

October 29, 1992

Docket Nos. 50-237
and 50-249

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

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Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M81293 AND M81294)

The Commission has issued the enclosed Amendment No. 120 to Facility Operating License No. DPR-19 for Dresden, Unit 2, and Amendment No. 116 to Facility Operating License No. DPR-25 for Dresden, Unit 3. The amendments are in response to your application dated August 9, 1991.

The primary purpose of the proposed amendment was to restructure Section 3/4.9 (Auxiliary Electrical Systems) as part of the Dresden TS Improvement Program. However, by letter of June 29, 1992, Section 3/4.9 was superseded in its entirety. The August 9, 1991, submittal also requested miscellaneous administrative changes to Technical Specification Sections 3/4.10.F (Spent Fuel Cask Handling), 3/4.11 (High Energy Piping Integrity), and 6.0 (Administrative Controls). The Administrative Controls change was also included in the Administrative Controls section revisions proposed in your May 7, 1992, submittal that was approved by the staff in Amendment Nos. 116 and 113 for Dresden, Units 2 and 3, respectively, dated July 24, 1992.

This enclosed Safety Evaluation has been limited to the proposed changes identified. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:
Byron L. Siegel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 120 to DPR-19
2. Amendment No. 116 to DPR-25
3. Safety Evaluation

cc w/enclosures:
see next page

*See previous concurrence

OFC NAME	LA: PDIII-2 CMOORE	PM: PDIII-2 BSIEGEL:sw	D: PDIII-2 RBARRETT	*OGC APH
DATE	10/15/92	10/17/92	10/27/92	10/20/92

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PDR

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 29, 1992

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and 50-249

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This enclosed Safety Evaluation has been limited to the proposed changes identified. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Byron L. Siegel".

Byron L. Siegel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 120 to DPR-19
2. Amendment No. 116 to DPR-25
3. Safety Evaluation

cc w/enclosures:
see next page

Mr. Thomas J. Kovach
Commonwealth Edison Company

Dresden Nuclear Power Station
Unit Nos. 2 and 3

cc:

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100 W. Randolph
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Chicago, Illinois 60601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated August 9, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 29, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 120

FACILITY OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

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3.10 LIMITING CONDITIONS FOR OPERATION
(Cont'd.)

2. Fuel cask handling in other than the RESTRICTED MODE will be permitted in emergency or equipment failure situations only to the extent necessary to get the cask to the closest acceptable stable location.

4.10 SURVEILLANCE REQUIREMENTS
(Cont'd.)

- a. Twelve (12) randomly distributed broken wires in one lay or four (4) broken wires in one strand of one rope lay.
 - b. Wear of one-third the original diameter of outside individual wire.
 - c. Kinking, crushing, or any other damage resulting in distortion of the rope.
 - d. Evidence of any type of heat damage.
 - e. Reductions from nominal diameter of more than 1/16 inch for a rope diameter from 7/8" to 1 1/4" inclusive.
2. Prior to operation in the RESTRICTED MODE
 - a. the controlled area limit switches will be tested;
 - b. the "two-block" limit switches will be tested;

3.10 LIMITING CONDITIONS FOR OPERATION
(Cont'd.)

4.10 SURVEILLANCE REQUIREMENTS
(Cont'd.)

3. Operation with a failed controlled area limit switch is permissible for 48 hours providing an operator is on the refueling floor to assure the crane is operated within the restricted zone painted on the floor.

c. the "inching hoist" controls will be tested.

3. The empty spent fuel cask will be lifted free of all support by a maximum of 1 foot and left hanging for 5 minutes prior to any series of fuel cask handling operations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated August 9, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 116 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 29, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 116

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

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3.10 LIMITING CONDITIONS FOR OPERATION
(Cont'd.)

2. Fuel cask handling in other than the RESTRICTED MODE will be permitted in emergency or equipment failure situations only to the extent necessary to get the cask to the closest acceptable stable location.

4.10 SURVEILLANCE REQUIREMENTS
(Cont'd.)

- a. Twelve (12) randomly distributed broken wires in one lay or four(4) broken wires in one strand of one rope lay.
 - b. Wear of one-third the original diameter of outside individual wire.
 - c. Kinking, crushing, or any other damage resulting in distortion of the rope.
 - d. Evidence of any type of heat damage.
 - e. Reductions from nominal diameter of more than 1/16 inch for a rope diameter from 7/8" to 1 1/4" inclusive.
2. Prior to operation in the RESTRICTED MODE
 - a. the controlled area limit switches will be tested:
 - b. the "two-block" limit switches will be tested;

3.10 LIMITING CONDITIONS FOR OPERATION
(Cont'd.)

3. Operation with a failed controlled area limit switch is permissible for 48 hours providing an operator is on the refueling floor to assure the crane is operated within the restricted zone painted on the floor.

