

April 10, 1997

Ms. Irene Johnson, Acting Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENT (TAC NOS. M97970 AND M97971)

Dear Ms. Johnson:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 156 to Facility Operating License No. DPR-19 and Amendment No. 157 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3. The amendments are in response to your application dated February 19, 1997, as supplemented April 3, 1997.

The amendments remove the 24/48 Volt direct current (Vdc) batteries and associated chargers and distribution systems from the Unit 3 Technical Specifications (TS), by adding a footnote to the 24/48 Vdc batteries, battery chargers and distribution systems TSs to indicate that the TS is only applicable to Unit 2. All safety-related loads associated with the 24/48 Vdc batteries for Unit 3 will be relocated to other safety-related battery systems which are in the TSs.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

John F. Stang, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-237 and 50-249

Enclosures: 1. Amendment No. 156 to DPR-19
2. Amendment No. 151 to DPR-25
3. Safety Evaluation

cc w/encl: see next page

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I. Johnson
Commonwealth Edison Company

Dresden Nuclear Power Station
Unit Nos. 2 and 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156
License No. DPR-19

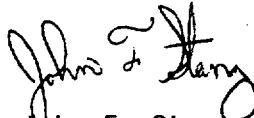
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated February 19, 1997, as supplemented April 3, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 156, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "John F. Stang", is written over the typed name.

John F. Stang, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 10, 1997



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151
License No. DPR-25

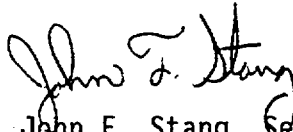
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated February 19, 1997, as supplemented April 3, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 151, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stang, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 10, 1997

ATTACHMENT TO LICENSE AMENDMENT NOS. 156 AND 151

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment numbers and contain marginal lines indicating the area of change.

REMOVE

3/4.9-12

3/4.9-16

3/4.9-18

3/4.9-20

INSERT

3/4.9-12

3/4.9-16

3/4.9-18

3/4.9-20

3.9 - LIMITING CONDITIONS FOR OPERATION**C. D.C. Sources - Operating**

As a minimum, the following D.C. electrical power sources shall be OPERABLE with the identified parameters within the limits specified in Table 4.9.C-1:

1. Two station 250 volt batteries, each with a full capacity charger.
2. Two station 125 volt batteries, each with a full capacity charger.
3. Two unit 24/48 volt batteries, each with a full capacity charger.^(d)

APPLICABILITY:

OPERATIONAL MODE(s) 1, 2, and 3.

ACTION:

1. With one of the above required 24/48 volt or 250 volt station batteries and/or chargers inoperable, restore the inoperable equipment to OPERABLE status within 2 hours^(b).

4.9 - SURVEILLANCE REQUIREMENTS**C. D.C. Sources - Operating**

Each of the required 24/48 volt, 125 volt and 250 volt batteries and chargers shall be demonstrated OPERABLE^(a):

1. At least once per 7 days by verifying that:
 - a. The parameters in Table 4.9.C-1 meet Category A limits, and
 - b. There is correct breaker alignment to the battery chargers and total battery terminal voltage is $\geq 26.0^{(d)}$, ≥ 125.9 , or ≥ 260.4 volts, as applicable, on float charge.
2. At least once per 92 days and within 7 days after a battery discharge with a battery terminal voltage below $21.7^{(d)}$, 105 or 210 volts, as applicable, or battery overcharge with battery terminal voltage above $30^{(d)}$, 150 or 300 volts, as applicable, by verifying that:
 - a. The parameters in Table 4.9.C-1 meet the Category B limits,
 - b. There is no visible corrosion at either terminals or connectors, or the connection resistance of these items is $\leq 150 \times 10^{-6}$ ohms or $\leq 20\%$ above baseline connection resistance, whichever is higher, and

d Applicable to Unit 2 only

- a An alternate 125 volt battery shall adhere to these same Surveillance Requirements to be considered OPERABLE, except the Unit 2 total battery terminal voltage on float charge shall be verified weekly as ≥ 130.2 volts.
- b Each 250 volt battery may be inoperable for a maximum of seven days per operating cycle for maintenance or testing. If it is determined that a 250 volt battery need be replaced as a result of maintenance or testing, a specific battery may be inoperable for an additional seven days per operating cycle.

3.9 - LIMITING CONDITIONS FOR OPERATION**D. D.C. Sources - Shutdown**

As a minimum, the following D.C. electrical power sources shall be OPERABLE:

1. One station 250 volt battery with a full capacity charger.
2. One station 125 volt battery with a full capacity charger.
3. The Unit 2A 24/48 volt battery with a full capacity charger.^(b)

APPLICABILITY:

OPERATIONAL MODE(s) 4 and 5, and when handling irradiated fuel in the secondary containment.

ACTION:

With any of the above required station batteries and/or associated charger(s) inoperable, suspend CORE ALTERATIONS, suspend handling of irradiated fuel in the secondary containment, and suspend operations with a potential for draining the reactor vessel.

