

Ms. Irene Johnson, Acting Manager
 Nuclear Regulatory Services
 Commonwealth Edison Company
 Executive Towers West III
 1400 Opus Place, Suite 500
 Downers Grove, IL 60515

September 10, 1997

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M98245 AND M98246)

Dear Ms. Johnson:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 162 to Facility Operating License No. DPR-19 and Amendment No. 157 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3. The amendments are in response to your application dated January 24, 1997.

The amendments revise the Technical Specification (TS) surveillance calibration of the reactor water level instrumentation used to initiate the emergency core cooling system (ECCS). The licensee made modifications to the Unit 3 instrumentation during a recent refueling outage (May 1997). The amendments correct the TS to reflect the modifications. The modifications were made to improve the reliability of ECCS initiation on low reactor water level. The modifications are similar to the modifications made to the Unit 2 reactor water level instrumentation. The surveillance requirement for calibration of the new reactor water level instrumentation is consistent with the ECCS low reactor water level initiation transmitter calibration requirements of NUREG 1433, "Standard Technical Specifications, General Electric Plants, BWR/4." The same TS change for Unit 2 has been previously reviewed and approved by the NRC staff in Amendment No. 145 dated June 28, 1996.

In addition minor editorial changes were made to the TS.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

John F. Stang, Senior Project Manager
 Project Directorate III-2
 Division of Reactor Projects - III/IV
 Office of Nuclear Reactor Regulation

Docket Nos. 50-237 and 50-249

Enclosures: 1. Amendment No. 162 to DPR-19
 2. Amendment No. 157 to DPR-25
 3. Safety Evaluation

cc w/encl: see next page

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DATE	08/27/97		08/27/97		09/10/97		08/27/97	

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NAME	JSTANG		CMOORE	<i>cy jolt</i>	RCAPRA	<i>RK</i>	<i>R Bachmann</i>	JWERMIEL*
DATE	09/01/97		08/27/97		09/10/97		09/11/97	08/27/97

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 10, 1997

Ms. Irene Johnson, Acting Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

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A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink that reads "John F. Stang".

John F. Stang, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-237 and 50-249

Enclosures: 1. Amendment No. 162 to DPR-19
2. Amendment No. 157 to DPR-25
3. Safety Evaluation

cc w/encl: see next page

I. Johnson
Commonwealth Edison Company

Dresden Nuclear Power Station
Unit Nos. 2 and 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 162
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated January 24, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

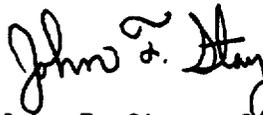
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 162, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stang, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 10, 1997



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. DPR-25

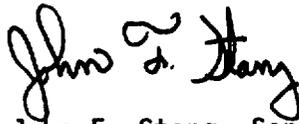
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated January 24, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.157 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stang, Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 10, 1997

ATTACHMENT TO LICENSE AMENDMENT NOS. 162 AND 157
FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR -25
DOCKET NOS. 50-237 AND 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4.2-18

