UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 12, 1983

Docket Nos. 50-237/249
LS05-83-12-010

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: CONTROL ROD DRIVE SCRAM TIME TESTING

Dresden Nuclear power Station, Unit Nos. 2 and 3

The Commission has issued the enclosed Amendment No. 79 to Provisional Operating License No. DPR-19 and Amendment No.70 to Facility Operating License No. DPR-25 for the Dresden Nuclear Power Station, Units 2 and 3, respectively. These amendments consist of changes to the Technical Specifications in response to your application dated May 2, 1983.

The amendments authorize changes to the Technical Specifications which specify what the reactor pressure should be when conducting scram testing following a refueling outage and revise the requirement of isolating the control rod drive pumps to apply only to single rod scram testing.

A Notice of Consideration of Issuance of Amendments to Licenses and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the Federal Register on October 26, 1983 (48 FR 49579). No request for hearing was received and no comments were received.

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16

A copy of our related Safety Evaluation is also enclosed. This action will appear in the Commission's Monthly Notice publication in the Federal Register.

Sincerely,

ORIGINAL SIGNED BY

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosures:
1. Amendment No. 79 to DPR-19
2. Amendment No. 70 to DPR-25
3. Safety Evaluation

cc w/enclosure:
See next page

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Mr. Dennis L. Farrar

cc
Isham, Lincoln & Beale
Counselors at Law
One First National Plaza, 42nd Floor
Chicago, Illinois 60603

Mr. Doug Scott
Plant Superintendent
Dresden Nuclear Power Station
Rural Route #1
Morris, Illinois 60450

U. S. Nuclear Regulatory Commission
Resident Inspectors Office
Dresden Station
Rural Route #1
Morris, Illinois 60450

Chairman
Board of Supervisors of
Grundy County
Grundy County Courthouse
Morris, Illinois 60450

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: Regional Radiation Representative
230 South Dearborn Street
Chicago, Illinois 60604

James G. Keppler, Regional Administrator
Nuclear Regulatory Commission, Region III
799 Roosevelt Street
Glen Ellyn, Illinois 60137

Mr. Gary N. Wright, Manager
Nuclear Facility Safety
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

Docket No. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 79
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated May 2, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I:

   B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission:

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
1. Changes to the Technical Specifications

Date of Issuance: December 12, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 79

PROVISIONAL OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Replace the following page of the Appendix A Technical Specifications with the enclosed page. This page contains the captioned amendment number and vertical lines indicating the changes.

Replace Page

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3.3 LIMITING CONDITION FOR OPERATION

C. Scram Insertion Times

1. The average scram insertion time, based on the de-energization of the scram pilot valve solenoids as time zero, of all operable control rods in the reactor power operation condition shall be no greater than:

% Inserted From Fully Withdrawn	Avg. Scram Insertion Times (sec)
5	0.375
20	0.900
50	2.00
90	3.50

The average of the scram insertion times for the three fastest control rods of all groups of four control rods in a two by two array shall be no greater than:

% Inserted From Fully Withdrawn	Avg. Scram Insertion Times (sec)
5	0.398
20	0.954
50	2.120
90	3.800

2. The maximum scram insertion time for 90% insertion of any operable control rod shall not exceed 7.00 seconds.

4.3 SURVEILLANCE REQUIREMENTS

C. Scram Insertion Times

1. After each refueling outage, prior to operation greater than 30 percent of rated thermal power, all control rods shall be subject to scram-time tests from the fully withdrawn position with reactor pressure above 800 psig. If the control rods are tested individually, their hydraulic control units shall be isolated from the control rod drive pumps.

2. At 16 week intervals, at least 50% of the control rod drives shall be tested as in 4.3.C.1 so that every 32 weeks all of the control rods shall have been tested. Whenever 50% or more of the control rod drives have been tested, an evaluation shall be made to provide reasonable assurance that proper control rod drive performance is being maintained.

3. Following completion of each set of scram testing as described above, the results will be compared against the average scram speed distribution used in the transient analysis to verify the applicability of the current MCPR Operating Limit. Refer to Specification 3.5.K.

Amendment No. 19, 75, 79

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

Docket No. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated May 2, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I:

   B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission:

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-25 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 70, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
1. Changes to the Technical Specifications

Date of Issuance: December 12, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-245

Replace the following page of the Appendix A Technical Specifications with the enclosed page. This page contains the captioned amendment number and vertical lines indicating the changes.

Replace Page

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3.3 LIMITING CONDITION FOR OPERATION

C. Scram Insertion Times

1. The average scram insertion time, based on the de-energization of the scram pilot valve solenoids as time zero, of all operable control rods in the reactor power operation condition shall be no greater than:

% Inserted From Fully Withdrawn	Avg. Scram Insertion Times (sec)
5	0.375
20	0.900
50	2.00
90	3.50

The average of the scram insertion times for the three fastest control rods of all groups of four control rods in a two by two array shall be no greater than:

% Inserted From Fully Withdrawn	Avg. Scram Insertion Times (sec)
5	0.398
20	0.954
50	2.120
90	3.800

2. The maximum scram insertion time for 90% insertion of any operable control rod shall not exceed 7.00 seconds.

4.3 SURVEILLANCE REQUIREMENTS

C. Scram Insertion Times

1. After each refueling outage, prior to operation greater than 30 percent of rated thermal power, all control rods shall be subject to scram-time tests from the fully withdrawn position with reactor pressure above 800 psig. If the control rods are tested individually, their hydraulic control units shall be isolated from the control rod drive pumps.

2. At 16 week intervals, at least 50% of the control rod drives shall be tested as in 4.3.C.1 so that every 32 weeks all of the control rods shall have been tested. Whenever 50% or more of the control rod drives have been tested, an evaluation shall be made to provide reasonable assurance that proper control rod drive performance is being maintained.

3. Following completion of each set of scram testing as described above, the results will be compared against the average scram speed distribution used in the transient analysis to verify the applicability of the current MCPR Operating Limit. Refer to Specification 3.5.K.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 79 TO PROVISIONAL OPERATING LICENSE NO. DPR-19

AND AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-25

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3

DOCKET NOs. 50-237 AND 50-249

1.0 INTRODUCTION

During the inspection period May 9 - June 5, 1981, as reported in an Inspection Report for Dresden Units 2 and 3 dated July 16, 1981, the NRC's resident inspectors noted that the licensee fulfilled the requirement of Technical Specification 4.3.C.1, which states that "[a]fter each refueling outage and prior to operation with reactor pressure above 800 psig, all control rods shall be subject to scram time tests from the fully withdrawn position", by performing the testing with the reactor depressurized. It was the inspectors' opinion that the wording of 4.3.C.1 would be better interpreted to mean that the tests should be performed above 800 psig but before power operation. Following a review, the staff proposed that the test should be conducted while the reactor pressure is greater than 800 psig and prior to exceeding 950 psig. It was also agreed that scram time tests would be conducted prior to exceeding 30% of the reactor licensed power following a refueling outage. In a letter dated May 2, 1983 the licensee proposed Technical Specification (TS) changes to clarify this procedure.

In the same submittal the licensee also requested TS changes that allowed Control Rod Drive (CRD) pumps to be isolated only during single rod scram testing. This request was based on a proposed purchase of equipment that would allow multiple scram testing; e.g. during any reactor scram. It is the licensee's position that during full core testing there is insufficient charging system capacity to bias scram insertion times and that the current BWR Standard Technical Specifications reflect this distinction.

A Notice of Consideration of Issuance of Amendments to Licenses and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related the requested action was published in the Federal Register on October 26, 1983 (48 FR 49579). No request for hearing was received and no comments were received.

2.0 EVALUATION

Based on an analysis of the proposed procedures for scram time testing following a refueling outage, the staff has determined that such testing should be conducted with the reactor pressure above 800 psig but before the pressure exceeds 950 psig. The tests should also be performed prior to exceeding 30% of the reactor licensed power. The changes proposed by the licensee reflect these staff determinations and are, therefore, acceptable. The staff also agrees, after a technical review and an examination of the BWR Standard Technical Specifications, that the request to have the CRD pumps isolated only during single rod scram testing meets staff criteria and is, therefore, also acceptable.

Thus, the staff finds that the licensee's proposal to improve the wording of Technical Specification Sections 4.3.C.1 and 4.3.C.2 for Dresden Units 2 and 3 to reflect the staff's position, as stated above, is acceptable.

3.0 ENVIRONMENTAL QUALIFICATION

The staff has determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, the staff further concludes that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal, need not be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

5.0 ACKNOWLEDGEMENT

R. A. Gilbert prepared this Safety Evaluation

Dated: December 12, 1983

Docket

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 12, 1983

Docket Nos. 50-237/249

MEMORANDUM FOR: James R. Miller, Chief
Operating Reactors Branch #3, DL

FROM: Dennis M. Crutchfield, Chief
Operating Reactors Branch #5, DL

SUBJECT: REQUEST FOR PUBLICATION IN MONTHLY FR NOTICE - NOTICE OF ISSUANCE OF AMENDMENTS TO OPERATING LICENSES

Commonwealth Edison Company, Docket Nos. 50-237 and 50-249, Dresden Nuclear Power Station, Units 2 and 3, Grundy County, Illinois

Date of application for amendments: May 2, 1983

Brief description of amendment: The amendments approve changes to the Technical Specifications which specify what the reactor pressure should be when conducting scram testing following a refueling outage and revise the requirement of isolating the control rod drive pumps to apply only to single rod scram testing.

Date of issuance: December 12, 1983

Effective date: December 12, 1983

Amendment No. 79

Provisional Operating License No.: DRP-19.

Amendment No. 70

Facility Operating License No.: DPR-25

Amendments revised the Appendix A Technical Specifications.

Date of initial notice in Federal Register: October 26, 1983 (48 FR 49579)

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated December 12, 1983. No public or State comments were received

with respect to the Commission's proposed determination that the amendment would not involve a significant hazards consideration.

Local Public Document Room location: Morris Public Library, 604 Liberty Street, Morris, Illinois 60451.

ORIGINAL SIGNED BY
Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

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with respect to the Commission's proposed determination that the amendments would not involve a significant hazards consideration.

Local Public Document Room location: Morris Public Library, 604 Liberty Street, Morris, Illinois 60451.

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing