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Docket No. 50-346

Mr. Lowell E. Roe
 Vice President-Facilities
 Development
 Toledo Edison Company
 Edison Plaza
 300 Madison Avenue
 Toledo, Ohio 43652

Dear Mr. Roe:

The Commission has issued the enclosed Amendment No. 21 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated July 27, 1979, as supplemented August 17, 1979.

This amendment changes the Technical Specifications by (1) extension of the calendar time to perform the first inservice inspections (ISI) of steam generator tubes after initial criticality, (2) special provisions for certain categorization of degraded and/or defective steam generator tubes in various defined groups, and (3) inspection interval requirements based on the categorization of the results of inspections of steam generator tubes.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
 Robert W. Reid

Robert W. Reid, Chief
 Operating Reactors Branch #4
 Division of Operating Reactors

- Enclosures:
1. Amendment No. 21
 2. Safety Evaluation
 3. Notice

cc w/enclosures:
 See next page

ORB#4 DOR
 M. Fairfile
 MBF 9/17/79

GP
 2
 concurrence in
 amendment +
 FR notice only

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OFFICE	ORB#4: DOR	ORB#4: DOR	C-ORB#4: DOR	AD-ORP: DOR	OELD	STSG
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DATE	9/17/79	9/17/79	9/18/79	9/18/79	9/28/79	9/28/79



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 2, 1979

Docket No. 50-346

Mr. Lowell E. Roe
Vice President-Facilities
Development
Toledo Edison Company
Edison Plaza
300 Madison Avenue
Toledo, Ohio 43652

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Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, reading "Robert W. Reid".

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

1. Amendment No. 21
2. Safety Evaluation
3. Notice

cc w/enclosures:
See next page

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Atomic Safety & Licensing Board Panel
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Atomic Safety and Licensing Appeal Panel
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Toledo Edison Company

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cc w/enclosure(s) and incoming
dtd.: 7/27/79 & 8/17/79

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450 East Town Street
Columbus, Ohio 43216



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 21
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated July 27, 1979, as supplemented August 17, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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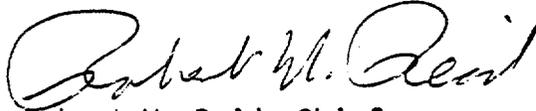
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 21, are hereby incorporated in the license. Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 2, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 21

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 4-6
3/4 4-7
3/4 4-8
3/4 4-9
3/4 4-10
3/4 4-12

REACTOR COOLANT SYSTEM

PRESSURIZER

LIMITING CONDITION FOR OPERATION

3.4.4 The pressurizer shall be OPERABLE with:

- a. A steam bubble,
- b. A water level between 45 and 305 inches.

APPLICABILITY: MODES 1 and 2.

ACTION:

With the pressurizer inoperable, be in at least HOT STANDBY with the control rod drive trip breakers open within 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.4 The pressurizer shall be demonstrated OPERABLE by verifying pressurizer level to be within limits at least once per 12 hours.

REACTOR COOLANT SYSTEM

STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE with a water level between 18 and 348 inches.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one or more steam generators inoperable due to steam generator tube imperfections, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.
- b. With one or more steam generators inoperable due to the water level being outside the limits, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the next 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

4.4.5.1. Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. The first sample inspection during each inservice inspection (subsequent to the baseline inspection) of each steam generator shall include:
 1. All nonplugged tubes that previously had detectable wall penetrations (> 20%).
 2. At least 50% of the tubes inspected shall be in those areas where experience has indicated potential problems.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

3. A tube inspection (pursuant to Specification 4.4.5.4.a.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a full tube inspection.
- b. Tubes in the following groups may be excluded from the first random sample if all tubes in a group in both steam generators are inspected. No credit will be taken for these tubes in meeting minimum sample size requirements.
 1. Group A-1: Tubes within one, two or three rows of the open inspection lane.
 2. Group A-2: Tubes having a drilled opening in the 15th support plate.
- c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to less than a full tube inspection provided:
 1. The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.
 2. The inspections include those portions of the tubes where imperfections were previously found.

The results of each sample inspection shall be classified into one of the following three categories:

<u>Category</u>	<u>Inspection Results</u>
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- Notes: (1) In all inspections, previously degraded tubes must exhibit significant (> 10%) further wall penetrations to be included in the above percentage calculations.
- (2) Where special inspections are performed pursuant to 4.4.5.2.b, defective or degraded tubes found as a result of the inspection shall be included in determining the Inspection Results Category for that special inspection but need not be included in determining the Inspection Results Category for the general steam generator inspection.

4.4.5.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The baseline inspection shall be performed to coincide with the first scheduled refueling outage but no later than April 30, 1980. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If the results of two consecutive inspections for a given group* of tubes following service under all volatile treatment (AVT) conditions fall into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval for that group may be extended to a maximum of 40 months.
- b. If the results of the inservice inspection of a steam generator performed in accordance with Table 4.4-2 at 40 month intervals for a given group* of tubes fall in Category C-3, subsequent inservice inspections shall be performed at intervals of not less than 10 nor more than 20 calendar months after the previous inspection. The increase in inspection frequency shall apply until a subsequent inspection meets the conditions specified in 4.4.5.3a and the interval can be extended to 40 months.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions:
 1. Primary-to-secondary tube leaks (not including leaks originating from tube-to tube sheet welds) in excess of the limits of Specification 3.4.6.2.

*A group of tubes means:

- (a) All tubes inspected pursuant to 4.4.5.2.b, or
- (b) All tubes in a steam generator less those inspected pursuant to 4.4.5.2.b.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

2. A seismic occurrence greater than the Operating Basis Earthquake.
 3. A loss-of-coolant accident requiring actuation of the engineered safeguards.
 4. A main steam line or feedwater line break.
- d. The provisions of Specification 4.0.2 are not applicable.

4.4.5.4 Acceptance Criteria

- a. As used in this Specification:

1. Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.
2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube.
3. Degraded Tube means a tube containing imperfections $\geq 20\%$ of the nominal wall thickness caused by degradation.
4. % Degradation means the percentage of the tube wall thickness affected or removed by degradation.
5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective.
6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 40% of the nominal tube wall thickness.
7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.3.c, above.
8. Tube Inspection means an inspection of the steam generator tube from the point of entry completely to the point of exit.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

9. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service to establish a baseline condition of the tubing. This inspection shall be performed prior to initial POWER OPERATION using the equipment and techniques expected to be used during subsequent inservice inspections.

b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2

4.4.5.5 Reports

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

b. The complete results of the steam generator tube inservice inspection shall be submitted on an annual basis in a report for the period in which this inspection was completed. This report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

3. Identification of tubes plugged.

c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the Commission shall be reported pursuant to Specification 6.9.1 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

4.4.5.6 The steam generator shall be demonstrated OPERABLE by verifying steam generator level to be within limits at least once per 12 hours.

TABLE 4.4-1
MINIMUM NUMBER OF STEAM GENERATORS TO BE
INSPECTED DURING INSERVICE INSPECTION

Preservice Inspection	No			Yes		
	Two	Three	Four	Two	Three	Four
No. of Steam Generators per Unit						
First Inservice Inspection	All			One	Two	Two
Second & Subsequent Inservice Inspections	One ¹			One ¹	One ²	One ³

Table Notation:

1. The inservice inspection may be limited to one steam generator on a rotating schedule encompassing 3 N % of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the sample sequence shall be modified to inspect the most severe conditions.
2. The other steam generator not inspected during the first inservice inspection shall be inspected. The third and subsequent inspections should follow the instructions described in 1 above.
3. Each of the other two steam generators not inspected during the first inservice inspections shall be inspected during the second and third inspections. The fourth and subsequent inspections shall follow the instructions described in 1 above.

