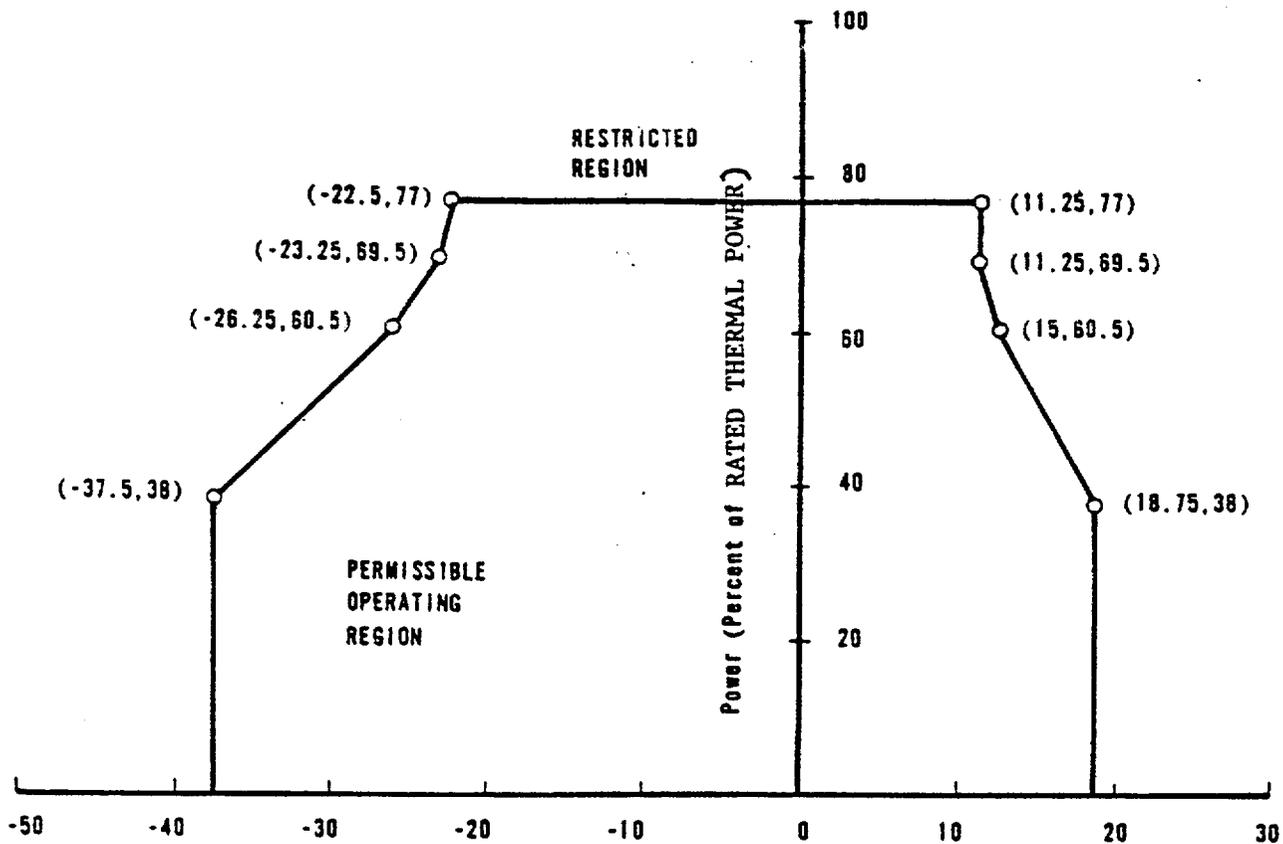
TECHNICAL SPECIFICATION FIGURE 3.2-1c. AXIAL POWER IMBALANCE Limits, After 250 ±10 EFPD, Four RCPs - Davis-Besse 1, Cycle 2

% of RATED THERMAL POWER



AXIAL POWER IMBALANCE (%)

TECHNICAL SPECIFICATION FIGURE 3.2-2b AXIAL POWER IMBALANCE Limits,
150 ± 10 to 250 ± 10 EFPD, Three
RCPs - Davis-Besse 1, Cycle 2



