

AUG 18 1978

Docket No. 50-346

Toledo Edison Company  
ATTN: Mr. Lowell E. Roe  
Vice President, Facilities  
Development  
Edison Plaza  
300 Madison Avenue  
Toledo, Ohio 43652

Gentlemen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 12 TO FACILITY OPERATING LICENSE  
NO. NPF-3 FOR DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 12 to Facility Operating License No. NPF-3 for the Davis Besse Nuclear Power Station, Unit 1. The amendment is effective as of the date of issuance.

Amendment No. 12 consists of changes to the Technical Specifications in response to your application dated June 12, 1978.

Amendment No. 12 revises the Administrative Control Section of the Technical Specifications to reflect the changes in the administrative organization of the Toledo Edison Company as specified in your application of June 12, 1978. Also, based on subsequent discussions with you regarding these matters, you agreed to certain changes which we requested, and Amendment No. 12 reflects these changes.

Finally, we have noted an error in the page numbers in the Administrative Control Section of the Technical Specifications which occurred with the issuance of Amendment No. 9. The corrected page numbers are included in Amendment No. 12.

We have determined that Amendment No. 12 does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

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OFFICE➤						
SURNAME➤						
DATE➤						

Toledo Edison Company

- 2 -

Copies of the Federal Register Notice of Issuance of Amendment No. 12 and the Safety Evaluation Supporting Amendment No. 12 to License NPF-3 are also enclosed.

Sincerely,

Original Signed by

John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

Enclosures:

1. Amendment No. 12 to NPF-3
2. FEDERAL REGISTER Notice
3. Safety Evaluation Supporting  
Amendment 12 to NPF-3

cc: See page 3

OFFICE ➤	LWR 1	LWR 1	CHAD	LWR 1		
SURNAME ➤	EH/ten/red	LEngler	GFess	JStolz		
DATE ➤	7/31/78	8/1/78	8/1/78	8/1/78		

cc: Mr. Donald H. Hauser, Esq.  
The Cleveland Electric Illuminating  
Company  
P. O. Box 5000  
Cleveland, Ohio 44101

Mr. Frederick O. Rouse, Chairman (2)  
Great Lakes Basin Commission  
P. O. Box 999  
Ann Arbor, Michigan 49106

Gerald Charnoff, Esq.  
Shaw, Pittman, Potts  
and Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

Leslie Henry, Esq.  
Fuller, Seney, Henry and Hodge  
300 Madison Avenue  
Toledo, Ohio 43604

Attorney General  
Department of Attorney General  
30 East Town Street  
Columbus, Ohio 43215

Ohio Department of Health  
ATTN: Director of Health  
450 East Town Street  
Columbus, Ohio 43216

Harold Kahn, Staff Scientist  
Power Siting Commission  
361 East Broad Street  
Columbus, Ohio 43216

Mr. Harry R. Johnson  
Ottawa County Courthouse  
Port Clinton, Ohio 43452

U. S. Environmental Protection Agency  
Federal Activities Branch  
ATTN: EIS Coordinator  
Region V Office  
230 South Dearborn Street  
Chicago, Illinois 60604

Mr. Jack E. Hemphill  
U. S. Fish & Wildlife Service  
Federal Building  
Fort Snelling  
Twin Cities, Minnesota 55111



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12  
License No. NPF-3

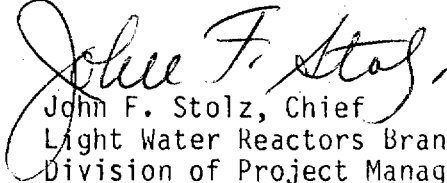
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The issuance of this license amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the amended Facility Operating License No. NPF-3 is hereby amended by changing the Technical Specifications as indicated in the attachment to this license amendment.

