

February 16, 1999

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M99550 AND M99551)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 171 to Facility Operating License No. DPR-19 and Amendment No. 166 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3. The amendments are in response to your application dated August 14, 1998, as supplemented by letters dated October 13, 1998, and December 23, 1998.

The amendments revise the Dresden Technical Specifications (TS) to reflect the use of Siemens Power Corporation ATRIUM-9B fuel. Specifically the amendments incorporate the following into the TS: (a) new methodologies that will enhance operational flexibility and reduce the likelihood of future plant derates; (b) administrative changes that eliminate the cycle-specific implementation of ATRIUM-9B fuel and adopt Improved Standard Technical Specification language where appropriate; and (c) changes to the Minimum Critical Power Ratio.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIG. SIGNED BY

Lawrence W. Rossbach, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000237
P PDR

Docket Nos. 50-237 and 50-249

Enclosures: 1. Amendment No. 171 to DPR-19
2. Amendment No. 166 to DPR-25
3. Safety Evaluation

cc w/encls: See next page

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Lawrence W. Rossbach, Project Manager
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 16, 1999

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
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A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Lawrence W. Rossbach".

Lawrence W. Rossbach, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-237 and 50-249

Enclosures: 1. Amendment No. 171 to DPR-19
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3. Safety Evaluation

cc w/encls: see next page

O. Kingsley
Commonwealth Edison Company

Dresden Nuclear Power Station
Units 2 and 3

cc:

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Downers Grove, Illinois 60515



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171
License No. DPR-19

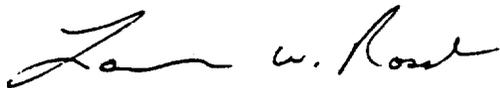
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated August 14, 1998, as supplemented on October 13, 1998, and December 23, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 171, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Lawrence W. Rossbach, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 16, 1999



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 166
License No. DPR-25

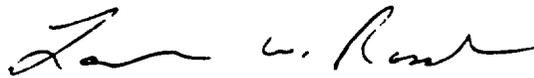
1. The Nuclear Regulatory Commission (the Commission) has found that:
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 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 166, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Lawrence W. Rossbach, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 16, 1999

ATTACHMENT TO LICENSE AMENDMENT NOS. 171 AND 166

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

I
1-1
2-1
3/4.11-1
3/4.11-4
5-5
6-16

INSERT

I
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2-1
3/4.11-1
3/4.11-4
5-5
6-16

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1.0 DEFINITIONS

The following terms are defined so that uniform interpretation of these specifications may be achieved. The defined terms appear in capitalized type and shall be applicable throughout these Technical Specifications.

ACTION

ACTION shall be that part of a Specification which prescribes remedial measures required under designated conditions.

AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)

The **AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)** shall be applicable to a specific planar height and is equal to the sum of the **LINEAR HEAT GENERATION RATE(s)** for all the fuel rods in the specified bundle at the specified height divided by the number of fuel rods in the fuel bundle.

CHANNEL

A **CHANNEL** shall be an arrangement of a sensor and associated components used to evaluate plant variables and generate a single protective action signal. A **CHANNEL** terminates and loses its identity where single action signals are combined in a **TRIP SYSTEM** or logic system.

CHANNEL CALIBRATION

A **CHANNEL CALIBRATION** shall be the adjustment, as necessary, of the **CHANNEL** output such that it responds with the necessary range and accuracy to known values of the parameter which the **CHANNEL** monitors. The **CHANNEL CALIBRATION** shall encompass the entire **CHANNEL** including the required sensor and alarm and/or trip functions, and shall include the **CHANNEL FUNCTIONAL TEST**. The **CHANNEL CALIBRATION** may be performed by any series of sequential, overlapping or total **CHANNEL** steps such that the entire **CHANNEL** is calibrated.

CHANNEL CHECK

A **CHANNEL CHECK** shall be the qualitative assessment of **CHANNEL** behavior during operation by observation. This determination shall include, where possible, comparison of the **CHANNEL** indication and/or status with other indications and/or status derived from independent instrument **CHANNEL(s)** measuring the same parameter.

2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS**THERMAL POWER, Low Pressure or Low Flow**

2.1.A THERMAL POWER shall not exceed 25% of RATED THERMAL POWER with the reactor vessel steam dome pressure less than 785 psig or core flow less than 10% of rated flow.

APPLICABILITY: OPERATIONAL MODE(s) 1 and 2.

ACTION:

With THERMAL POWER exceeding 25% of RATED THERMAL POWER and the reactor vessel steam dome pressure less than 785 psig or core flow less than 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours and comply with the requirements of Specification 6.7.

THERMAL POWER, High Pressure and High Flow

2.1.B The MINIMUM CRITICAL POWER RATIO (MCPR) shall not be less than 1.10 for Unit 3 and 1.09 for Unit 2 with the reactor vessel steam dome pressure greater than or equal to 785 psig and core flow greater than or equal to 10% of rated flow. During single recirculation loop operation, this MCPR limit shall be increased by 0.01.

APPLICABILITY: OPERATIONAL MODE(s) 1 and 2.

ACTION:

With MCPR less than the above applicable limit and the reactor vessel steam dome pressure greater than or equal to 785 psig and core flow greater than or equal to 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours and comply with the requirements of Specification 6.7.

3.11 - LIMITING CONDITIONS FOR OPERATION

A. AVERAGE PLANAR LINEAR HEAT GENERATION RATE

All AVERAGE PLANAR LINEAR HEAT GENERATION RATES (APLHGR) shall not exceed the limits specified in the CORE OPERATING LIMITS REPORT.

APPLICABILITY:

OPERATIONAL MODE 1, when THERMAL POWER is greater than or equal to 25% of RATED THERMAL POWER.

ACTION:

With an APLHGR exceeding the limits specified in the CORE OPERATING LIMITS REPORT:

1. Initiate corrective action within 15 minutes, and
2. Restore APLHGR to within the required limit within 2 hours.

With the provisions of the ACTION above not met, reduce THERMAL POWER to less than 25% of RATED THERMAL POWER within the next 4 hours.

4.11 - SURVEILLANCE REQUIREMENTS

A. AVERAGE PLANAR LINEAR HEAT GENERATION RATE

The APLHGRs shall be verified to be equal to or less than the limits specified in the CORE OPERATING LIMITS REPORT.

1. At least once per 24 hours,
2. Within 12 hours after completion of a THERMAL POWER increase of at least 15% of RATED THERMAL POWER, and
3. Initially and at least once per 12 hours when the reactor is operating with a LIMITING CONTROL ROD PATTERN for APLHGR.
4. The provisions of Specification 4.0.D are not applicable.

3.11 - LIMITING CONDITIONS FOR OPERATION

4.11 - SURVEILLANCE REQUIREMENTS

D. STEADY STATE LINEAR HEAT GENERATION RATE

D. STEADY STATE LINEAR HEAT GENERATION RATE

The LINEAR HEAT GENERATION RATE (LHGR) shall not exceed the STEADY STATE LINEAR HEAT GENERATION RATE (SLHGR) limits specified in the CORE OPERATING LIMITS REPORT.

The SLHGR shall be determined to be equal to or less than the limit:

APPLICABILITY:

OPERATIONAL MODE 1, when THERMAL POWER is greater than or equal to 25% of RATED THERMAL POWER.

1. At least once per 24 hours,
2. Within 12 hours after completion of a THERMAL POWER increase of at least 15% of RATED THERMAL POWER, and
3. Initially and at least once per 12 hours when the reactor is operating with a LIMITING CONTROL ROD PATTERN for SLHGR.
4. The provisions of Specification 4.0.D are not applicable.

ACTION:

With an LHGR exceeding the SLHGR limits specified in the CORE OPERATING LIMITS REPORT:

1. Initiate corrective ACTION within 15 minutes, and
2. Restore the LHGR to within the SLHGR limit within 2 hours.

With the provisions of the ACTION above not met, reduce THERMAL POWER to less than 25% of RATED THERMAL POWER within the next 4 hours.

5.0 DESIGN FEATURES

5.3 REACTOR CORE**Fuel Assemblies**

5.3.A The reactor core shall contain 724 fuel assemblies. Each assembly consists of a matrix of Zircaloy clad fuel rods with an initial composition of natural or slightly enriched uranium dioxide as fuel material. The assemblies may contain water rods or a water box. Limited substitutions of Zircaloy or ZIRLO or stainless steel filler rods for fuel rods, in accordance with NRC-approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with applicable NRC staff-approved codes and methods, and shown by tests or analyses to comply with all fuel safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in non-limiting core regions.

Control Rod Assemblies

5.3.B The reactor core shall contain 177 cruciform shaped control rod assemblies. The control material shall be boron carbide powder (B_4C) and/or hafnium metal. The control rod assembly shall have a nominal axial absorber length of 143 inches.

ADMINISTRATIVE CONTROLS

(12) ANF-1125 (P)(A), ANFB Critical Power Correlation Determination of ATRIUM-9B Additive Constant Uncertainties, Supplement 1, Appendix E, Siemens Power Corporation, September 1998.

- c. The core operating limits report shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met. The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions of supplements thereto shall be provided on issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

6.9.B Special Reports

Special reports shall be submitted to the Regional Administrator of the NRC Regional Office within the time period specified for each report.

6.10 INTENTIONALLY LEFT BLANK



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 171 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 166 TO FACILITY OPERATING LICENSE NO. DPR-25
COMMONWEALTH EDISON COMPANY
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3
DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated August 14, 1998 (Reference 1) Commonwealth Edison Company (ComEd, the licensee) requested changes to the Technical Specifications (TS) for Dresden Nuclear Power Station, Units 2 and 3. Additional information was submitted by letters dated October 13, 1998 (Reference 2) and December 23, 1998 (Reference 3). The December 23, 1998, submittal provided additional clarifying information that did not change the initial proposed no significant hazards consideration determination. The proposed changes are due to the transition to Siemens Power Corporation (SPC) ATRIUM-9B fuel. The key items are:

1. Incorporation of SPC's new methodology that will enhance operational flexibility and reduce the likelihood of future plant derates,
2. administrative changes that both eliminate the cycle specific implementation of ATRIUM-9B fuel and adopt Improved Technical Specification language where appropriate, and
3. changes to the Dresden Minimum Critical Power Ratio (MCPR) safety limits.

2.0 EVALUATION

The requested TS changes can be categorized into four different topics:

1. Addition of SPC Topical for ATRIUM-9B fuel - ANF-1125(P)(A) Supplement 1, Appendix E (Reference 4);
2. change to MCPR safety limit;
3. removal of footnotes limiting operation with ATRIUM-9B Fuel reloads; and
4. revision to the thermal limit descriptions.

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Currently Dresden, Units 2 and 3, are completing the transition from General Electric (GE) to SPC fuel, including the use of associated methodologies. The SPC Topical for ATRIUM-9B fuel (ANF-1125(P)(A), Supplement 1, Appendix E) was recently reviewed and approved by NRC (Reference 4). The NRC approval of this topical listed three conditions to which ComEd has committed (Reference 3). Thus, the SPC Topical for ATRIUM-9B fuel (ANF-1125(P)(A), Supplement 1, Appendix E) is acceptable for use for Dresden, Units 2 and 3. The addition of this methodology will ensure that values for cycle specific parameters are determined such that all applicable limits of the safety analysis are met.

The change to MCPR safety limit was due to the change to SPC fuel. Using the S.C. AFB critical power correlation methodology and the ATRIUM-9B additive constant uncertainty resulting from the approval of Appendix E (Reference 4), the MCPR safety limit for Dresden, Unit 3, will be 1.10. The MCPR safety limit for Dresden, Unit 2, will remain 1.09. The applicability of the MCPR safety limit will be confirmed on a cycle-by-cycle basis. These values are anticipated to bound the actual MCPR safety limit for future Dresden SPC reloads. Since the MCPR safety limits were calculated with an approved methodology, and used the approved additive constant uncertainty from Appendix E, the change to this value will ensure that 99.9 percent of the fuel rods will avoid transition boiling during normal operation and anticipated operational occurrences and is acceptable.

The removal of the footnote limiting operation with ATRIUM-9B fuel to specific reloads is due to the fact the footnote is no longer needed since the methodology for ATRIUM-9B fuel has been approved. Thus, this change is acceptable.

The change to revise the thermal limit descriptions is to generalize the definition of the average planar linear heat generation rate (APLHGR) limit to allow either a bundle average or an average planar exposure based APLHGR limit, consistent with the loss-of-coolant accident (LOCA) analysis of record. Currently the Dresden TSs specify the APLHGR limit as a function of the average planar exposure. However, the results of S.C.'s NRC-approved LOCA methodology may be applied on either a bundle average or average planar exposure. Thus, a less stringent description of APLHGR is proposed by this amendment such that the detailed information to which the APLGHR is monitored is specified in the core operating limits report (COLR). This generalization of the definition of APLHGR is consistent with the Improved Standard Technical Specifications (NUREG 1433/1434, Revision 1) wording. Both maximum average planar linear heat generation rates (MAPLHGR) (bundle average exposure based and planar average exposure based) are acceptable for Appendix K of 10 CFR Part 50. Thus, this change is acceptable.

3.0 TECHNICAL SPECIFICATION CHANGES

Technical Specifications -Table of Contents - Delete item for definition of Average Planar Exposure and delete definition of Average Planar Exposure. This is acceptable because the average planar exposure is no longer used.

Technical Specification 2.1.B - Change the MCPR to 1.10 for Unit 3. This change results from the use of ATRIUM 9B fuel, will ensure that less than 0.1 percent of the fuel rods in the core undergo transition boiling and is, therefore, acceptable.

Technical Specification 3/4.11A and 3/4.11D - The description of the APLHGR limiting condition of operation (LCO) is changed to not specify that the APLHGR should be a function of average planar exposure and the description of the linear heat generation rate (LHGR) is changed to not specify that the LHGR should be a function of average planar exposure.

These changes are acceptable because the APLHGR and LHGR are based on the bundle average exposure consistent with the LOCA analysis.

Technical Specification 5.3 - Removal of the Dresden, Unit 3, Cycle 15, and Dresden, Unit 2, Cycle 16, specific footnote. This change is acceptable because the cycle specific footnote is no longer needed, since the methodology described in the topical has been approved.

Technical Specification 6.9.A.6.b - Addition of the S.C. topical report, ANF-1125, Supplement 1, Appendix E, for the ATRIUM-9B additive constant uncertainties. This change is needed for use of the ATRIUM-9B fuel and addition of this methodology will ensure that values for cycle-specific parameters are determined such that all applicable limits of the safety analysis are met.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (63 FR 59588). The amendments also change recordkeeping or reporting requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c) (10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principle Contributor: M. Chatterton

Date: February 16, 1999

7.0 REFERENCES

1. Letter from R.M. Krich, Commonwealth Edison Company to NRC, dated August 14, 1998.
2. Letter from R.M. Krich, Commonwealth Edison Company to NRC, dated October 13, 1998.
3. Letter from R.M. Krich, Commonwealth Edison Company to NRC, dated December 23, 1998.
4. ANF-1125(P), Supplement 1, Appendix E, "AFB Critical Power Correlation Determination of ATRIUM-9B Additive Constant Uncertainties," and NPC SER, "Acceptance for Referencing of Licensing Topical Report ANF-1125(P), Supplement 1, Appendix E, Critical Power Correlation Determination of ATRIUM-9B Additive Constant Uncertainties," T.H. Essig to H.D. Curet, September 23, 1998.