4.10 SURVEILLANCE REQUIREMENTS
(Cont'd.)

- c. the "inching hoist" controls will be tested.
3. The empty spent fuel cask will be lifted free of all support by a maximum of 1 foot and left hanging for 5 minutes prior to any series of fuel cask handling operations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. DPR-25

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3

DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated August 9, 1991, Commonwealth Edison Company (CECo, the licensee) proposed to restructure Section 3/4.9 (Auxiliary Electrical Systems) as part of the Dresden TS Improvement Program. However, by letter of June 29, 1992, Section 3/4.9 was superseded in its entirety. The August 9, 1991, submittal also requested miscellaneous administrative changes to Technical Specification (TS) Section 3/4.10.F (Spent Fuel Cask Handling), 3/4.11 (High Energy Piping Integrity), and 6.0 (Administrative Controls) for the Dresden Nuclear Power Station, Units 2 and 3. The Administrative Controls change was also included in the Administrative Controls section revisions proposed in CECo's May 7, 1992, submittal that was approved by the staff in Amendment Nos. 116 and 113 for Dresden, Units 2 and 3, respectively, dated July 24, 1992. This Safety Evaluation has been limited to the proposed changes identified.

2.0 DISCUSSION AND EVALUATION

Technical Specification Section 3/4.10.F provides the operating provisions and surveillance requirements which govern spent fuel cask handling. Currently, LCO statement 3.10.F.3 contains interim operating restrictions for fuel cask handling which were applicable prior to August 30, 1976. Similarly, Surveillance Requirement 4.10.F.2 contains interim testing requirements for the reactor building overhead crane handling system which were applicable prior to August 30, 1976. These interim operating restrictions and surveillance requirements were established because the overhead crane handling system had not yet been equipped with a redundant mechanical limit switch in the main hoist power circuit, an electrical interlock system to prevent crane travel in restricted areas, and a slow speed drive motor. Commonwealth Edison Company stated in their submittal that the overhead crane handling system has subsequently been equipped with these features and, therefore, proposed to remove the interim operating restrictions (in LCO statement 3.10.F.3) and surveillance requirements (in Surveillance Requirement 4.10.F.2) which were applicable prior to August 30, 1976. Based on the fact that the overhead crane has since been modified to include the missing features that led to the incorporation of these operating restrictions and surveillance requirements

into the TS, the staff has determined that CECO's amendment request to remove them is acceptable.

Technical Specification Section 3/4.11 (High Energy Piping Integrity) contains requirements to assure the integrity of specific sections of piping in high energy systems. These sections of high energy piping had the potential, in the event of failure, to inhibit safe plant shutdown. As indicated in LCO statement 3.11.2, the requirements of Section 3/4.11 were applicable on an interim basis until modifications that would minimize the consequences from a line break in those specific sections of high energy piping could be completed. Commonwealth Edison Company stated in its submittal that these modifications were completed for Units 2 and 3 in 1978 and 1977, respectively. Since these modifications have been implemented, TS Section 3/4.11 is no longer applicable; therefore, CECO's amendment request to remove TS Section 3/4.11 and its associated bases section is acceptable to the staff.

In accordance with the guidance provided by Generic Letters 86-10 and 88-12, Amendment Nos. 106 (DPR-19) and 101 (DRP-25) removed the Fire Protection TS (3/4.12) for Units 2 and 3, respectively. However, the amendments inadvertently failed to remove the listing of Surveillance Requirements 4.12.A through 4.12.H from the Table of Contents (page vi). In its submittal, CECO proposed removal of the listing of these surveillance requirements from the Table of Contents. Since the proposed change is only administrative in nature, the staff has determined CECO's request is acceptable.

Commonwealth Edison Company's submittal also proposed to revise TS Section 6.0 (Administrative Controls) to incorporate a title change within the CECO corporate organization structure. Since this change was also included in the administrative controls section revision proposed in CECO's May 7, 1992, submittal that was reviewed and approved by the staff in Amendment Nos. 116 and 113 for Dresden, Units 2 and 3, respectively, dated July 24, 1992, no further action is required.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards

consideration, and there has been no public comment on such finding (57 FR 18171). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). This amendment also relates to a change in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Siegel

Date: October 29, 1992