

JUL 08 1977

Docket No. 50-346

Toledo Edison Company  
ATTN: Mr. Lowell E. Roe  
Vice President, Facilities  
Development  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

Gentlemen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 4 TO FACILITY OPERATING LICENSE  
NO. NPF-3 FOR DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 4 to the Facility Operating License No. NPF-3 which is effective as of the date of issuance. Amendment No. 4 revises the Technical Specification, Appendix A, by changing Table 3.3-11 on Pages 3/4 3-25 and 3/4 3-26 to specify the new level switch numbers. Table 3.3-12 on Page 3/4 3-28 is changed to show that the allowable value for Functional Unit 2 is greater than or equal to ( $\geq$ ) 20 inches H<sub>2</sub>O for the Channel Functional Test and greater than or equal to ( $\geq$ ) 18.87 inches H<sub>2</sub>O for the Channel Calibration. Also, Table 4.3-11 on Page 3/4 3-30 is changed to show a Channel Calibration frequency for Item 1b to be once every 18 months.

This license is amended by making the appropriate changes as listed above to the technical specifications on pages 3/4 3-25, 3/4 3-26, 3/4 3-28, and 3/4 3-30.

We have determined that Amendment No. 4 does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves actions which are insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Encl. 1  
GD

OFFICE >						
SURNAME >						
DATE >						

Toledo Edison Company

- 2 -

A copy of the Federal Register Notice of Issuance of Amendment No. 4 and the related Safety Evaluation supporting Amendment No. 4 to License No. NPF-3 are also enclosed.

Sincerely,

Original Signed by

John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

Enclosures:

1. Amendment No. 4 to License  
No. NPF-3
2. Federal Register Notice
3. Safety Evaluation Supporting  
Amendment No. 4 to License  
No. NPF-3

cc w/enclosures:  
See page 3

*NOTE with change as noted*

*Major STG/OR*  
*7/8/77*

OFFICE	LWR-#1	LWR-#1	LWR-#1	LWR-#1		
SURNAME	EM:con:klj	LE:le	GFess	JStolz		
DATE	6/30/77	6/30/77	6/30/77	7/8/77		

cc: Gerald Charnoff, Esq.  
Shaw, Pittman, Potts & Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

Leslie Henry, Esq.  
Fuller, Seney, Henry and Hodge  
300 Madison Avenue  
Toledo, Ohio 43604

Mr. Donald H. Hauser, Esq.  
The Cleveland Electric Illuminating Company  
P. O. Box 5000  
Cleveland, Ohio 44101

Mr. Harry R. Johnson  
Ottawa County Courthouse  
Port Clinton, Ohio 43452

Ohio Department of Health  
ATTN: Director of Health  
450 East Town Street  
Columbus, Ohio 43216

Atomic Energy Control Board  
P. O. Box 1046  
Ottawa, Ontario, Canada

Harold Kahn, Staff Scientist  
Power Siting Commission  
361 East Broad Street  
Columbus, Ohio 43216

Mr. Bruce Blanchard  
Environmental Projects Review  
Department of the Interior  
Room 5321  
18th and C Street, N. W.  
Washington, D. C. 20240

OFFICE ➤						
SURNAME ➤						
DATE ➤						

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Toledo Edison Company (the licensee) dated June 24, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of facility Operating License No. NPF-3 is hereby amended to read as follows:

OFFICE ➤						
SURNAME ➤						
DATE ➤						

2.C(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 4, are hereby incorporated in the license. Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by

John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: JUL 08 1977

7/8/77

OFFICE	LWR-#1	LWR-#1	OELD	LWR-#1		
SURNAME	Edison:klj	LEngle	GROSS	JStolz		
DATE	6/30/77	6/30/77	6/30/77	7/6/77		

ATTACHMENT TO LICENSE AMENDMENT NO. 4

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-25

3/4 3-26

3/4 3-28

3/4 3-30

3/4 4-1

3/4 4-2

Also enclosed are pages 3/4 4-1 and 3/4 4-2 which were incorrectly numbered as 3/4 4-2 and 3/4 4-3 by Amendment No. 1. The replacement pages submitted with Amendment No. 1 should be destroyed.

OFFICE ➤						
SURNAME ➤						
DATE ➤						

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4  
License No. NPF-3

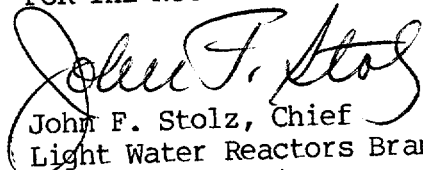
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Toledo Edison Company (the licensee) dated June 24, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of facility Operating License No. NPF-3 is hereby amended to read as follows:

2.C(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 4, are hereby incorporated in the license. Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "John F. Stolz", is written over the typed name and title.

John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: JUL 08 1977



ATTACHMENT TO LICENSE AMENDMENT NO. 4

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-25

3/4 3-26

3/4 3-28

3/4 3-30

3/4 4-1

3/4 4-2

Also enclosed are pages 3/4 4-1 and 3/4 4-2 which were incorrectly numbered as 3/4 4-2 and 3/4 4-3 by Amendment No. 1. The replacement pages submitted with Amendment No. 1 should be destroyed.

TABLE 3.3-11 (Continued)

STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>		<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
2.	Feedwater/Steam Generator Differential Pressure - High Instrument Channels	2	1	2	13#
a.	PDS 2685A Feedwater/Steam Generator 2 Channel 2 PDS 2685B Feedwater/Steam Generator 2 Channel 2				
b.	PDS 2685C Feedwater/Steam Generator 2 Channel 1 PDS 2685D Feedwater/Steam Generator 2 Channel 1				
c.	PDS 2686A Feedwater/Steam Generator 1 Channel 1 PDS 2686B Feedwater/Steam Generator 1 Channel 1				
d.	PDS 2686C Feedwater/Steam Generator 1 Channel 2 PDS 2686D Feedwater/Steam Generator 1 Channel 2				
3.	Steam Generator Level - Low Instrument Channels	2	1	2	13#
a.	LSLL SP9B8 Steam Generator 1 Channel 1 LSLL SP9B9 Steam Generator 1 Channel 1				
b.	LSLL SP9A6 Steam Generator 2 Channel 1 LSLL SP9A7 Steam Generator 2 Channel 1				
c.	LSLL SP9A8 Steam Generator 2 Channel 2 LSLL SP9A9 Steam Generator 2 Channel 2				

DAVIS-BESSE, UNIT 1

3/4 3-25

Amendment No. 4

TABLE 3.3-11 (Continued)

STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
3. Steam Generator Level - Low Instrument Channels (continued)				
d. LSSL SP9B6 Steam Generator 1 Channel 2 LSSL SP9B7 Steam Generator 1 Channel 2				
4. Loss of RCP Channels	2	1	2	13#
5. Manual Initiation (Push buttons)				
a. Steam Pressure - Low	4	2	4	14
b. Level Steam Generator - Low	2	1	2	14
c. Feedwater $\Delta P$ - High	2	1	2	14
d. Loss of RCP's	2	1	2	14

DAVIS-BESSE, UNIT 1

3/4 3-26

Amendment No. 4

TABLE 3.3-11 (Continued)

TABLE NOTATION

- \* May be bypassed when steam pressure is below 650 psig. Bypass shall be automatically removed when the steam pressure exceeds 650 psig.
- # The provisions of Specification 3.0.4 are not applicable.

ACTION STATEMENTS

- ACTION 13 - With the number of OPERABLE Channels one less than the Total Number of Channels operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST provided the inoperable section of the channel is placed in the tripped condition within 1 hour.
- ACTION 14 - With the number of OPERABLE Channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

TABLE 3.3-12  
STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM  
INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNITS</u>	<u>TRIP SETPOINTS</u>	<u>ALLOWABLE VALUES</u>
1. Steam Line Pressure - Low	$\geq 591.6$ psig	$\geq 591.6$ psig* $\geq 586.6$ psig**
2. Steam Generator Level - Low <sup>(1)</sup>	$\geq 20"$ H <sub>2</sub> O	$\geq 20"$ H <sub>2</sub> O* $\geq 18.87$ H <sub>2</sub> O**
3. Steam Generator Feedwater Differential Pressure - High <sup>(2)</sup>	$\leq 197.6$ psid	$\leq 197.6$ psid* $\leq 199.6$ psid**
4. Reactor Coolant Pumps - Loss of	High $\leq 1384.6$ amps Low $\geq 106.5$ amps	$\leq 1384.6$ amps* $\geq 106.5$ amps#

(1) Measured above the lower steam generator tubesheet.

(2) Where differential pressure is steam generator minus feedwater pressure.

\* Allowable Value for CHANNEL FUNCTIONAL TEST

\*\* Allowable Value for CHANNEL CALIBRATION

# Allowable Value for CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION

TABLE 3.3-13

STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM RESPONSE TIMES

<u>ACTUATED EQUIPMENT</u>	<u>RESPONSE TIME IN SECONDS</u>
1. Auxiliary Feed Pump	$\leq 40$
2. Main Steam Isolation Valves	$\leq 6$
3. Main Feedwater Valves	
a. Main Control	$\leq 8$
b. Startup Control	$\leq 13$
c. Stop Valve	$\leq 16$
4. Turbine Stop Valves	$\leq 6$

TABLE 4.3-11

STEAM AND FEEDWATER RUPTURE CONTROL SYSTEM  
INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. Instrument Channel			
a. Steam Line Pressure - Low	S	R	M
b. Steam Generator Level - Low	S	R	M
c. Steam Generator - Feedwater Differential Pressure-High	S	R	M
d. Reactor Coolant Pumps-Loss of	S	R	M
2. Manual Actuation	NA	NA	R

### 3/4.4 REACTOR COOLANT SYSTEM

#### REACTOR COOLANT LOOPS

#### LIMITING CONDITION FOR OPERATION

3.4.1 Both reactor coolant loops and both reactor coolant pumps in each loop shall be in operation.

APPLICABILITY: As noted below, but excluding MODE 6.\*

ACTION:

MODES 1 and 2:

- a. With one reactor coolant pump not in operation, STARTUP and POWER OPERATION may be initiated and may proceed provided THERMAL POWER is restricted to less than 80.7% of RATED THERMAL POWER and within 4 hours the setpoints for the following trips have been reduced to the values specified in Specification 2.2.1 for operation with three reactor coolant pumps operating:
  1. High Flux
  2. Flux- $\Delta$ Flux-Flow
- b. With one reactor coolant pump in each loop not in operation, STARTUP and POWER OPERATION may be initiated and may proceed provided THERMAL POWER is restricted to less than 53.0% of RATED THERMAL POWER and within 4 hours the setpoints for the following trips have been reduced to the values specified in Specification 2.2.1 for operation with one reactor coolant pump operating in each loop:
  1. High Flux
  2. Flux- $\Delta$ Flux-Flow

---

\* See Special Test Exception 3.10.3.



## REACTOR COOLANT SYSTEM

### LIMITING CONDITION FOR OPERATION (Continued)

#### MODES 3, 4 and 5:

- a. Operation may proceed provided at least one reactor coolant loop is in operation with an associated reactor coolant pump or decay heat removal pump.\*
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

\*All reactor coolant pumps and decay heat removal train pumps may be de-energized for up to 1 hour to accommodate decay heat removal pump switching operations, provided no operations are permitted which could cause dilution of the reactor coolant system boron concentration.

### SURVEILLANCE REQUIREMENTS

4.4.1 The Reactor Protective Instrumentation channels specified in the applicable ACTION statement above shall be verified to have had their trip setpoints changed to the values specified in Specification 2.2.1 for the applicable number of reactor coolant pumps operating either:

- a. Within 4 hours after switching to a different pump combination if the switch is made while operating, or
- b. Prior to reactor criticality if the switch is made while shutdown.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-346

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY

OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 4 to the Facility Operating License No. NPF-3, issued to the Toledo Edison Company and the Cleveland Electric Illuminating Company, which revised Technical Specifications for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of its date of issuance.

This license is amended by making the appropriate changes to the technical specifications on pages 3/4 3-25, 3/4 3-26, 3/4 3-28, 3/4 3-30.

The amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

OFFICE ➤						
SURNAME ➤						
DATE ➤						

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) Amendment No. 4 to License No. NPF-3, (2) the Commission's related Safety Evaluation supporting Amendment No. 4 to License No. NPF-3. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Ida Rupp Public Library, 310 Madison Street, Port Clinton, Ohio 43452. A copy of items (1) and (2) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Project Management.