3/4.2-19

3/4.2-20

INSERT

3/4.2-18

3/4.2-19

3/4.2-20

TABLE 4.2.B-1

ECCS ACTUATION INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

INSTRUMENTATION

ECCS Actuation 3/4.2.B

DRESDEN - UNITS 2 & 3

3/4.2-18

Amendment Nos.162 and 157

<u>Functional Unit</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>Applicable OPERATIONAL MODE(s)</u>
<u>1. CORE SPRAY (CS) SYSTEM</u>				
a. Reactor Vessel Water Level - Low Low	S	M	E ^(f)	1, 2, 3, 4 ^(b) , 5 ^(b)
b. Drywell Pressure - High ^(d)	NA	M	Q	1, 2, 3
c. Reactor Vessel Pressure - Low (Permissive)	NA	M	Q	1, 2, 3, 4 ^(b) , 5 ^(b)
d. CS Pump Discharge Flow - Low (Bypass)	NA	Q	E ^(e)	1, 2, 3, 4 ^(b) , 5 ^(b)
<u>2. LOW PRESSURE COOLANT INJECTION (LPCI) SUBSYSTEM</u>				
a. Reactor Vessel Water Level - Low Low	S	M	E ^(f)	1, 2, 3, 4 ^(b) , 5 ^(b)
b. Drywell Pressure - High ^(d)	NA	M	Q	1, 2, 3
c. Reactor Vessel Pressure - Low (Permissive)	NA	M	Q	1, 2, 3, 4 ^(b) , 5 ^(b)
d. LPCI Pump Discharge Flow - Low (Bypass)	NA	Q	E ^(e)	1, 2, 3, 4 ^(b) , 5 ^(b)
<u>3. HIGH PRESSURE COOLANT INJECTION (HPCI) SYSTEM^(a)</u>				
a. Reactor Vessel Water Level - Low Low	S	M	E ^(f)	1, 2, 3
b. Drywell Pressure - High ^(d)	NA	M	Q	1, 2, 3
c. Condensate Storage Tank Level - Low	NA	M	NA	1, 2, 3
d. Suppression Chamber Water Level - High	NA	M	NA	1, 2, 3
e. Reactor Vessel Water Level - High (Trip)	NA	M	E ^(f)	1, 2, 3
f. HPCI Pump Discharge Flow - Low (Bypass)	NA	Q	Q	1, 2, 3
g. Manual Initiation	NA	E	NA	1, 2, 3

TABLE 4.2.B-1 (Continued)

ECCS ACTUATION INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

INSTRUMENTATION

ECCS Actuation 3/4.2.B

<u>Functional Unit</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>Applicable OPERATIONAL MODE(s)</u>
4. <u>AUTOMATIC DEPRESSURIZATION SYSTEM</u>^(a)				
a. Reactor Vessel Water Level - Low Low	S	M	E ^(f)	1, 2, 3
b. Drywell Pressure - High ^(d)	NA	M	Q	1, 2, 3
c. Initiation Timer	NA	E	E	1, 2, 3
d. Low Low Level Timer	NA	E	E	1, 2, 3
e. CS Pump Discharge Pressure - High (Permissive)	NA	M	Q	1, 2, 3
f. LPCI Pump Discharge Pressure - High (Permissive)	NA	M	Q	1, 2, 3
5. <u>LOSS OF POWER</u>				
a. 4.16 kv Emergency Bus Undervoltage (Loss of Voltage)	NA	E	E	1, 2, 3, 4 ^(c) , 5 ^(c)
b. 4.16 kv Emergency Bus Undervoltage (Degraded Voltage)	NA	E	E	1, 2, 3, 4 ^(c) , 5 ^(c)

DRESDEN - UNITS 2 & 3

3/4.2-19

Amendment Nos. 162 and 157

TABLE 4.2.B-1 (Continued)

ECCS ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATION

- (a) Not required to be OPERABLE when reactor steam dome pressure is ≤ 150 psig.
- (b) When the system is required to be OPERABLE per Specification 3.5.B.
- (c) Required when the associated diesel generator is required to be OPERABLE per Specification 3.9.B.
- (d) This function is not required to be OPERABLE when PRIMARY CONTAINMENT INTEGRITY is not required.
- (e) Trip units are calibrated at least once per 92 days and transmitters are calibrated at the frequency identified in the table.
- (f) Trip units are calibrated at least once per 92 days and transmitters are calibrated at the frequency identified in the table.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 162 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 157 TO FACILITY OPERATING LICENSE NO. DPR-25
COMMONWEALTH EDISON COMPANY
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3
DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated January 24, 1997, Commonwealth Edison Company (ComEd, the licensee), submitted a license amendment to change the Technical Specifications (TS) frequency of the calibration to be performed on the reactor water level instrumentation. The TS changes have been requested because of the modifications made to the instrumentation during the recent Unit 3 refueling outage. The modifications were performed primarily to improve the reliability of emergency core cooling system (ECCS) initiation on low low reactor water level. The proposed new TS frequency for calibration of the new level instrumentation is consistent with the ECCS low reactor water level initiation transmitter calibration requirements of NUREG 1433, "Standard Technical Specifications, General Electric Plants, BWR/4" for similar instrumentation. The same TS change for Unit 2 has been previously reviewed and approved by the NRC staff in Amendment No. 145 dated June 28, 1996.

