

Docket
File

OCT 20 1970

Docket No. 50-237

Commonwealth Edison Company
ATTN: Mr. Byron Lee, Jr.
Assistant to the President
P.O. Box 767
Chicago, Illinois 60690

Change No. 6
License No. DPR-19

Gentlemen:

We have reviewed your Proposed Change No. 6, dated September 14, 1970, requesting a change to (a) Table 4.2.1, "Minimum Testing and Calibration Frequency for Core Cooling Instrumentation, Rod Blocks and Isolation;" (b) Table 4.6.1 on Inservice Inspection Requirements for Dresden Unit 2; and (c) Sections 3.7.D and 4.7.D on Primary Containment Isolation Valves, of the Technical Specifications (Appendix A) of Provisional Operating License DPR-19 (Dresden Nuclear Power Station Unit 2). These matters were further discussed with you at meetings held on September 18, 1970 and October 7, 1970.

At these meetings we discussed revisions to the proposed in-service inspection program for furnace sensitized stainless steel components, as submitted, to include visual inspection of the jet pump riser supports at the first refueling outage and inspection requirements for non-flowing safe ends. In your telegram to us of October 13, 1970, you agreed to these changes, and they are now included in this change authorization.

On the basis of our review we have concluded that these proposed changes do not present significant hazards considerations not described or implicit in the safety analysis report and that there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, the following changes are made to the Technical Specifications (Appendix A) of Provisional Operating License No. DPR-19:

1. Add the containment monitoring instrumentation testing and frequency requirements as indicated on the enclosed "Table 4.2.1 (continued)."
2. On Page 118 add the following after Category M and before the note explaining the Category J Weld Breakdown:

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Note: The following inservice inspection program will be carried out on the parts identified below rather than that described in Table 4.6.1:

- A. During the outage following the malfunction which led to the plant shutdown on June 5, 1970, all of the furnace sensitized wrought stainless steel safe ends and their welds were PT and UT inspected and all unacceptable defect indications were removed.
- B. All of the furnace sensitized wrought stainless steel safe ends and their welds will be reinspected at the first refueling outage:
- (1) General Electric Company has drilled holes in the internal piping attached to the thermal sleeves of the two 10-inch core spray inlet nozzles to improve the flow that can be demonstrated to remove pocketed gases and prevent future gas pocketing in the upper part of their furnace sensitized wrought stainless steel safe ends.
- C. The sensitized heat affected zones adjacent to the welds and the welds in the stainless steel reactor coolant piping will be PT and UT examined during the first refueling outage in accordance with a program which will provide for examination of 10% of the welds.
- D. The furnace sensitized stainless steel internal brackets and their attaching welds inside the reactor pressure vessel including the jet pump riser supports, will be visually examined during the first refueling outage.
- E. All of those safe ends (including safe end welds) connected to piping containing non-flowing reactor coolant will be inspected at least once at any shutdown of one week's duration, or refueling outage, occurring within the first year of operation, but in any case not later than one year after plant startup for normal reactor operation.

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- F. The results of the inspection program shall be submitted and discussed with the Commission.
- G. Inspections B, C, and D above, shall be repeated at the second refueling. Pending review of the results of the inspection at the first refueling outage, the program for this second inspection may be modified with the Commission's approval to reflect the results of the first examination, improvements in the art, changes in approved codes and standards, and possible repair or replacement of components.

3. On Page 132

- A. Change Specification 3.7.D.3 to read "If Specifications 3.7.D.1 and 3.7.D.2 ..."
- B. Add a Specification 3.7.D.4 to read "The temperature of the main steamline air pilot valves shall be less than 150°F except as specified in 3.7.D.5 below."
- C. Add a Specification 3.7.D.5 to read "From and after the date that the temperature of any main steamline air pilot valve is found to be greater than 150°F, reactor operation is permissible only during the succeeding seven days provided the main steamline isolation valves are operable."
- D. Add a Specification 3.7.D.6 to read "When it is determined that it will take longer than seven days to reduce the temperature of any main steamline air pilot valve to less than 150°F, a report detailing the circumstances, actions and the estimated date for returning the air pilot valve temperature to a value less than 150°F shall be submitted to the AEC prior to the end of the seven-day period."
- E. Add a Specification 4.7.D.4 to read "The temperature of the main steamline air pilot valves shall be recorded daily."
- F. Add a Specification 4.7.D.5 to read "When it is determined that the temperature of any main steamline air pilot valve is greater than 150°F, the main steamline isolation valves

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shall be demonstrated to be operable immediately and daily thereafter. The demonstration of operability shall be according to Specification 4.7.D.1.c."

Sincerely,

Original Signed by
Peter A. Morris

Peter A. Morris, Director
Division of Reactor Licensing

Enclosure:

1. Table 4.2.1 (continued)

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DRL R/F
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E. E. Hall, GMR/H
J. A. Harris, PI
R. Leigh, OC
J. Buchanan, ORNL
A. A. Wells, ASLB
T. W. Laughlin, DTIE

See attached

CRESS	OFFICE ▶	DRL:BWR-2	DRL:BWR-2	DRL:BWR	DRL	DRL:DIR
DRL12 R12-115	NAME ▶	GCLainas:th	RTedesco	RSBoyd	RSchroeder	PAMorris
	DATE ▶	10/16/70	10/1 /70	10/17/70	10/2 /70	10/20/70

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Sincerely,

Peter A. Morris, Director
Division of Reactor Licensing

Enclosures:

1. Safety Evaluation
2. Table 4.2.1 (continued)

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bcc:

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OFFICE ▶	DRL: BWR-2	DRL: BWR-2	DRL: BWR	DRL	DRL: DIR
NAME ▶	GCLainas/gl	RLTedesco	RSBoyd	ASchroeder	PAMorris
DATE ▶	10/15/70	10/15/70	10/16/70	10/15/70	10/17/70