

April 12, 1982

Docket No. 50-346



Mr. Richard P. Crouse
Vice President, Nuclear
Toledo Edison Company
Edison Plaza
300 Madison Avenue
Toledo, Ohio 43652

Dear Mr. Crouse:

The Commission has issued the enclosed Amendment No. 44 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in partial response to your application dated March 23, 1979. Action on the remaining portion of this application will be handled separately.

This amendment modifies the TSs concerning containment isolation signals for the reactor coolant pump seal injection and return lines and for the reactor coolant system makeup line.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED BY
JOHN F. STOLZ

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

- 1. Amendment No. 44
- 2. Safety Evaluation
- 3. Notice

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*T. Barnhart
L. Schneider*

ACRS-10

no legal objection to audit and FR notice only

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD-OR:DL	OELD		
SURNAME	RIngram	Garner;cf	JStolz	Novak	S. Tubby		
DATE	6/16/81	6/16/81	6/16/81	6/16/81	6/19/81		

Toledo Edison Company

cc w/enclosure(s):

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U.S. Nuclear Regulatory Commission
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Mrs. Julia Baldwin, Librarian
Government Documents Collection
William Carlson Library
University of Toledo
2801 W. Bancroft Avenue
Toledo, Ohio 43606

Regional Radiation Representative
EPA Region V
230 South Dearborn Street
Chicago, Illinois 60604

cc w/enclosure(s) and incoming dtd.:
3/23/79

Ohio Department of Health
ATTN: Radiological Health
Program Director
P. O. Box 118
Columbus, Ohio 43216



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated March 23, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-3 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

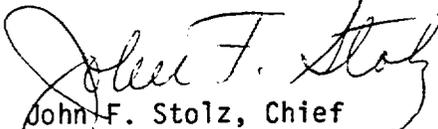
Revise paragraph 2.C.(2) to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.44 , are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page
3/4 3-19

TABLE 3.3-5 (Continued)

SAFETY FEATURES SYSTEM RESPONSE TIMES

INITIATING SIGNAL AND FUNCTION

RESPONSE TIME IN SECONDS

4. RCS Pressure-Low (continued)

i. Containment Isolation Valves

1.	Vacuum Relief	<	30*
2.	Normal Sump	<	25*
3.	RCS Letdown Delay Coil Outlet	<	30*
4.	RCS Letdown High Temperature	<	30*
5.	Pressurizer Sample	<	45*
6.	Service Water to Cooling Water	<	45*
7.	Vent Header	<	15*
8.	Drain Tank	<	15*
9.	Core Flood Tank Vent	<	15*
10.	Core Flood Tank Fill	<	15*
11.	Steam Generator Sample	<	15*
12.	Atmospheric Vent	<	15*
13.	Quench Tank	<	17*
14.	Emergency Sump	<	15*
15.	Air Systems	<	15*
16.	N ₂ System	<	15*
17.	Quench Tank Sample	<	35*
18.	Main Steam Warmup Drain	<	15*
19.	Core Flood Tank Sample	<	15*
20.	RCP Standpipe Demin Water Supply	<	15*
21.	Containment H ₂ Dilution Inlet	<	75*
22.	Containment H ₂ Dilution Outlet	<	75*

j. BWST Outlet Valves NA*

5. RCS Pressure--Low-Low

a. Low Pressure Injection

1.	Decay Heat Pumps	<	30*
2.	Low Pressure Injection Valves	<	NA*
3.	Decay Heat Pump Suction Valves	<	NA*
4.	Decay Heat Cooler Outlet Valves	<	NA*
5.	Decay Heat Cooler Bypass Valves	<	NA*

b. Component Cooling Isolation Valves

1.	Auxiliary Equipment Inlet	<	90*
2.	Inlet to Air Compresso.	<	90*
3.	Component Cooling from Decay Heat Cooler	<	NA*

c. Containment Isolation Valves

1.	RCP Seal Return	<	45*
2.	Makeup	<	30*
3.	RCP Seal Inlet	<	17*

TABLE 3.3-5 (Continued)