TECHNICAL SPECIFICATION FIGURE 3.2-2c. AXIAL POWER IMBALANCE Limits, After 250 ±10 EFPD, Three RCPs - Davis-Besse 1, Cycle 2

Figure 3.2-3a Deleted

DAVIS-BESSE, UNIT 1

3/4 2-4

Amendment No. ~~11~~, 33



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO.42 TO FACILITY OPERATING LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

1.0 Introduction

By letter of November 5, 1981 (Reference 1), Toledo Edison Company (TECO or the licensee) requested amendment to Facility Operating License NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. The proposed amendment would modify the Technical Specifications (TSs) concerning regulating rod group position limits, axial power shaping rod group position limits, and axial power imbalance limits. Also proposed were modifications to the corrective action required by the TSs in the event axial power imbalance limits are exceeded and to the TS Surveillance Requirement applicable to the axial power shaping rod group position.

The proposed TS modifications would accommodate an extension of the present core cycle length to 306 + 10-30 Effective Full Power Days (EFPD) from the current expected 250 EFPD by allowing complete withdrawal of the axial power shaping rods at 250 EFPD and by coasting down power level towards the end of the cycle. The proposed modification to Cycle 2 operation would be used by TECO only, if upon reaching 250 EFPD, additional at-power operation is needed to enter the scheduled refueling outage period.

The proposed modification to the TS action requirement in the event axial power imbalance is not within allowable limits would relax the requirement to avoid unnecessary hot-shutdown of the unit. The proposed modification to the TS surveillance requirement for the axial power shaping rod group position would make this surveillance requirement similar to the requirement applicable to regulating rod group position.

2.0 Discussion and Evaluation

2.1 Nuclear Analysis

With the exception of the increased cycle length from approximately 250 to 306 EFPD and cycle burnup from 8240 to 10,167 MWd/mtU, there are no significant nuclear parameter differences between the original Cycle 2 design and that proposed for extended Cycle 2 operation. The end-of-cycle (EOC) Doppler coefficient becomes more negative from -1.53 to $-1.58 \times 10^{-5} \Delta k/k/^\circ F$ and the EOC moderator coefficient becomes more negative from -2.73 to $-2.95 \times 10^{-4} \Delta k/k/^\circ F$. Both of these revised key safety analysis parameters,

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as well as all others for Cycle 2, are bounded by the values used in the Final Safety Analysis Report (FSAR).

Analysis of shutdown margin shows that 1.57 percent $\Delta k/k$ exists at EOC compared to the required 1.0 percent $\Delta k/k$ for hot shutdown.

Based on the fact that approved methods have been used to obtain the revised Cycle 2 core characteristics, that margin exists to limiting values of the parameters, and that startup testing was used at the beginning of Cycle 2 to verify important parameters, we find the revised physics parameters for proposed modified Cycle 2 operation acceptable.

2.2 Thermal-Hydraulic Performance

The licensee has indicated that extended Cycle 2 operation is bounded by previously approved FSAR and Cycle 2 analysis with the following exception:

- a) Rod Bow Departure from Nucleate Boiling Ratio (DNBR) Penalty - The rod bow DNBR penalty for extended Cycle 2 operation was calculated using approved methods (Ref. 2). The burnup used to calculate this penalty was 25,564 MWd/mtU compared to a burnup of 23,020 MWd/mtU for a normal Cycle 2. The resulting net rod bow penalty is 1.5% after a credit for one percent flow area reduction factor is included. The flux/flow trip setpoint for Cycle 2 operation and other plant operating limits are based on DNBR criteria that contain sufficient margin to offset the rod bow penalty (Ref. 3). We find this acceptable.

Based on the compatibility of the thermal-hydraulic parameters to values previously accepted for Cycle 2 operation and to the FSAR values, we conclude that the extended Cycle 2 operation of Davis-Besse with currently approved limits is acceptable.

2.3 Technical Specifications

We have reviewed the proposed TS revisions related to the extended operation of Cycle 2 which include the following changes in limiting conditions of operation:

1. Regulating Rod Insertion Limits
2. Axial Power Shaping Rod Insertion Limits
3. Axial Power Imbalance

The same techniques and models were used to derive the revised TSs for limiting conditions of operation as were used to derive those for Cycle 1 and the original Cycle 2 submittal. The modified TSs required to operate beyond 250 EFPD by completely withdrawing the axial power shaping rods and coasting down in power level to the EOC (approximately 306 EFPD) have been reviewed and discussed with Teco. On the basis of these discussions and the fact that previously approved methods are used, we conclude that the new specifications, to accommodate extended Cycle 2 operation to

306 + 10-30 EFPD if required by complete withdrawal of the axial power shaping rods at approximately 250 EFPD and coasting down in power level, are acceptable.

The present TS Limiting Conditions for Operation, 3.2.1, requires that if the axial power imbalance exceeds specified limits, the power imbalance must be restored to within the limits within 15 minutes or the unit must be brought to hot-standby within 2 hours. TS 3.2.1 is applicable only when the unit is operating above 40% of rated thermal power; at 40% or less of rated power there is no Limiting Conditions for Operation applicable to axial power imbalance. The proposed modification to the action requirement would relax the requirement to place the unit in hot-standby by requiring that within one hour the unit's power be reduced until imbalance limits are met or to 40% of rated thermal power or less. Because of the shape of the axial power imbalance limit envelope, a reduction in power may bring axial power imbalance within permissible limits. At 40% of rated power or less, the limiting condition for operation is no longer applicable. This proposed modification reduces the requirement for rapid shutdown of the unit since thermal power can be readily reduced within the specified time to restore axial power imbalance to within limits or to 40% rated power or less. We find this proposed TS modification acceptable.

The present TS Surveillance Requirement, 4.1.3.9, requires that the position of the axial power shaping rod group be determined to be within the insertion limits once every 4 hours. The Station Computer System provides an alarm to the operator whenever the axial power shaping rod group position is not within the acceptable region. The TSs require operation to be brought to within limits within 2 hours or be brought to hot-standby within 6 hours. However, with the Station Computer System alarm operating the operator is immediately alerted to the out-of-limit status of the axial power shaping rod group position and corrective action can be initiated. The proposed modification to the TSs would reduce the Surveillance Requirement to at least once every 12 hours instead of every 4 hours.

If the Station Computer System alarm is inoperative, the Surveillance Requirement would remain at least once every 4 hours. The proposed change would relieve the operator of excessive surveillance and would make the axial power shaping rod group position Surveillance Requirement identical to that for the regulating rod group position. We find this proposed TS modification acceptable.

3.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 23, 1981

REFERENCES

1. Letter, R. P. Crouse (Toledo Edison) to J. F. Stolz (NRC), November 5, 1981.
2. Letter, L. S. Rubenstein (NRC) to J. H. Taylor (B&W), "Evaluation of Interim Procedure for Calculating DNBR Reductions Due to Rod Bow," October 18, 1981.
3. Davis-Besse Nuclear Power Station, Unit 1, Cycle 2 Reload Report, BAW-1598, Rev. 2, Babcock and Wilcox, Lynchburg, VA., September 1981.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-346THE TOLEDO EDISON COMPANYANDTHE CLEVELAND ELECTRIC ILLUMINATING COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 42 to Facility Operating License No. NPF-3, issued to The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees), which revised Technical Specifications (TSs) for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of its date of issuance.

This amendment modifies the TSs concerning regulating rod group position limits, axial power shaping rod group position limits, and axial power imbalance limits. It also modifies 1) the corrective action required by the TSs in the event axial power imbalance limits are exceeded and 2) the surveillance requirement applicable to the axial power shaping rod group position. These modifications will accommodate an extension of the present core cycle length.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

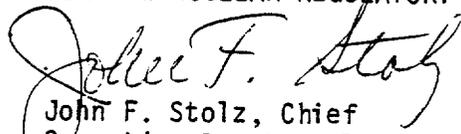
-2-

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 5, 1981, (2) Amendment No. 42 to License No. NPF-3, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, DC, and at the University of Toledo Library Documents Department, 2801 West Bancroft Avenue, Toledo, Ohio 43606. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this, 23rd day of December 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing