Docket Nos. 50-237/249/254/265 LS05-84-02-054

> Mr. Dennis L. Farrar Director of Nuclear Licensing Commonwealth Edison Company Post Office Box 767 Chicago, Illinois 60690

Dear Mr. Farrar:

SUBJECT: SAFETY/RELIEF VALVE POSITION INDICATION

Re:

Dresden Nuclear Power Station, Unit Nos. 2 and 3

Ouad Cities Station, Unit Nos. 1 and 2

The Commission has issued the enclosed Amendment No. 81 to Provisional Operating License No. DPR-19 for Dresden Unit 2 and Amendment Nos. 73, 87 and 82 to Facility Operating License Nos. DPR-25, DPR-29 and DPR-30 for Dresden Unit 3 and Quad Cities Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 12, 1982.

The amendments authorize changes to the Technical Specifications which revise footnotes to Table 4.2.1 in the Dresden 2 and 3 TS and Table 3.2.4 in the Quad Cities TS. These changes require the restoration of all safety/relief valve inoperable position indication prior to startup only if the reactor is in a cold shutdown condition for more than seventy-two hours.

A Notice of Consideration of Issuance of Amendments to Licenses and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the <u>Federal Register</u> on November 22, 1983 (48 FR 52807). No request for hearing was received and no comments were received.

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SEO/ EX (16)

A copy of related Safety Evaluation is also enclosed. This action will appear in the Commission's Monthly Notice publication in the Federal Register.

Sincerely,

### Original signed by

Dennis M. Crutchfield, Chief Operating Reactors Branch #5 Division of Licensing

#### Enclosures:

Amendment No. 81 to
 License No. DPR-19
 Amendment No. 73 to
 License No. DPR-25
 Amendment No. 87 to
 License No. DPR-29
 Amendment No. 82 to
 License No. DPR-30

5. Safety Evaluation

cc w/enclosures:
See next page

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A/1/84

DE ORB #2 DVassallo 1 /4/84

0ELD DREPKA 7 /70/84 cc Isham, Lincoln & Beale Counselors at Law One First National Plaza, 42nd Floor Chicago, Illinois 60603

Mr. Doug Scott Plant Superintendent Dresden Nuclear Power Station Rural Route #1 Morris, Illinois 60450

U. S. Nuclear Regulatory Commission Resident Inspectors Office Dresden Station Rural Route #1 Morris, Illinois 60450

Chairman
Board of Supervisors of
Grundy County
Grundy County Courthouse
Morris, Illinois 60450

U. S. Environmental Protection Agency Federal Activities Branch Region V Office ATTN: Regional Radiation Representative 230 South Dearborn Street Chicago, Illinois 60604

James G. Keppler, Regional Administrator Nuclear Regulatory Commission, Region III 799 Roosevelt Street Glen Ellyn, Illinois 60137

Mr. Gary N. Wright, Manager Nuclear Facility Safety Illinois Department of Nuclear Safety 1035 Outer Park Drive, 5th Floor Springfield, Illinois 62704

The Honorable Tom Corcoran United States House of Representatives Washington, D.C. 20515

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Mr. Marcel DeJaegher, Chairman Rock Island County Board of Supervisors Rock Island County Court House Rock Island, Illinois 61201

Mr. D. R. Stichnoth, President Iowa-Illinois Gas & Electric Co. 206 East Second Avenue Davenport, Iowa 52801

Susan N. Sekuler Assistant Attorney General Environmental Control Division 188 W. Randolph Street Suite 2315 Chicago, Illinois 60601



## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

### DRESDEN NUCLEAR POWER STATION, UNIT NO. 2

#### AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 81 License No. DPR-19

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated January 12, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Provisional Operating License No. DPR-19 is hereby amended to read as follows:
  - Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield, Chief Operating Reactors Branch #5

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: February 24, 1984

### ATTACHMENT TO LICENSE AMENDMENT NO. 81

### PROVISIONAL OPERATING LICENSE NO. DPR-19 AND

### AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NOS. 50-237/249

Revise the Technical Specifications by deleting the following page and inserting the enclosed page.

PAGE

45

### NOTES:

- 7. Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular Safety/Relief Valve.
- 8. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however if the reactor is in a cold shutdown condition for more than seventy-two hours, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be restored in thirty days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.
- 9. The functional Test of the Scram Discharge Volume float switch shall include actuation of the switch using a water column.
- 10. Functional test shall include verification of the second level undervoltage (degraded voltage) timer bypass and shall verify operation of the degraded voltage 5-minute timer and inherent 7-second timer.



### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

### DRESDEN NUCLEAR POWER STATION, UNIT NO. 3

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73 License No. DPR-25

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated January 12, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-25 is hereby amended to read as follows:
  - Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield, Chief Operating Reactors Branch #5

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: February 24, 1984

### ATTACHMENT TO LICENSE AMENDMENT NO. 81

### PROVISIONAL OPERATING LICENSE NO. DPR-19 AND

### AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NOS. 50-237/249

Revise the Technical Specifications by deleting the following page and inserting the enclosed page.

PAGE

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### NOTES:

- 7. Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular Safety/Relief Valve.
- 8. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however if the reactor is in a cold shutdown condition for more than seventy-two hours, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be restored in thirty days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.
- 9. The functional Test of the Scram Discharge Volume float switch shall include actuation of the switch using a water column.
- 10. Functional test shall include verification of the second level undervoltage (degraded voltage) timer bypass and shall verify operation of the degraded voltage 5-minute timer and inherent 7-second timer.



### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

# COMMONWEALTH EDISON COMPANY AND IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES STATION, UNIT NO. 1

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87 License No. DPR-29

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated January 12, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-29 is hereby amended to read as follows:
  - Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief Operating Reactors Branch #2

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: February 24, 1984

# ATTACHMENT TO LICENSE AMENDMENT NOS. 87 AND 82 FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30 DOCKET NOS. 50-254/265

Revise the Appendix "A" Technical Specifications by deleting the following page and inserting the enclosed page.

PAGE

3.2/4.2-15a

#### Notes

- 1. Instrument channels required during power operation to monitor postaccident conditions.
- 2. Provisions are made for local sampling and monitoring of drywell atmosphere.
- 3. In the event any of the instrumentation becomes inoperable for more than 7 days during reactor operation, initiate an orderly shutdown and be in the cold shutdown condition within 24 hours.
- 4. From and after the date that one of these parameters is reduced to one indication, continued operation is not permissible beyond thirty days unless such instrumentation is sooner made operable. In the event that all indication of these parameters is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition in twenty-four (24) hours.
- 5. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however if the reactor is in a cold shutdown condition for more than seventy-two (72) hours, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be restored in thirty days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.



## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D, C, 20555

# COMMONWEALTH EDISON COMPANY AND IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES STATION, UNIT NO. 2

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82 License No. DPR-30

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated January 12, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-30 is hereby amended to read as follows:
  - B. Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 82, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief Operating Reactors Branch #2

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: February 24, 1984

# ATTACHMENT TO LICENSE AMENDMENT NOS. 87 AND 82 FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30 DOCKET NOS. 50-254/265

Revise the Appendix "A" Technical Specifications by deleting the following page and inserting the enclosed page.

PAGE

3.2/4.2-15a

#### QUAD-CITIES DPR-30

#### Notes

- 1. Instrument channels required during power operation to monitor postaccident conditions.
- 2. Provisions are made for local sampling and monitoring of drywell atmosphere.
- 3. In the event any of the instrumentation becomes inoperable for more than 7 days during reactor operation, initiate an orderly shutdown and be in the cold shutdown condition within 24 hours.
- 4. From and after the date that one of these parameters is reduced to one indication, continued operation is not permissible beyond thirty days unless such instrumentation is sooner made operable. In the event that all indication of these parameters is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition in twenty-four (24) hours.
- 5. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however if the reactor is in a cold shutdown condition for more than seventy-two (72) hours, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be restored in thirty days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 81 TO PROVISIONAL OPERATING LICENSE NO. DPR-19

AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-25

AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-29

AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NOS. 50-237, 50-249, 50-254, AND 50-265

### 1.0 INTRODUCTION

On January 12, 1982, Commonwealth Edison Company (CECo) proposed amendments to Appendix A of Operating Licenses DPR-19, DPR-25, DPR-29, and DPR-30. The proposal requested a change to a Limiting Condition for Operation (LCO) concerning safety and relief valve position indications. The present LCO, proposed in fulfillment of TMI-2 lessons learned Category A requirements, prohibited plant startup from a shutdown condition unless all position indication is operable. The change requested is a modification of the LCO to require the restoration of all inoperable position indication prior to startup only if the reactor is in a cold shutdown condition for more than seventy-two hours.

A Notice of Consideration of Issuance of Amendments to Licenses and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to the requested action was published in the <u>Federal Register</u> on November 22, 1983 (48 FR 52807). No request for hearing was received and no comments were received.

### 2.0 EVALUATION

There are two methods of position indication (thermocouples and acoustical monitors) available for use on each safety or safety/relief valve. Continued operation without restriction is presently allowed if one indication becomes inoperable. If both become inoperable on one or more valves and indication cannot be restored within thirty days, the reactor must be brought to cold shutdown within twenty-four hours and indication restored. In imposing the present requirements, it was the staff's intent to provide assurance that, since the indication provides no direct safety function, it would generally be available. The staff believes that the thirty-day requirement meets

this intent and, therefore, that the present requirement to restore all position indication during all outages, no matter how short, is overly restrictive since it could cause extensions of short-term outages associated with scram recoveries and minor maintenance. The proposed change to position indication restoration applies only if the reactor is in cold shutdown for more than seventy-two hours. This is acceptable to the staff.

### 3.0 ENVIRONMENTAL CONSIDERATION

The staff has determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, the staff further concludes that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to  $10 \ \text{CFR} \ 51.5(d)(4)$ , that an environmental impact statement, or negative declaration and environmental impact appraisal, need not be prepared in connection with the issuance of these amendments.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

### 5.0 ACKNOWLEDGEMENT

This evaluation was prepared by R. Gilbert.

Dated: February 24, 1984