2.C.(3) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 12 are hereby incorporated in the license. Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: **AUG 18 1978**

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12  
License No. NPF-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
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  - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
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OFFICE ➤						
SURNAME ➤						
DATE ➤						

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FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by

John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: **AUG 18 1978**

OFFICE ➤	LWR 1	LWR 1	OELD	LWR 1		
SURNAME ➤	EHon/red	LEngle	GFess	JStolz		
DATE ➤	8/1/78	8/1/78	8/1/78	8/1/78		

ATTACHMENT TO LICENSE AMENDMENT NO. 12

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

6-2  
6-3  
6-6  
6-9  
6-11  
6-12  
6-13  
6-14  
6-15  
6-16  
6-17  
6-18  
6-19  
6-20



## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The (Plant Superintendent) shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

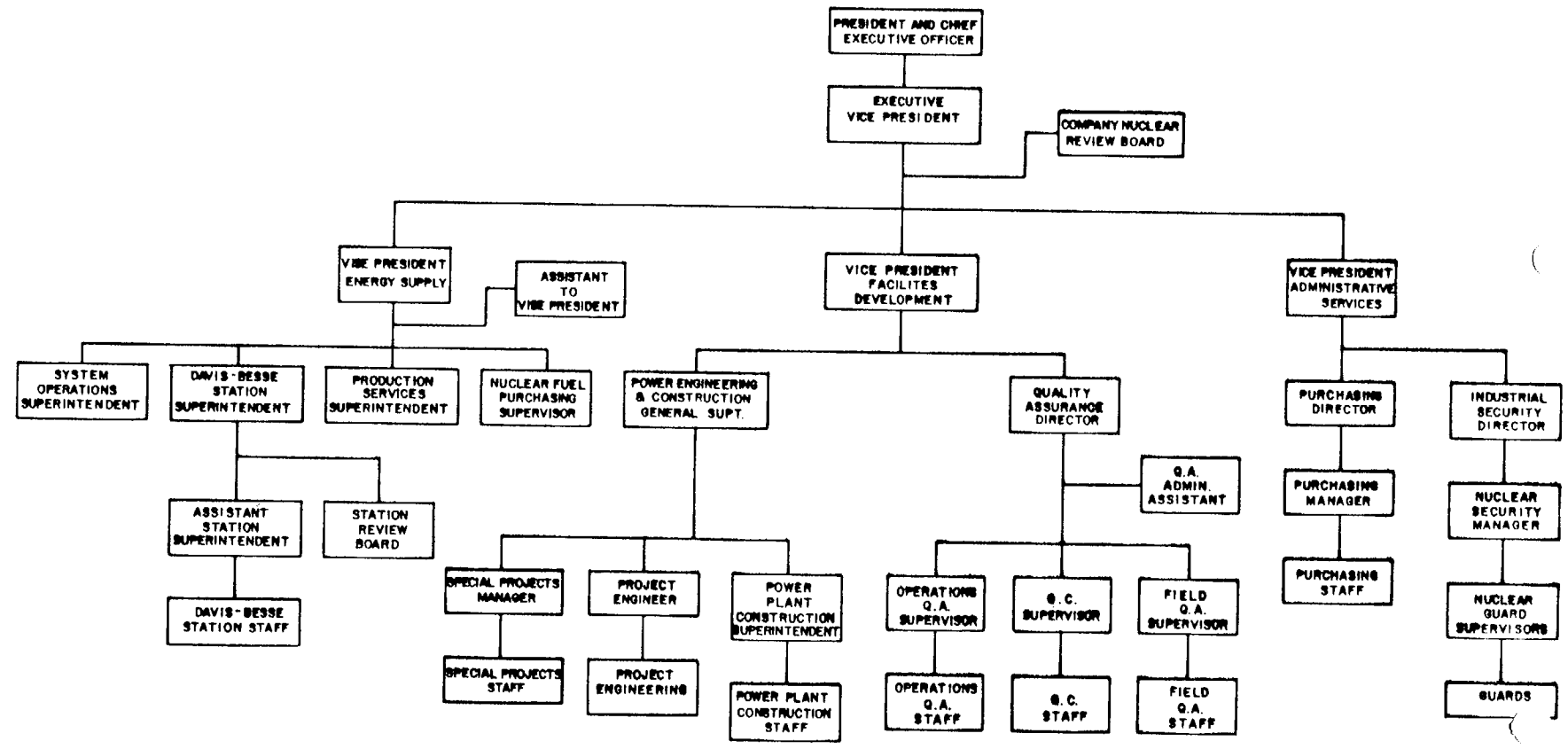
#### OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