In addition minor editorial changes to the TS are being proposed.

2.0 EVALUATION

During the recent Dresden Unit 3 refueling outage the licensee modified portions of the ECCS low low water level initiation instrumentation. The modification was made to improve the reliability of ECCS initiation on low low reactor water level. The existing Yarway level switches which provided the reactor low low water level indication had become prone to instrument drift. The licensee had found, following several surveillances, that the Yarway level switches calibration had drifted into the nonconservative range. Therefore the licensee chose to replace the existing Yarway level switches with new trip units and trip relays and use the existing Rosemount level transmitter that provides the anticipated transient without scram (ATWS) initiation to also transmit the ECCS low low reactor water level initiation signal.

During the recent Unit 3 refueling outage (May 1997) the four Yarway level switches that provided the initiation signal to the ECCS system on reactor vessel water level low low were removed from the plant. New trip units and trip relays (one for each Yarway contact pair) were installed and coupled to

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the existing Rosemount level transmitters that already provided the ATWS trip of the reactor recirculation pumps on reactor water level low low.

The modification was evaluated by the licensee under the provisions of 10 CFR 50.59 and was found not to result in an unreviewed safety question. In addition, the use of the ATWS signal Rosemount level transmitters does not contradict the diversity requirements of the ATWS rule since the reactor protection system (RPS) scram on low reactor water level has not been affected by this modification. The RPS low reactor vessel water level scram is provided by a different set of sensors and transmitters.

The current frequency for performing the calibration of the ECCS Yarway level switches is shown in TS Table 4.2.B-1 as "Q" for quarterly. This was an appropriate frequency for the Yarway level switches and assured that they would trip within an acceptable range. The proposed TS would also require calibration of the new trip units to be performed on a quarterly basis.

The proposed TS would require calibration of the ECCS transmitter associated with the new reactor vessel low low water level trip units and trip relays to be performed every 18 months. The 18-month frequency for calibration of Rosemount transmitters is consistent with the current requirements for the ATWS low low reactor water level trip (Table 4.2.C-1, item 1) and the RPS low reactor vessel water level scram (Table 4.1.A-1, item 4).

The proposed TS change is consistent with the calibration requirements for the ECCS low low reactor water level initiation transmitter calibration requirements of NUREG 1433. In addition the staff has reviewed and approved the same TS change for Dresden, Unit 2, as part of Amendment 145 dated June 28, 1996.

Based on the above the staff finds the proposed change to the TS concerning the change in the calibration frequency of the instrumentation associated with the ECCS low low water level initiation is acceptable.

The proposed amendment would make changes to TS Table 4.2.B-1. The changes are editorial in nature and do not change any requirements of the current TS. The proposed TS changes would make the nomenclature in Table 4.2.B-1 consistent with other places in the TS where the distinction between calibration frequencies for trip units and transmitters is necessary. For example, for the reactor level reactor scram function listed in Table 4.1.A-1, Item 4, the calibration frequency for the transmitter is shown in the table and the trip unit calibration frequency is shown in the associated footnote. For the ECCS level switches as they are currently listed in the TS, the nomenclature is reversed, i.e., the calibration frequency of the trip unit is listed in the table. The proposed TS would revise the nomenclature for the ECCS low low reactor water level trip unit and transmitter to have the calibration frequency for the transmitter shown in TS Table 4.2.B-1, and the trip unit calibration frequency shown in the associated footnote. The proposed change would make the TS consistent with regards to transmitter and trip unit calibration frequencies and where each is listed throughout the TS. The staff finds the proposed changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 19143). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John Stang

Date: **September 10, 1997**