April 28, 1986

Docket No. 50-237

Mr. Dennis L. Farrar
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

DISTRIBUTION

Docket File	OELD	EButcher
NRC PDR	ELJordan	
Local PDR	BGrimes	
BWD#1 Rdg	ACRS (10)	
RBernero	TBarnhart (4)	
DVassallo	RGilbert	
CJamerson	RDiggs (w/TACs)	
MVirgilio	WJones	
CMiles	LHarmon	
JPartlow	MJohns, RIII	

SUBJECT: TECHNICAL SPECIFICATION RELATING TO A TEMPORARY EXTENSION OF DIESEL GENERATOR ALLOWABLE OUTAGE PERIOD (TAC 60894)

Re: Dresden Nuclear Power Station, Unit No. 2

The Commission has issued the enclosed Amendment No. 92 to Provisional Operating License No. DPR-19 for Dresden Unit 2. This amendment is in response to your application dated March 10, 1986.

The amendment changes the requirements in Technical Specification Section 3.9.B.2 relating to the allowable outage period for the Dresden 2/3 diesel generator (DG) from the present seven days to fourteen days on a one-time-only basis. This is needed by Commonwealth Edison Company so 10 CFR Part 50 Appendix R modifications to the 2/3 DG can be completed. This work can not be completed within the allowed seven-day period and can not be broken into smaller increments.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

ORIGINAL SIGNED BY

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:
1. Amendment No. 92 to License No. DPR-19
2. Safety Evaluation

cc w/enclosures:
See next page

DBL:PD#1	DBL:PD#1	OELD	DBL:PD#1
CJamerson	RGilbert:tm	Subject to concurrence of Mat'l Type	JZwolinski
4/3/86	4/4/86	4/8/86	4/29/86

cc:
Mr. Michael I. Miller
Isham, Lincoln & Beale
Three First National Plaza
Suite 5200
Chicago, Illinois 60602

Mr. Doug Scott
Plant Superintendent
Dresden Nuclear Power Station
Rural Route #1
Morris, Illinois 60450

U. S. Nuclear Regulatory Commission
Resident Inspectors Office
Dresden Station
Rural Route #1
Morris, Illinois 60450

Chairman
Board of Supervisors of
Grundy County
Grundy County Courthouse
Morris, Illinois 60450

Regional Administrator
Nuclear Regulatory Commission, Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Gary N. Wright, Manager
Nuclear Facility Safety
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 92
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated March 10, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 92, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 28, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 92

PROVISIONAL OPERATING LICENSE DPR-19

DOCKET NO. 50-237

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
3/4.9-4	3/4.9-4

3.9 LIMITING CONDITION FOR OPERATION (Cont'd.)

4.9 SURVEILLANCE REQUIREMENT (Cont'd.)

operable, provided that during such seven days the operable diesel generator shall be demonstrated to be operable at least once each day and two off-site lines are available.

The specified 7 day outage period will apply except for spring of 1986, at which time an additional 7 days (14 total) will be allowed, on a one time only basis, to complete 10CFR 50 Appendix R modifications to the 2/3 Diesel Generator.

3. From and after the date that one of the two 125 or 250V battery systems is made or found to be inoperable, except as specified in 3.9.B.4a or b, Unit shutdown shall be initiated within 2 hours and the unit shall be in cold shutdown in 24 hours unless the failed battery can be sooner made operable.

4. a. Each 125 or 250 volt battery may be inoperable for a maximum of 7 days per operating cycle for maintenance and testing.

   b. If it is determined that a battery need be replaced as a result of maintenance or testing, a specific battery may be inoperable for an additional 7 days per operating cycle.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 92 TO PROVISIONAL OPERATING LICENSE NO. DPR-19

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-237

1.0 INTRODUCTION

By letter dated March 10, 1986, Commonwealth Edison Company (the licensee) made application to amend the Technical Specifications (TS) of Dresden Station Unit 2 to extend the allowable outage period for the Unit 2/3 (Swing) Diesel Generator (DG) from seven to fourteen days. The extension was requested on a one-time-only basis to allow installation of 10 CFR Part 50 Appendix R modifications on the DG. The licensee stated that the required work cannot be completed within the existing seven-day provision nor can the modification be broken into smaller increments.

2.0 EVALUATION

The current limiting conditions for operation for the auxiliary electrical systems indicates in TS Section 3.9.B.2 that from the date that one of the DGs....is made or found to be inoperable for any reason, reactor operation is permissible....only during the succeeding seven days....provided that during such seven days the operable DG shall be demonstrated to be operable at least once each day and two off-site lines are available. As discussed in Section 1.0 above, the licensee is requesting that the seven-day period be extended to fourteen days on a one-time basis only.

The licensee furnished, in the amendment submittal, the following evaluation of the alternate sources of AC power which will available during the seven-day extension requested:

The 2/3 DG is used primarily to provide AC power, concurrently with the respective unit's dedicated DG, to those unit auxiliaries required for safe unit shutdown in the event of loss of off-site power and a loss of coolant accident. Presently, Dresden TS require the 2/3 DG to be operable during the operation of either unit with the above mentioned seven day repair period being the only exception. Lack of the 2/3 DG's availability for the extended period would have no effect on Dresden's Unit 3 as it is currently defueled and, therefore, by Unit 3 TS Section 3.9.D, required to have only one DG at its disposal. The Unit 3 dedicated DG would serve that purpose during the requested repair period extension.

Normally, the Unit 2 auxiliary loads are fed by both the unit auxiliary transformer and the reserve auxiliary transformer. The unit transformer is fed by the main unit generator and the reserve transformer is fed by the 138kV switchyard. In the past, the 138kV switchyard has been a highly reliable power source. If either the unit or reserve auxiliary transformer is lost, loads are automatically transferred to the remaining transformer. If power feed from both transformers is lost, the necessary loads may be carried by any one of three alternatives. The first source that would be normally utilized with this occurrence would be the Unit 2 dedicated DG. Although meant to run in conjunction with the 2/3 DG, it is amply sized to alone carry all necessary Unit 2 loads required to mitigate an accident (i.e., one of the two divisions of ECCS equipment). Historically, Dresden's DG have proven quite reliable, maintaining a failure rate of less than 4%. Based on this fact alone, there is minimal increased risk due to operating Unit 2 during the requested outage period. Additionally, per TS, the operability of the Unit 2 DG would be verified daily throughout the outage.

The likelihood of the Unit 2 DG failing after having lost Unit 2 off-site power is very small. Should this occur, Unit 2's required loads can be fed by the Unit 3 reserve auxiliary transformer which is fed by the 345kV switchyard. This is accomplished via a 4kV bus-tie linking the bus of the Unit 2 DG (24-1) with the Unit 3 Reserve Auxiliary Transformer. Actuation of this tie may be completed in a matter of minutes by closing breakers from the main control room. The Unit 3 reserve auxiliary transformer is sufficiently sized to carry these additional loads. To provide assurance that the bus-tie capability is available throughout the proposed extended outage, those breakers which allow the linking would be demonstrated operable by cycling the breakers at the onset of the outage.

The third available supply of AC power is the dedicated Unit 3 DG. The same 4kV bus tie which enables the Unit 2 emergency bus to be fed from the 345kV system may be used to supply that bus from the Unit 3 dedicated DG. Utilization of the Unit 3 DG in this way is allowable, by Unit 3 TS, since no fuel will be in the Unit 3 reactor vessel. In an effort to ensure operability of the Unit 3 DG, an operability check would be made at the onset of the second seven day period of the outage.

The staff finds the availability of AC power during the additional seven-day period acceptable.

In addition to the above evaluation, the licensee proposed compensatory measures to provide additional assurance of backup AC power availability. These are:

1. At the onset of the repair period, the 4kV bus cross-tie breakers will be separately checked for operability.

2. The Unit 2 dedicated DG will be operationally checked daily (currently required by TS - will be continued during extended outage period).

3. Unit 3 fuel load will be delayed until the completion of the requested outage.

4. At the onset of the second seven day period of the outage operability of the Unit 3 DG will be verified.

5. Prior to the outage, the unit operators will be alerted as to the available alternate sources of AC power and their proper implementation.

The staff requested that the licensee provide clarification of the word "alerted" in Item 5 and were informed that it would be by the issuance of a special training memorandum to all licensed personnel. With this clarification the staff finds that the compensatory measures are acceptable and will further ensure that backup AC power will be available during the additional seven-day period.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a change to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: R. Gilbert

Dated: April 28, 1986