4.9 - SURVEILLANCE REQUIREMENTS**D. D.C. Sources - Shutdown**

The required batteries and chargers shall be demonstrated OPERABLE^(a) per the surveillance requirements in Specification 4.9.C.

b Applicable to Unit 2 only

a An alternate 125 volt battery shall adhere to these same Surveillance Requirements to be considered OPERABLE, except the Unit 2 total battery terminal voltage on float charge shall be verified weekly as ≥ 130.2 volts.

3.9 - LIMITING CONDITIONS FOR OPERATION**4.9 - SURVEILLANCE REQUIREMENTS**

4. For Unit 3, 125 volt D.C. power distribution, consisting of:
 - a. TB Main Bus Nos. 2A-1, 3A and 3A-1,
 - b. TB Res. Bus Nos. 3B and 3B-1, and
 - c. RB Distribution Panel No. 3.
5. 24/48 volt D.C. power distribution consisting of Bus Nos. 2A and 2B^(a).

APPLICABILITY:

OPERATIONAL MODE(s) 1, 2, and 3.

ACTIONS:

1. With one of the above required A.C. distribution systems not energized, re-energize the system within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
2. With one of the above required D.C. distribution systems not energized, re-energize the system within 2 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

a Applicable to Unit 2 only

3.9 - LIMITING CONDITIONS FOR OPERATION

4.9 - SURVEILLANCE REQUIREMENTS

4. For 24/48 volt D.C. distribution,
Bus Nos. 2A^(a).

APPLICABILITY:

OPERATIONAL MODE(s) 4, 5, and when
handling irradiated fuel in the secondary
containment.

ACTIONS:

With less than the above required A.C. or
D.C. distribution systems energized,
suspend CORE ALTERATIONS, suspend
handling of irradiated fuel in the secondary
containment, and suspend operations with
a potential for draining the reactor vessel.

a Applicable to Unit 2 only



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 156 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-25

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3

DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated February 19, 1997, as supplemented April 3, 1997, Commonwealth Edison Company (ComEd, the licensee) submitted a license amendment requesting a change to the Technical Specifications (TSs) to remove the 24/48 Volt direct current (Vdc) batteries, battery changers and distribution systems from the TS.

At Dresden Nuclear Power Station, the Unit 3, Division I and II, Analog Trip System (ATS) instrumentation loads and the Unit 2, Division I, ATS loads are currently supplied by the 24/48 Vdc safety-related system, while the Unit 2, Division II, loads are supplied by the 125 Vdc safety-related system. During the D3R14 refueling outage, the direct current (DC) supply to the Unit 3, Division I and II, ATS instrumentation loads will be switched to the safety-related 125 Vdc system. Because the Unit 3, 24/48 Vdc system will not provide a safety-related function, ComEd proposes to remove the TS requirements for the 24/48 Vdc batteries, chargers, and distribution systems to licensee-controlled documents in accordance with the NRC's final policy statement as specified in Section 50.36 of Title 10 of the Code of Federal Regulations (10 CFR 50.36). The April 3, 1997, submittal provided additional clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The proposed amendment would delete the 24/48 volt batteries, chargers, and distribution systems from the Dresden TSs for Unit 3. The proposed change will affect TSs for the DC sources required for operation by TS Section (3.9.C), the DC sources required for shutdown by TS Section (3.9.D), the distribution systems required for operation by TS Section (3.9.E), the distribution systems required during shutdown by TS Section (3.9.F), and the associated surveillance requirements for the respective operating conditions.

At Dresden, the Unit 3, Division I and II, ATS instrumentations loads and the Unit 2, Division I, ATS loads are currently supplied by the 24/48 Vdc system. During the upcoming D3R14 outage, the DC supply to the Unit 3, Division I

and II, ATS instrumentation loads will be switched to the safety-related 125 Vdc system. Removal of the ATS loads from the Unit 3, 24/48 Vdc system removes the only loads required for safe shutdown, and the batteries, chargers, and distribution systems will no longer be required to assure safe operation of Unit 3. The remaining loads on the Unit 3, 24/48 Vdc system do not provide a safety-related function.

The Unit 2, Division I, ATS instrumentation are supplied by the 24/48 Vdc batteries; the Division II, ATS instrumentation loads are supplied by the safety-related 125 Vdc batteries. The Unit 2, Division I, ATS instrumentation loads will remain supplied by the 24/48 Vdc system until the next Unit 2 refueling outage, at which time a license amendment application may be submitted to remove this TS requirement for Unit 2. Therefore, because the TSs for Unit 2 and Unit 3 are combined, ComEd will leave the reference to the 24/48 Vdc batteries, chargers, and distribution systems in the Dresden TSs until all the safety-related loads are removed from the Unit 2, 24/48 Vdc system. However, there will be a note added to the TSs that will specify that the reference to the 24/48 Vdc battery system is applicable only to Unit 2.

3.0 SUMMARY

The removal of the 24/48 Vdc system from the TSs for Unit 3 is found to be acceptable because the system no longer provides a safety-related function. Additionally, the staff finds the note added to the Dresden TSs specifying applicability of the 24/48 Vdc system to Unit 2 acceptable. The proposed note is appropriate because the 24/48 Vdc system for Unit 2 still provides a safety-related function and would be required to remain in the Dresden TSs in accordance with 10 CFR 50.36. Based on the above, the staff finds the proposed TS changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 10088). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR

51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: M. Pratt
S. Saba

Date: April 10, 1997