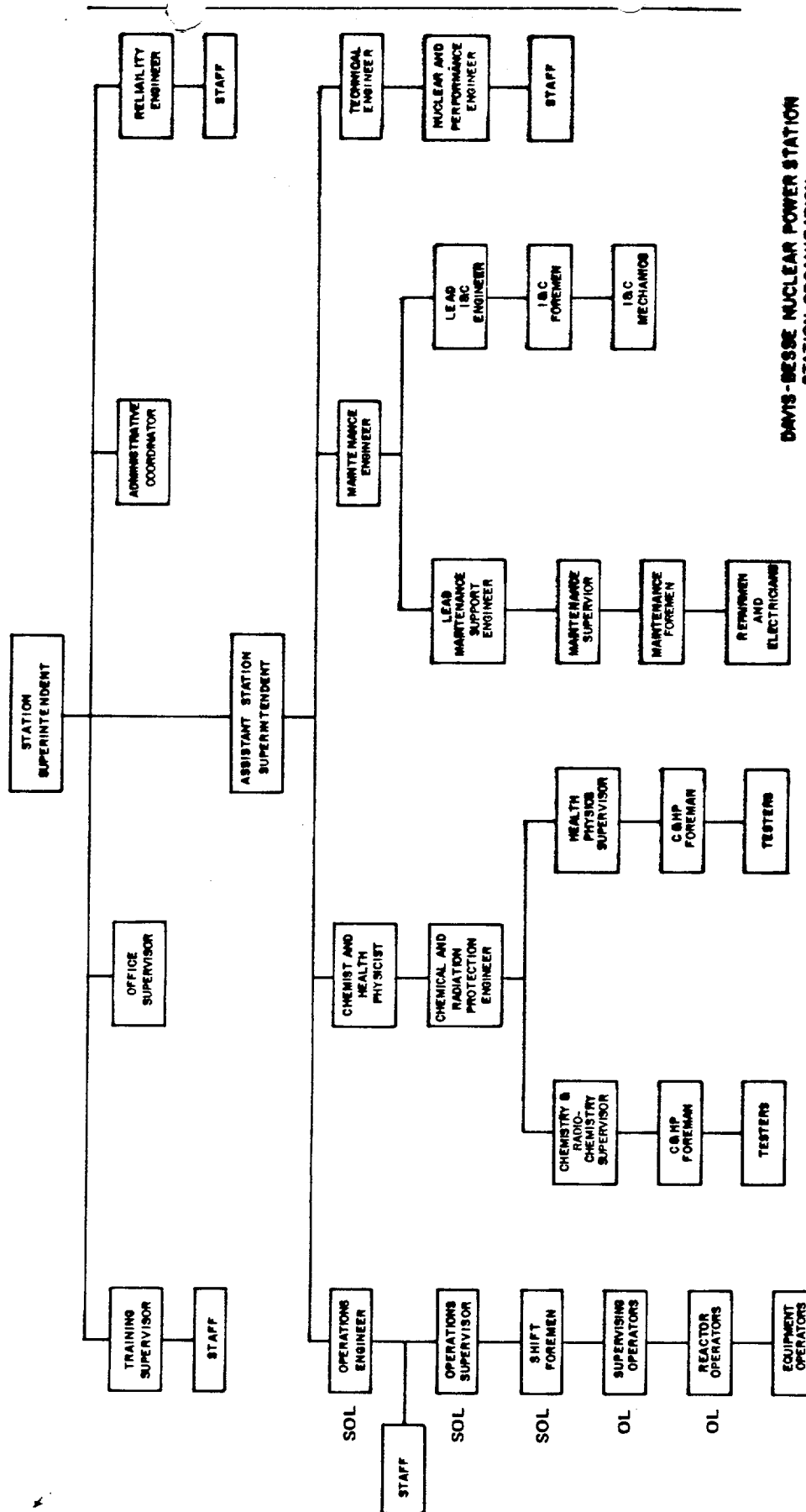
#### FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least 3 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew shown in Table 6.2-1 or any personnel required for other essential functions during a fire emergency.



DAVIS-BESSE NUCLEAR POWER STATION  
UNIT No. 1  
OFF SITE ORGANIZATION CHART  
FIGURE 6.2-1



DAVIS-BESSE NUCLEAR POWER STATION  
STATION ORGANIZATION  
FIGURE 6.2-2

TABLE 6.2-1  
MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	1	1*
OL	2	1
Non-Licensed	2	1

\*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling supervising CORE ALTERATIONS.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

## ADMINISTRATIVE CONTROLS

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### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the (Radiation Protection Manager) who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the (position title) and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Bridage shall be maintained under the direction of the Fire Marshall and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION REVIEW BOARD (SRB)

##### FUNCTION

6.5.1.1 The Station Review Board (SRB) shall function to advise the Station Superintendent on all matters related to nuclear safety.

## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.1.2 The Station Review Board shall be composed of the:

Chairman:	Assistant Station Superintendent	I
Member:	Operations Engineer	
Member:	Technical Engineer	
Member:	Maintenance Engineer	
Member:	Lead Instrument and Control Engineer	I
Member:	Nuclear and Performance Engineer	
Member:	Chemist and Health Physicist	
Member:	Reliability Engineer	
Member:	Station Superintendent	I

### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRB activities at any one time.

### MEETING FREQUENCY

6.5.1.4 The SRB shall meet at least once per calendar month and as convened by the SRB Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SRB shall consist of the Chairman or his designated alternate and four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The Station Review Board shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Superintendent to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.

## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.2.2 The Company Nuclear Review Board shall be composed of the:

Chairman:	Vice President, Facilities Development
Member:	Vice President, Energy Supply
Member:	General Superintendent, Power Engineering and Construction
Member:	General Superintendent, Transmission and Substations
Member:	Superintendent, Davis-Besse Station
Member:	Superintendent, Production Services
Member:	Nuclear Project Engineer, Power Engineering
Member:	Nuclear Engineer, Power Engineering
Member:	Director, Quality Assurance
Member:	General Superintendent, Fossil Operations
Member:	Others as deemed advisable by the CNRB Chairman*

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the CNRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in CNRB activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the CNRB Chairman to provide expert advice to the CNRB.

### MEETING FREQUENCY

6.5.2.5 The CNRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

### QUORUM

6.5.2.6 A quorum of CNRB shall consist of the Chairman or his designated alternate and at least half of the appointed CNRB members or their alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

\*Others as deemed advisable by the CNRB chairman, who are appointed to the Company Nuclear Review Board shall have an academic degree in an Engineering or Physical Science Field; and in addition, shall have a minimum of five years of technical experience, of which a minimum of three years shall be in one or more of the areas specified in Specification 6.5.2.1.

## ADMINISTRATIVE CONTROLS

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### REVIEW

6.5.2.7 The Company Nuclear Review Board shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Station Review Board.



## ADMINISTRATIVE CONTROLS

### AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the CNRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire station staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Station Emergency Plan and implementing procedures at least once per 24 months.
- f. The Station Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the CNRB.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

### AUTHORITY

6.5.2.9 The Company Nuclear Review Board shall report to and advise the Executive Vice President, Operations on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8. |

## ADMINISTRATIVE CONTROLS

### RECORDS

6.5.2.10 Records of Company Nuclear Review Board activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each CNRB meeting shall be prepared, approved and forwarded to the Executive Vice President, Operations and CNRB members within 14 days following each meeting. |
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Executive Vice President, Operations and CNRB members within 14 days following completion of the review. |
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Vice President, Operations and CNRB members and to the management positions responsible for the areas audited within 30 days after completion of the audit. |

### 6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SRB and submitted to the CNRB.

## ADMINISTRATIVE CONTROLS

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the Vice President, Energy Supply and to the CNRB within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SRB. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the CNRB and the Vice President, Energy Supply within 14 days of the violation.

### 6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the SRB and approved by the Station Superintendent prior to implementation and reviewed periodically as set forth in administrative procedures.

## ADMINISTRATIVE CONTROLS

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the SRB and approved by the Station Superintendent within 14 days of implementation.

## 6.9 REPORTING REQUIREMENTS

### ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

### STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial

## ADMINISTRATIVE CONTROLS

power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

### ANNUAL OPERATING REPORT<sup>1/</sup>

6.9.1.4 Annual reports covering the activities of the unit during the previous calendar year shall be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

6.9.1.5 Reports required on an annual basis shall include:

- a. A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,<sup>2/</sup> e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- b. The complete results of steam generation tube inservice inspections (Specification 4.5.5.b).

### MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, to arrive no later than the 15th of each month following the calendar month covered by the report.

<sup>1/</sup> A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

<sup>2/</sup> This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

## ADMINISTRATIVE CONTROLS

### REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

### PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP

6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.
- b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
- c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.
- d. Reactivity anomalies involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation greater than or equal to 1%  $\Delta k/k$ ; a calculated reactivity balance indicating a SHUTDOWN MARGIN less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 0.5%  $\Delta k/k$ ; or occurrence of any unplanned criticality.

## ADMINISTRATIVE CONTROLS

- e. Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- f. Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.
- g. Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- h. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
- i. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

## THIRTY DAY WRITTEN REPORTS

6.9.1.9 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

- b. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.
- c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- d. Abnormal degradation of systems other than those specified in 6.9.1.8.c above designed to contain radioactive material resulting from the fission process.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- b. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- d. Seismic event analysis, Specification 4.3.3.3.2.
- e. Fire Detection Instrumentation, Specification 3.3.3.8.
- f. Fire Suppression Systems, Specifications 3.7.9.1 and 3.7.9.2.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.



## ADMINISTRATIVE CONTROLS

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- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient of operational cycles for those facility components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SRB and the CNRB.

## ADMINISTRATIVE CONTROLS

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is 1000 mrem/hr or less shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit\*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Health Physicist in the Radiation Work Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Foreman on duty and/or the Health Physicist.

\*Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-346

THE TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to the Facility Operating License No. NPF-3, issued to the Toledo Edison Company and the Cleveland Electric Illuminating Company, for operation of the Davis-Besse Nuclear Power Station, Unit No. 1 (the facility) located in Ottawa County, Ohio. The amendment is effective as of the date of its issuance.

The amendment incorporates changes in the Administrative Control Section of the Technical Specifications to reflect changes in the organizational structure at the corporate level of the Toledo Edison Company.

The amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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DATE ➤						

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 12, 1978, (2) Amendment No. 12 to License NPF-3, and (3) the Commission's related Safety Evaluation supporting Amendment No. 12 to License No. NPF-3. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555 and at the Ida Rupp Public Library, 310 Madison Street, Port Clinton, Ohio 43452. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Project Management.

Dated at Bethesda, Maryland, this 18th day of August 1978.

FOR THE NUCLEAR REGULATORY COMMISSION:

Original Signed by

John F. Stolz, Chief  
Light Water Reactors Branch No. 1  
Division of Project Management

OFFICE ➤	LWR 1	LWR 1	OELD	LWR 1		
SURNAME ➤	Eaton/red	Leagle	<del>CSB</del>	JStolz		
DATE ➤	8/1/78	8/1/78	8/1/78	8/1/78		

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 12 TO LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

INTRODUCTION

By letter dated June 12, 1978, the Toledo Edison Company requested changes in the Technical Specifications for the Davis-Besse Nuclear Power Station, Unit 1. Also, we have noted an administrative error which requires a change to the Technical Specifications. These changes are identified below:

1. We have noted an administrative error in the Administrative Control Section of the Technical Specifications.
2. The Toledo Edison Company has requested changes to the Technical Specifications to reflect changes which have been proposed for the administrative organization of the Toledo Edison Company.

DISCUSSION

1. Changes to the Technical Specifications which were made at the time of issuance of Amendment No. 9 incorrectly transcribed page numbers no longer applicable to the revised pages. The corrected page numbers, 6-13 through 6-20, are provided in Amendment No. 12.
2. On June 12, 1978, the Toledo Edison Company informed us that changes were being made at the corporate level entailing a new reporting change for the Vice President, Energy Supply, and for the Company Nuclear Review Board. At the station level they include the establishment of the new positions of Assistant Station Superintendent, and Chemical and Radiations Protection Engineer. The Station Operations Engineer, Chemist and Health Physicist, Maintenance Engineer, and Technical Engineer report to the new position of Assistant Station Superintendent. The requested changes are delineated in Pages 6-6, 6-9, 6-11, and 6-12 and also Figures 6.2-1 and 6.2-2 of the Technical Specifications.

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SURNAME ➤						
DATE ➤						

### EVALUATION

1. The changes to the Technical Specifications for Item (1) as discussed above, serve to correct an administrative error, and do not involve any significant hazards considerations, and therefore, we find these changes to be acceptable.
2. We find that the corporate level changes, as discussed above for Item (2), do not weaken the reporting chain of the Station Superintendent or the Company Nuclear Review Board. Also, we find that these changes for the station organization meet the regulatory position stated in Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)" and are, therefore, acceptable.

### ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.51(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public. Also, we reaffirm our conclusions as otherwise stated in our Safety Evaluation Report.