TABLE 4.4-2

STEAM GENERATOR TUBE INSPECTION (2)

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S. G. (1)	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect additional 2S tubes in this S. G.	C-1	None	N/A	N/A
			C-2	Plug defective tubes and inspect additional 4S tubes in this S. G.	C-1	None
			C-3	Perform action for C-3 result of first sample	C-2	Plug defective tubes
	C-3	Inspect all tubes in this S. G., plug de- fective tubes and inspect 2S tubes in each other S. G. Prompt notification to NRC pursuant to specification 6.9.1	C-3	Perform action for C-3 result of first sample	C-3	Perform action for C-3 result of first sample
			All other S. G.s are C-1	None	N/A	N/A
			Some S. G.s C-2 but no additional S. G. are C-3	Perform action for C-2 result of second sample	N/A	N/A
		Additional S. G. is C-3	Inspect all tubes in each S. G. and plug defective tubes. Prompt notification to NRC pursuant to specification 6.9.1	N/A	N/A	

(1) $S = 3 \frac{N}{n} \%$ Where N is the number of steam generators in the unit, and n is the number of steam generators inspected during an inspection

(2) For tubes inspected pursuant to 4.4.5.2.b: No action is required for C-1 results. For C-2 results in one or both steam generators plug defective tubes. For C-3 results in one or both steam generators, plug defective tubes and provide prompt notification of NRC pursuant to Specification 6.6,



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NO. NPF-3

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

DOCKET NO. 50-346

Introduction

By letter dated July 27, 1979, supplemented by letter dated August 17, 1979, the Toledo Edison Company (TECO or the licensee) requested an amendment to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit No. 1 (Davis-Besse 1).

The amendment would change the Technical Specifications (TSs) by (1) extension of the calendar time to perform the first inservice inspections (ISI) of steam generator (SG) tubes after initial criticality, (2) special provisions for certain categorization of degraded and/or defective SG tubes in various defined groups, and (3) inspection interval requirements based on the categorization of the results of inspections of SG tubes.

Discussion

With regard to item (1) above, Section 4.4.5.3.a of the TSs requires that the first ISI of the SG tubes be performed after six effective full power months, but within 24 calendar months of initial criticality. Because Davis-Besse 1 has incurred a number of plant outages (seven months, 22 days - or approximately 232 effective full power days) since initial criticality on August 12, 1977, the first scheduled refueling outage has slipped to March 1980. Therefore, the licensee has requested relief from the TS provisions to permit the first ISI to be coincident with the first scheduled refueling outage.

With respect to item (2) above, operating experience to date with Babcock and Wilcox (B&W) designed SGs indicates that tube degradation is most likely to occur in localized areas adjacent to the tube inspection lane and in the vicinity of the 15th tube support plate where tubes pass through drilled, as opposed to broached, holes*. It is believed that degradation preferentially occurs in these areas because of the local combination of flow conditions and fluid properties. The current TS for SG tubes requires, and the proposed TS change would require,

* A broached hole is typically a fluted circle rather than a plain circle.

that 50% of the first sample of tubes selected for inspection (3% of the total number of tubes in all SGs) be from this area (where experience indicates critical areas to be inspected). As an alternative to this requirement, the licensee proposed to define one or more areas in the SGs where experience has indicated that degradation is most likely, and to optionally perform an inspection of all of the tubes in these areas in both SGs. In addition, the licensee would inspect the tubes not so inspected in accordance with the general provisions of the proposed TS. The criteria for the second and third sample inspections for the general SG group would be based on the results of the general SG tube group inspection, independent of the defined group inspection results when the defined group would be 100% inspected. According to the licensee's proposal, the number of tubes inspected in the defined potential problem area(s) would not reduce the number of tubes examined in the associated general inspection; but at the same time, degraded or defective tubes identified in defined potential problem areas would not be used in determining the results category for the general inspection and vice versa.

With regard to item (3) above, the current TS requires the inspection results to be classified into one of three categories (Specification 4.4.5.2) depending upon the number of defective or degraded tubes discovered. This results category determines the repairs that must be performed; the additional inspection required at that outage, if any; whether prompt reporting of the results to the NRC is required; and the maximum permissible interval until the next inspection is conducted. The licensee is proposing that the maximum inspection interval to the next ISI be determined separately for each SG tube group based upon the results category for that group. This will provide for an increased frequency of inspection for a problem area, without reducing the frequency of general SG tube inspections.