Dated at Bethesda, Maryland, this 8<sup>th</sup> day of July 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by

John Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

OFFICE →	LWR-#1	LWR-#1	DEIR	LWR-#1		
SURNAME →	EH:klj	LEngle	Grass	JStolz		
DATE →	6/30/77	6/30/77	6/30/77	7/9/77		

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 4 TO LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

INTRODUCTION

By letter dated June 24, 1977, the Toledo Edison Company requested changes in the technical specifications appended to Facility Operating License, NPF-3, for the Davis-Besse Nuclear Power Station, Unit 1. The requested changes to the technical specifications are required to reflect the proper allowable values and surveillance requirements for the steam generator level transmitters which have been installed in lieu of the steam generator level switches in the facility steam and feedwater rupture control system.

DISCUSSION

The Toledo Edison Company specified that the following changes are required to properly address the allowable values and surveillance requirements for the newly installed steam generator level transmitters: (1) Table 3.3-11 on Page 3/4 3-25 and Page 3/4 3-26 be changed to show the new level switch numbers, (2) Table 3.3-12 on Page 3/4 3-28 be changed to show that the allowable Value for Functional Unit 2 is greater than or equal to ( $\geq$ ) 20 inches of water and greater than or equal to ( $\geq$ ) 18.87 inches water for the Channel calibration, and (3) Table 4.3-11 on Page 3/4 3-30 be changed to show a Channel Calibration frequency for Item 1b of once every 18 months (R) instead of once every 92 days (Q).

The Toledo Edison Company states that the requested changes to the technical specifications are required prior to facility operation in Mode 3 (hot standby) since the steam generator level switches which have been replaced are not presently functional

OFFICE >						
SURNAME >						
DATE >						

### EVALUATION

Attachment 2, item E.1, to Facility Operating License, NPF-3, stipulates that the four steam generator level switches in each steam generator inside containment be replaced with four steam generator level transmitters prior to proceeding to facility operation in Mode 2 (startup).

The allowable value for channel calibration of greater than or equal to 18.87 inches of water instead of the greater than or equal to 20.0 inches of water is based on an allowance of 1.13 inches for transmitter drift over an 18 month period as determined by qualification testing of the steam generator level transmitters. Based on our review of the modified system, we have concluded that the allowance for transmitter drift is acceptable.

The change in the frequency of channel calibration to once per 18 months instead of once per 92 days is based on recommendations contained in IEEE 279-1971, and is acceptable.

Therefore, based on our evaluation as set forth in this report and on our evaluation and findings as delineated in our Safety Evaluation Report and Supplement No. 1, Section 7.4.1(4), we find the Toledo Edison Company's request for technical specification changes, as discussed above and specifically relating to the recently installed steam generator level transmitters, to be acceptable.

### ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public. Also, we reaffirm our conclusions as otherwise stated in our Safety Evaluation Report.

OFFICE ➤						
SURNAME ➤						
DATE ➤	JUL 08 1977					

DISTRIBUTION FOR DAVIS-BESSE, UNIT NO. 1 AMENDMENT NO. 4 TO OPERATING  
LICENSE NO. NPF-3 DATED **JUL 08 1977**

Docket File 

NRC PDR  
Local PDR  
LWR-#1 File  
GFess, OELD  
RCDeYoung  
DBVassallo  
JFStolz  
LEngle  
FJWilliams  
HSmith  
BScott  
IE (5)  
NDube  
BJones (4)  
WMiller, ADM  
ACRS (16)  
VAMoore, DSE  
HDenton, DSE  
RHVollmer, DSE  
MLErnst, DSE  
WPGammill, DSE  
RHeineman, SS  
JKnight, SS  
DFRoss, SS  
RLTedesco, SS  
BScharf, ADM (10)  
DSkovholt  
AToalston, AIG  
IDinitz, AIG  
EHughes  
PCota, EP  
MSlater, EP  
HBristow, NMSS  
SDuncan, NMSS  
VStello  
KGoller  
JMcGough  
DEisenhut  
WPasciak (Appendix B only)  
EGHylton (5)  
PWagner, OR  
CParrish, OR

BCC: JRBuchanan, NSIC  
TBAbernathy, TIC  
ARosenthal, ASLAB  
JYore, ASLBP