Date: AUG 18 1978

SEE PREVIOUS YELLOW FOR PREVIOUS CONCURRENCES\*

OFFICE ➤	LWR 1 <i>RF</i>	STS/DOR*	QAB*	LWR 1		
SURNAME ➤	LEngle/red	JWetmore	FAIllenspach	JStol <i>JS</i>		
DATE ➤	8/18/78	8/3/78	8/3/78	8/17/78		

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 12 TO LICENSE NO. NPF-3

TOLEDO EDISON COMPANY

AND

CLEVELAND ELECTRIC ILLUMINATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

INTRODUCTION

By letter dated June 12, 1978, the Toledo Edison Company requested changes in the Technical Specifications for the Davis-Besse Nuclear Power Station, Unit 1. Also, we have noted an administrative error which requires a change to the Technical Specifications. These changes are identified below:

1. We have noted an administrative error in the Administrative Control Section of the Technical Specifications.
2. The Toledo Edison Company has requested changes to the Technical Specifications to reflect changes which have been proposed for the administrative organization of the Toledo Edison Company.

DISCUSSION

1. Changes to the Technical Specifications which were made at the time of issuance of Amendment No. 9 incorrectly transcribed page numbers no longer applicable to the revised pages. The corrected page numbers, 6-13 through 6-20, are provided in Amendment No. 12.
2. On June 12, 1978, the Toledo Edison Company informed us that changes were being made at the corporate level entailing a new reporting change for the Vice President, Energy Supply, and for the Company Nuclear Review Board. At the station level they include the establishment of the new positions of Assistant Station Superintendent, and Chemical and Radiations Protection Engineer. The Station Operations Engineer, Chemist and Health Physicist, Maintenance Engineer, and Technical Engineer report to the new position of Assistant Station Superintendent. The requested change are delineated in Pages 6-6, 6-9, 6-11, and 6-12 and also Figures 6.2-1 and 6.2-2 of the Technical Specifications.

OFFICE >						
SURNAME >						
DATE >						

# EVALUATION

1. The changes to the Technical Specifications for item (1) as discussed above, serve to correct an administrative error, and do not involve any significant hazards considerations, and therefore, we find these changes to be acceptable.
2. We find that the corporate level <sup>chain</sup> changes, as discussed above for Item (2), do not weaken the reporting <sup>change</sup> of the Station Superintendent or the Company Nuclear Review Board. Also, we find that these changes for the station organization meet the regulatory position stated in Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)" and are, therefore, acceptable.

## ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.51(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

## CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public. Also, we reaffirm our conclusions as otherwise stated in our Safety Evaluation Report.

Date:

OFFICE ➤	LWR 1 <i>JOE</i>	STS <i>Red</i>	QAB <i>Red</i>	LWR 1		
SURNAME ➤	LEngle/red	JWetmore	FAllenspach	JStolz		
DATE ➤	8/1 /78	8/3 /78	8/3 /78	8/ /78		





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

August 22, 1978

DISTRIBUTION:  
Docket ✓  
LWR #1 Rdg  
EGHylton  
LEngle

Docket No. 50-346

Docketing and Service Section  
Office of the Secretary of the Commission

SUBJECT: NOTICE OF ISSUANCE OF AMENDMENT NO. 12 TO DAVIS-BESSE OPERATING  
LICENSE NO. NPF-3

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies ( 15 ) of the Notice are enclosed for your use.

- ☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- ☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- ☐ Notice of Availability of Applicant's Environmental Report.
- ☐ Notice of Proposed Issuance of Amendment to Facility Operating License.
- ☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- ☐ Notice of Availability of NRC Draft/Final Environmental Statement.
- ☐ Notice of Limited Work Authorization.
- ☐ Notice of Availability of Safety Evaluation Report.
- ☐ Notice of Issuance of Construction Permit(s).
- ☒ Notice of Issuance of Facility Operating License(s) or Amendment(s) No. 12
- ☐ Other: \_\_\_\_\_

P.S. An extra copy of entire package enclosed for NRC ~~RM~~.PDR.


Enclosure:  
As Stated

Office of Nuclear Reactor Regulation

OFFICE →	LWR #1 Rdg					
SURNAME →	EGHylton					
DATE →	8/22/78					

August 18, 1978

DISTRIBUTION FOR AMENDMENT NO. 12 FOR FACILITY OPERATING LICENSE  
NO. NPF-3 FOR DAVIS BESSE NUCLEAR POWER STATION, UNIT NO. 1

Docket File 

NRC PDR

Local PDR

LWR 1 File

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