

September 11, 1992

Docket Nos. 50-237
and 50-249

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

DISTRIBUTION:

Docket File	BBoger
PDIII-2 r/f	JZwolinski
RBarrett	BSiegel
CMoore	OGC
DHagan	BClayton, RIII
GHill(8)	WJones
CGrimes	ACRS(10)
OPA	OC/LFMB
PDIII-2 Gray	NRC & Local PDRs

Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M83297 AND M83298)

The Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. DPR-19 for Dresden Unit 2 and Amendment No. 114 to Facility Operating License No. DPR-25 for Dresden Unit 3. The amendments are in response to your application dated April 24, 1992.

The amendments change the Technical Specifications to add two new sections under Administrative Controls; Section 6.11 - Radiation Protection Program and Section 6.12 - High Radiation Area. Some of the administrative changes associated with this application for amendment request were also requested in the procedures review application for amendment dated May 17, 1992.

The procedures review amendments (116 for Dresden 2 and 113 for Dresden 3), issued on July 24, 1992, incorporated the administrative changes requested in the May 17, 1992, application and thus these changes are not included in the enclosed amendments.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Byron L. Siegel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 118 to DPR-19
2. Amendment No. 114 to DPR-25
3. Safety Evaluation

cc w/enclosures:
See next page

* Please see previous concurrence

OFC	LA PDIII-2	PM: PDIII-2	D: PDIII-2	OGC	
NAME	CMOORE	BSIEGEL	RBARRETT	SHom*	
DATE	9/11/92	9/11/92	9/11/92	9/1/92	

9209170197 920911
PDR ADOCK 05000237
P PDR

NRC FILE CENTER COPY

DF01

Mr. Thomas J. Kovach
Commonwealth Edison Company

Dresden Nuclear Power Station
Unit Nos. 2 and 3

cc:

Michael I. Miller, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60690

Mr. C. Schroeder
Plant Manager
Dresden Nuclear Power Station
6500 North Dresden Road
Morris, Illinois 60450-9765

U. S. Nuclear Regulatory Commission
Resident Inspectors Office
Dresden Station
6500 North Dresden Road
Morris, Illinois 60450-9766

Chairman
Board of Supervisors of
Grundy County
Grundy County Courthouse
Morris, Illinois 60450

Regional Administrator
Nuclear Regulatory Commission, Region III
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137

Illinois Department of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Robert Neumann
Office of Public Counsel
State of Illinois Center
100 W. Randolph
Suite 11-300
Chicago, Illinois 60601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated April 24, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 11, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

INSERT

vi

vi

6-26

6-26

-

6-27

(Table of Contents, Cont'd.)

	<u>Page</u>
4.9 Auxiliary Electrical Systems	3/4.9 - 1
4.9.A Station Batteries	3/4.9 - 1
4.9.B (N/A)	
4.9.C Diesel Fuel	3/4.9 - 5
4.9.D Diesel Generator Operability	3/4.9 - 5
4.10 Refueling	3/4.10- 1
4.10.A Refueling Interlocks	3/4.10- 1
4.10.B Core Monitoring	3/4.10- 1
4.10.C Fuel Storage Pool Water Level	3/4.10- 2
4.10.D Control Rod Drive and Control Rod Drive Maintenance	3/4.10- 3
4.10.E Extended Core Maintenance	3/4.10- 4
4.10.F Spent Fuel Cask Handling	3/4.10- 5
4.10.G Fuel Storage Reactivity Limit	3/4.10- 8
4.11 High Energy Piping Integrity	3/4.11- 1
4.12 Fire Protection Systems - Sections 4.12.A through 4.12.H - Deleted per Generic Letters 86-10 and 88-12 (Amendment 106)	
5.0 Design Features	5 - 1
5.1 Site	5 - 1
5.2 Reactor	5 - 1
5.3 Reactor Vessel	5 - 1
5.4 Containment	5 - 1
5.5 Fuel Storage	5 - 1
5.6 Seismic Design	5 - 2
6.0 Administrative Controls	6 - 1
6.1 Organization, Review, Investigation and Audit	6 - 1
6.2 Procedures and Programs	6 - 13
6.3 Action to be Taken in the Event of a REPORTABLE EVENT in Plant Operation	6 - 15
6.4 Action to be taken in the Event a Safety Limit is Exceeded	6 - 15
6.5 Plant Operating Records	6 - 15
6.6 Reporting Requirements	6 - 17
6.7 Environmental Qualification	6 - 22
6.8 Offsite Dose Calculation Manual (ODCM)	6 - 24
6.9 Process Control Program (PCP)	6 - 25
6.10 Major Changes to Radioactive Waste Treatment Systems (Liquid, Gaseous, Solid)	6 - 25
6.11 Radiation Protection Program	6 - 26
6.12 High Radiation Area	6 - 26

6.0 ADMINISTRATIVE CONTROLS (Cont'd.)

- d. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments;
 - e. A comparison of the predicted releases of radioactive materials in liquid and gaseous effluents and in solid waste to the actual releases for the period in which the changes were made;
 - f. An estimate of the exposure to plant operating personnel as a result of the change; and
 - g. Documentation of the fact that the change was reviewed and found acceptable by the On-site Review Function.
2. The change shall become effective upon review and acceptance by the On-site Review Function.

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

* Health Physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirements during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

6.0 ADMINISTRATIVE CONTROLS (Cont'd.)

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.
- c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by Health Physics in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Radiation Protection Supervisor. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area.

For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem** that are located within large areas, such as the containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted, and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote (such as use of closed circuit TV cameras), may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

** Measurement made at 30 cm (11.8 in) from the source of radioactivity.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated April 24, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 11, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 114

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

vi

6-25

-

INSERT

vi

6-25

6-26

(Table of Contents, Cont'd.)

	<u>Page</u>
4.9 Auxiliary Electrical Systems	3/4.9-1
4.9.A Station Batteries	3/4.9-1
4.9.B (N/A)	
4.9.C Diesel Fuel	3/4.9-5
4.9.D Diesel Generator Operability	3/4.9-5
4.10. Refueling	3/4.10-1
4.10.A Refueling Interlocks	3/4.10-1
4.10.B Core Monitoring	3/4.10-1
4.10.C Fuel Storage Pool Water Level	3/4.10-2
4.10.D Control Rod Drive and Control Rod Drive Maintenance	3/4.10-3
4.10.E Extended Core Maintenance	3/4.10-4
4.10.F Spent Fuel Cask Handling	3/4.10-5
4.10.G Fuel Storage Reactivity Limit	3/4.10-8
4.11 High Energy Piping Integrity	3/4.11-1
4.12 Fire Protection Systems - Sections 4.12.A through 4.12.H - Deleted per Generic Letters 86-10 and 88-12 (Amendment 101)	
5.0 Design Features	5-1
5.1 Site	5-1
5.2 Reactor	5-1
5.3 Reactor Vessel	5-1
5.4 Containment	5-1
5.5 Fuel Storage	5-1
5.6 Seismic Design	5-2
6.0 Administrative Controls	6-1
6.1 Organization, Review, Investigation and Audit	6-1
6.2 Procedures and Programs	6-13
6.3 Action to be Taken in the Event of a REPORTABLE EVENT in Plant Operation	6-15
6.4 Action to be taken in the Event a Safety Limit is Exceeded	6-15
6.5 Plant Operating Records	6-15
6.6 Reporting Requirements	6-17
6.7 Environmental Qualification	6-21
6.8 Offsite Dose Calculation Manual (ODCM)	6-23
6.9 Process Control Program (PCP)	6-24
6.10 Major Changes to Radioactive Waste Treatment Systems (Liquid, Gaseous, Solid)	6-24
6.11 Radiation Protection Program	6-25
6.12 High Radiation Area	6-25

6.0 ADMINISTRATIVE CONTROLS (Cont'd.)

- d. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments;
 - e. A comparison of the predicted releases of radioactive materials in liquid and gaseous effluents and in solid waste to the actual releases for the period in which the changes were made;
 - f. An estimate of the exposure to plant operating personnel as a result of the change; and
 - g. Documentation of the fact that the change was reviewed and found acceptable by the On-site Review Function.
2. The change shall become effective upon review and acceptance by the On-site Review Function.

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

* Health Physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirements during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

6.0 ADMINISTRATIVE CONTROLS (Cont'd.)

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.
- c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by Health Physics in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Radiation Protection Supervisor. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area.

For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem** that are located within large areas, such as the containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted, and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote (such as use of closed circuit TV cameras), may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

** Measurement made at 30 cm (11.8 in) from the source of radioactivity.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-25

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3

DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated April 24, 1992, Commonwealth Edison Company (CECo, the licensee) proposed changes to the Technical Specifications for Dresden Nuclear Power Station, Units 2 and 3. The proposed changes add Section 6.11 - Radiation Protection Program and Section 6.12 - High Radiation Area. A similar amendment issued February 14, 1992, was approved for Quad Cities, Units 1 and 2.

2.0 EVALUATION

Regarding the first of the two new sections, since the change to Section 6.11 (Radiation Protection Program), is consistent with Section 6.11 of the Standard Technical Specifications (STS) it is administrative in nature and, therefore, we find it acceptable.

The second section being added is Section 6.12, High Radiation Area. Dresden Station currently complies with 10 CFR 20.203(c)(2) and (4) by requiring that all areas greater than 100 mrem per hour be locked, except during periods of access, or by providing direct surveillance to prevent unauthorized entry. 10 CFR 20.203(c)(5) allows licensees to apply for approval of methods not included in paragraphs (c)(2) and (c)(4) for controlling access to high radiation areas. The staff considers Section 6.12 of the STS to be an acceptable alternative method for controlling high radiation areas. CECo has proposed a new Section 6.12 entitled, "High Radiation Area" that is identical to the STS with two exceptions. Section 6.12.c of the STS uses the title "Health Physicist." The licensee has changed this to "Health Physics" which is the comparable function at Dresden Station. We find this acceptable.

The other exception from the STS involves the distance from the source for taking the dose rate measurement. The STS say 18 inches (45 cm), while the licensee proposes 30 cm (11.8 inches). The licensee's proposed distance is more conservative than the distance in the STS and is the distance specified in the revised version of 10 CFR 20, published May 21, 1991 (56 FR 23393). Thus, we find it acceptable to use this distance in Section 6.12 for Dresden Station.

The proposed changes to Dresden's Technical Specifications are identical to changes proposed by CECO for Quad Cities in a letter dated October 11, 1991, and approved by the staff in a letter dated February 14, 1992. The only difference within Section 6.11 and 6.12 between the Dresden amendment and Quad Cities amendment is that the keys specified in Section 6.12.2 of the proposed Technical Specifications for areas accessible to radiation doses greater than 1,000 mrem in one hour are maintained under the administrative control of the radiation protection supervisor at Dresden and under administrative control of the shift foreman on duty and/or the health physics supervision at Quad Cities. We find this difference acceptable.

In conclusion, based on the fact that these changes are consistent with the STS and these changes have been previously approved for Quad Cities, the staff finds the proposed changes for Dresden, Units 2 and 3, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change administrative procedures or requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (57 FR 24668). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Siegel

Date: September 11, 1992