

December 28, 1993

Docket Nos. 50-237
and 50-249

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III, Suite 500
1400 OPUS Place
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PDIII-2 p/f	ACRS (10)
OPA	OC/LFDCB

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M87201 AND M87202)

The Commission has issued the enclosed Amendment No. 124 to Facility Operating License No. DPR-19 and Amendment No. 118 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3. The amendments are in response to your application dated June 2, 1993.

The amendments would include Commonwealth Edison Company's Topical Report NFSR-0091, in Section 6.6 of the Technical Specifications. Topical Report NFSR-0091 has been reviewed and approved by the NRC staff.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

John F. Stang, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 124 to DPR-19
2. Amendment No. 118 to DPR-25
3. Safety Evaluation

cc w/enclosures:
See next page

OFC	LA:PD III-2	PM:PD III-2	D:PD III-2	OGC		
NAME	C. MOORE	J. STANG	J. DYER	S. HOM		
DATE	1/3/93	1/13/93	12/27/93	12/13/93	1/ /93	1 /93
COPY	YES/NO	YES/NO	YES/NO	YES/NO	YES/NO	YES/NO

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NRC FILE CENTER COPY

Mr. D. L. Farrar
Commonwealth Edison Company

Dresden Nuclear Power Station
Unit Nos. 2 and 3

cc:

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Resident Inspectors Office
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Board of Supervisors of
Grundy County
Grundy County Courthouse
Morris, Illinois 60450

Regional Administrator
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799 Roosevelt Road, Bldg. #4
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124
License No. DPR-19

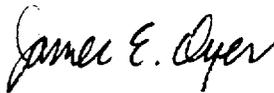
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated June 2, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 124, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 124

FACILITY OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

6-19

INSERT

6-19

6.0 ADMINISTRATIVE CONTROLS (Cont'd.)

4. Core Operating Limits Report

a. Core operating limits shall be established and documented in the Core Operating Limits Report before each reload cycle or any remaining part of a reload cycle for the following:

- 1) The Control Rod Withdrawal Block Instrumentation for Table 3.2-3 of Specification 3.2.C.
- 2) The Average Planar Linear Heat Generation Rate (APLHGR) Limit and associated APLHGR multipliers for Specifications 3.5.I, 3.5.D.2, and 3.6.H.3.f.
- 3) The Local Steady State Linear Heat Generation Rate (LHGR) for Specification 3.5.J.
- 4) The Local Transient Linear Heat Generation Rate (LHGR) for Specification 3.5.K.
- 5) The Minimum Critical Power Operating Limit for Specification 3.5.L. This includes rated and off-rated flow conditions.

b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC in the latest approved revision or supplement of the topical reports describing the methodology. For Dresden Unit 2, the topical reports are:

- 1) ANF-1125(P)(A), "Critical Power Correlation - ANFB."
- 2) ANF-524(P)(A), "ANF Critical Power Methodology for Boiling Water Reactors."
- 3) XN-NF-79-71(P)(A), "Exxon Nuclear Plant Transient Methodology for Boiling Water Reactors."
- 4) XN-NF-80-19(P)(A), "Exxon Nuclear Methodology for Boiling Water Reactors."
- 5) XN-NF-85-67(P)(A), "Generic Mechanical Design for Exxon Nuclear Jet Pump Boiling Water Reactors Reload Fuel."
- 6) XN-NF-81-22(P)(A), "Generic Statistical Uncertainty Analysis Methodology."
- 7) ANF-913(P)(A), "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analyses."
- 8) Commonwealth Edison Company Topical Report NFSR-0091, "Benchmark of CASMO/MICROBURN BWR Nuclear Design Methods," and associated Supplements on Neutronic Licensing Analyses (Supplement 1) and LaSalle County Unit 2 Benchmarking (Supplement 2).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-25

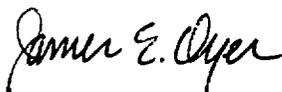
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated June 2, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

6-19

INSERT

6-19

6.0 ADMINISTRATIVE CONTROLS (Cont'd.)

- 3) The Local Steady State Linear Heat Generation Rate (LHGR) for Specification 3.5.J.
 - 4) The Local Transient Linear Heat Generation Rate (LHGR) for Specification 3.5.K.
 - 5) The Minimum Critical Power Operating Limit for Specification 3.5.L. This includes rated and off-rated flow conditions.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC in the latest approved revision or supplement of the topical reports describing the methodology. For Dresden Unit 3, the topical reports are:
- 1) ANF-1125(P)(A), "Critical Power Correlation - ANFB."
 - 2) ANF-524(P)(A), "ANF Critical Power Methodology for Boiling Water Reactors."
 - 3) XN-NF-79-71(P)(A), "Exxon Nuclear Plant Transient Methodology for Boiling Water Reactors."
 - 4) XN-NF-80-19(P)(A), "Exxon Nuclear Methodology for Boiling Water Reactors."
 - 5) XN-NF-85-67(P)(A), "Generic Mechanical Design for Exxon Nuclear Jet Pump Boiling Water Reactors Reload Fuel."
 - 6) XN-NF-81-22(P)(A), "Generic Statistical Uncertainty Analysis Methodology."
 - 7) ANF-913(P)(A), "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analyses."
 - 8) Commonwealth Edison Company Topical Report NFSR-0091, "Benchmark of CASMO/MICROBURN BWR Nuclear Design Methods," and associated Supplements on Neutronic Licensing Analyses (Supplement 1) and LaSalle County Unit 2 Benchmarking (Supplement 2).
- c. The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.
- d. The Core Operating Limits Report, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

B. Reportable Events

Reportable events will be submitted as required by 10 CFR 50.73.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 124 TO FACILITY OPERATING LICENSE NO. DPR-19
AND AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. DPR-25
COMMONWEALTH EDISON COMPANY
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3
DOCKET NOS. 50-237 AND 50-249

1.0 INTRODUCTION

By letter dated June 2, 1993, the Commonwealth Edison Company (CECo, the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-19 and DPR-25 for the Dresden Nuclear Power Station, Units 2 and 3. The proposed amendments would include CECo's Topical Report NFSR-0091 in Section 6.6 of the TS.

2.0 EVALUATION

Section 6.6 of the TS lists the specific NRC staff approved analytical methods used to determine the core operating limits. CECo has requested that Topical Report NFSR-0091 be added to Section 6.6 of the TS. By letter dated March 22, 1993, the NRC staff reviewed and approved the use of Topical Report NFSR-0091 for determining core operating limits at the Dresden Station. The TS change is an administrative change to add the Topical Report to Section 6.6 of the TS. This will enable CECo to use the approved analytical methods to determine further core operating limits. Based on the above, the staff finds the change to the TS (1) is purely administrative in nature, (2) will leave all current safety limits unchanged, and (3) will not result in a reduction in the margin of safety. Therefore, the staff finds the proposed change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John F. Stang

Date: December 28, 1993