SAFETY FEATURES SYSTEM RESPONSE TIMES

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME IN SECONDS</u>
6. Containment Radiation - High	
a. Emergency Vent Fans	≤ 25*
b. HV & AC Isolation Valves	
1. ECCS Room	< 75*
2. Emergency Ventilation	< 75*
3. Containment Air Sample	< 30*
4. Containment Purge	< 15*
5. Penetration Room Purge	< 75*
c. Control Room HV & AC Units	≤ 10*

TABLE NOTATION

- * Diesel generator starting and sequence loading delays included when applicable. Response time limit includes movement of valves and attainment of pump or blower discharge pressure.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. NPF-3

THE TOLEDO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

Introduction

By letter dated March 23, 1979, the Toledo Edison Company (TECo) proposed several amendments to the Davis-Besse Unit No. 1 operating license. One of these proposed amendments would allow a modification of the containment isolation signals for the reactor coolant pump (RCP) seal injection and return lines and for the reactor coolant system (RCS) makeup line.

Discussion

A loss of offsite power at Davis-Besse will cause tripping of the reactor, main turbine, and all RCPs. Code safety valves will open to relieve high pressure in the steam generator caused by the load rejection, and auxiliary feedwater flow will be initiated immediately upon loss of the RCPs. TECo states that these events can cause an RCS cooldown and depressurization sufficient to actuate High Pressure Injection (actuates at pressures less than 1620 psig). This low RCS pressure signal also serves to isolate containment, which includes isolation of the RCP seal injection and return lines and the RCS makeup line. However, TECo states that isolation of seal cooling at RCS pressures greater than 450 psig may damage the seals and subsequently cause their failure. TECo also states that the cooldown following a loss of offsite power can lead to pressurizer level dropping sufficiently to uncover the heaters. TECo is proposing that the isolation signal for these lines be changed from 1620 psig to 420 psig RCS pressure. This will assure continued RCP seal cooling on a loss of offsite power and help control RCS volume so that adequate pressurizer level is maintained.

Evaluation

The effect upon the RCP seals due to loss of seal cooling has been described by TECo in a letter dated December 28, 1979. On the basis of our preliminary review of this information, we consider that isolation of seal cooling for Davis-Besse on a loss of offsite power is undesirable since seal damage could result causing primary coolant leakage into containment equivalent to a small break loss of coolant accident. We also consider that loss of makeup flow due to system isolation on a loss of offsite power is undesirable

since loss of pressurizer level indication could result. Pressurizer level should remain within the indicating band during anticipated transients to assure that the operators are provided with complete information to guide their actions.

We find that changing the isolation signal for the RCP seal injection and return lines from 1620 psig to 420 psig is acceptable since this would help assure RCP operability over a wider range of plant conditions. We also find that the change in the makeup line isolation signal is acceptable since this will help in maintaining pressurizer level within the indicating range. Diversity in isolation signals is maintained with this modification since the high reactor building pressure (4 psig) isolation signal is retained. Thus, protection from fission product release through these lines during a loss of coolant accident will be adequately provided by this diversity.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 12, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-346THE TOLEDO EDISON COMPANYANDTHE CLEVELAND ELECTRIC ILLUMINATING COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 44 to Facility Operating License No. NPF-3, issued to The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees), which revised Technical Specifications (TSs) for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of its date of issuance.

This amendment modifies the TSs concerning containment isolation signals for the reactor coolant pump seal injection and return lines and for the reactor coolant system makeup line.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

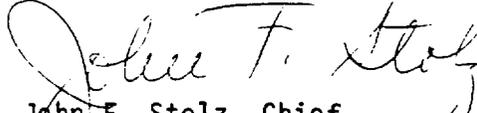
-2-

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated March 23, 1979, (2) Amendment No. 44 to License No. NPF-3, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, DC, and at the William Carlson Library, University of Toledo, 2801 Bancroft Avenue, Toledo, Ohio 43606. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 12th day of April 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing