

MARCH 20 1981

Docket No. 50-346

Mr. Richard P. Crouse  
Vice President, Nuclear  
Toledo Edison Company  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

Dear Mr. Crouse:

Amendment No. 36 to Facility Operating License No. NPF-3, issued January 24, 1981, contained two pages with administrative errors. Please correct these errors by replacing pages 3/4 3-48 (incorrectly numbered 3/4 4-48) and 3/4 5-4 of the Davis-Besse TSs with the enclosed corrected pages. We regret any inconvenience this may have caused.

Sincerely,

Original signed by

John F. Stolz, Chief  
Operating Reactors Branch #4  
Division of Licensing

Enclosure:  
Corrected TS pages  
3/4 3-48 and 3/4 5-4

cc w/enclosure:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket  
50-346

March 20, 1981

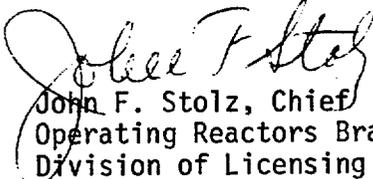
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cc w/enclosure:  
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Toledo Edison Company

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3/4 3-47

TABLE 3.3-10

POST-ACCIDENT MONITORING INSTRUMENTATION

| <u>INSTRUMENT</u>                             | <u>MINIMUM CHANNELS OPERABLE</u> |
|---|----------------------------------|
| 1. SG Outlet Steam Pressure                   | 1/Steam Generator                |
| 2. RC Loop Outlet Temperature                 | 2/Loop                           |
| 3. RC Loop Pressure                           | 2/Loop                           |
| 4. Pressurizer Level                          | 2                                |
| 5. SG Startup Range Level                     | 2/Steam Generator                |
| 6. Auxiliary Feedwater Status                 | 1/AFW System                     |
| 7. Containment Vessel Hydrogen                | 2                                |
| 8. Containment Vessel Post-Accident Radiation | 2                                |
| 9. Containment Vessel Isolation Status        | 1/Valve                          |
| 10. SFAS Status                               | 1/Channel                        |
| 11. Safety Features Equipment Status          | 1/System                         |
| 12. RPS Status                                | 1/Channel                        |
| 13. SFRCS Status                              | 1/Channel                        |
| 14. High Pressure Injection Flow              | 1/Channel                        |

TABLE 3.3-10 (Continued)POST-ACCIDENT MONITORING INSTRUMENTATION

| <u>INSTRUMENT</u>                            | <u>MINIMUM CHANNELS OPERABLE</u> |
|--|----------------------------------|
| 15. Low Pressure Injection (DHR) Flow        | 1/Channel                        |
| 16. HPI System Pump and Valve Status         | 1/System                         |
| 17. LPI System Pump and Valve Status         | 1/System                         |
| 18. Containment Spray Pump and Valve Status  | 1/System                         |
| 19. Core Flood Valve Status                  | 1/System                         |
| 20. BWST Valve Status                        | 1/System                         |
| 21. Containment Emergency Sump Valve Status  | 1/Valve                          |
| 22. Containment Air Recirculation Fan Status | 1/Fan                            |
| 23. Containment Air Cooling Fan Status       | 1/Fan                            |
| 24. EVS Fan and Damper Status                | 1/System                         |
| 25. BWST Level                               | 3                                |

## EMERGENCY CORE COOLING SYSTEMS

ECCS SUBSYSTEMS -  $T_{avg} > 280^{\circ}\text{F}$

### LIMITING CONDITION FOR OPERATION

3.5.2 Two independent ECCS subsystems shall be OPERABLE with each subsystem comprised of:

- a. One OPERABLE high pressure injection (HPI) pump.
- b. One OPERABLE low pressure injection (LPI) pump.
- c. One OPERABLE decay heat cooler, and
- d. An OPERABLE flow path capable of taking suction from the borated water storage tank (BWST) on a safety injection signal and manually transferring suction to the containment sump during the recirculation phase of operation.

APPLICABILITY: MODES 1, 2 and 3.

#### ACTION:

- a. With one ECCS subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 72 hours or be in HCT SHUTDOWN within the next 12 hours.
- b. In the event the ECCS is actuated and injects water into the Reactor Coolant System, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 90 days describing the circumstances of the actuation and the total accumulated actuation cycles to date.

### SURVEILLANCE REQUIREMENTS

4.5.2 Each ECCS subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position.

## EMERGENCY CORE COOLING SYSTEMS

### SURVEILLANCE REQUIREMENTS

- b. At least once per 18 months, or prior to operation after ECCS piping has been drained by verifying that the ECCS piping is full of water by venting the ECCS pump casings and discharge piping high points.
- c. By a visual inspection which verifies that no loose debris (rags, trash, clothing, etc.) is present in the containment which could be transported to the containment emergency sump and cause restriction of the pump suction during LOCA conditions. This visual inspection shall be performed:
  - 1. For all accessible areas of the containment prior to establishing CONTAINMENT INTEGRITY, and
  - 2. Of the areas affected within containment at the completion of each containment entry when CONTAINMENT INTEGRITY is established.
- d. At least once per 18 months by:
  - 1. Verifying that the interlocks:
    - a) Close DH-11 and DH-12 and deenergize the pressurizer heaters, if either DH-11 or DH-12 is open and a simulated reactor coolant system pressure which is greater than the trip setpoint (<438 psig) is applied.
    - b) Prevent the opening of DH-11 and DH-12 when a simulated or actual reactor coolant system pressure which is greater than the trip setpoint (<438 psig) is applied.
  - 2. A visual inspection of the containment emergency sump which verifies that the subsystem suction inlets are not restricted by debris and that the sump components (trash racks, screens, etc.) show no evidence of structural distress or corrosion.
  - 3. Verifying a total leak rate  $\leq$  20 gallons per hour for the LPI system at:
    - a) Normal operating pressure or hydrostatic test pressure of  $\geq$  150 psig for those parts of the system downstream of the pump suction isolation valve, and
    - b)  $\geq$  45 psig for the piping from the containment emergency sump isolation valve to the pump suction isolation valve.
  - 4. Verifying that a minimum of 72 cubic feet of solid granular trisodium phosphate dodecahydrate (TSP) is contained within the TSP storage baskets.

MAY 31 1981