Evaluation

As for item (1), the licensee is requesting relief from Specification 4.4.5.3.a to permit the first ISI to coincide with the first refueling outage now scheduled for March 1980. This represents approximately a seven-month extension and is justified by the licensee on the basis of the total accumulated outage time of seven months 22 days since initial criticality. It is our evaluation that any tube degradation which may have occurred during the periods of wet layup has not added significantly to the tube degradation that would normally be incurred in a two-year interval. The most significant form of tube degradation observed in B&W units to date is circumferential cracks induced by fatigue. These fatigue cracks are due to mechanical sources such as flow induced vibration which does not exist during periods of wet layup. Experience has shown that tube degradation due to corrosion mechanisms such as wastage and denting is not a problem for B&W units. Indeed, the denting phenomena has yet to be observed in any B&W unit. Therefore, we find it acceptable to extend the specified 24-calendar month interval before the first inservice inspection by the seven month 22 day outage time accumulated to date.

To ensure that any subsequent outages which may occur prior to March 1980 will not result in further slippage of the first ISI, the licensee has agreed to a provision in the TSs which states that the first ISI shall be performed by April 30, 1980.

As for item (2), the licensee is proposing that the tubes in the SGs be classified into two groups: (1) a group of tubes in well-defined areas where experience has indicated that tube degradation is most likely (the defined group)

and (2) the balance of the tubes in the SGs. The licensee is also proposing that, at their option, these groups may be subject to different inspection requirements. Specifically, the licensee may or may not elect to perform an inspection of every tube in the defined group in both SGs. If they elect to perform such an inspection, the balance of the SG tubes will be subject to the normal inspection requirements with no reduction of sample size. At the same time, degraded or defective tubes identified within the defined group will only be used to establish the results category for that group and not for the overall population of tubes.

On the other hand, if the licensee elects to not inspect every tube in the defined group in both SGs, the specifications would require that the normal inspection be performed. In this case, the specifications would require that at least 50% of the tubes inspected be in areas where experience has indicated potential problems. Accordingly, with either option, inspection of tubes in potential problem areas is emphasized. Under the provisions of the licensee's proposed revision, however, all of the tubes in these areas may be inspected. Therefore, we conclude that with the proposed revision the extent of the inspection of tubes in potential problem areas is not diminished and may be increased. In addition, we conclude that the extent of the inspection of the balance of the SG tubes is not reduced.

As for item (3), we find the proposed requirements governing inspection intervals of the inspection of a defined group of tubes would increase the number of inspections of a problem area, yet not reduce the general SG tube inspections.

Based upon the foregoing, we conclude that extending the 24 calendar month interval before the first inservice inspection by the outage time accumulated to date does not significantly increase the probability of a tube failure prior to the first inspection more than what currently exists for the specified 24 calendar month interval assuming no outages. Thus, this change to the TSs is acceptable. We further conclude that the inclusion in the TSs of provisions for (1) electively inspecting all tubes in defined areas and (2) determining the inspection intervals for the defined areas from the inspection results of the respective defined areas does not reduce the effectiveness of the overall SG tube inspection program and is therefore acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 2, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-346

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING
COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 21 to Facility Operating License No. NPF-3, issued to The Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees), which revised Technical Specifications for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specifications by (1) extension of the calendar time to perform the first inservice inspections (ISI) of steam generator tubes after initial criticality, (2) special provisions for certain categorization of degraded and/or defective steam generator tubes in various defined groups, and (3) inspection interval requirements based on the categorization of the results of inspections of steam generator tubes.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR § 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 27, 1979, as supplemented August 17, 1979, (2) Amendment No. 21 to License No. NPF-3, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Ida Rupp Public Library, 310 Madison Street, Port Clinton, Ohio.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 2nd day of October 1979.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors