

JAN 8 1973

Docket No. 50-237

Commonwealth Edison Company  
ATTN: Mr. L. D. Butterfield, Jr.  
Nuclear Licensing Administrator  
Post Office Box 767  
Chicago, Illinois 60690

Change No. 17  
License No. DPR-19

Gentlemen:

We have reviewed your Proposed Change No. 17 dated January 31, 1972, as supplemented by letter dated July 31, 1972, requesting changes to Table 4.6.1 to the Technical Specifications for Facility Operating License No. DPR-19 for Dresden Unit 2. It is our understanding that Proposed Change No. 17 supersedes Proposed Change No. 10 dated April 14, 1971. Proposed Change No. 17 would bring certain inservice inspection requirements for Dresden Unit 2 into conformance with those of Section XI of the ASME Boiler and Pressure Vessel Code.

Based on our review of use and inspections of furnace sensitized steel, we have modified your proposed specifications to require a special inservice inspection of the furnace sensitized stainless steel components at the first refueling outage after five years of operation.

We have concluded that the proposed changes, as modified, do not present significant hazards considerations not described or implicit in the Dresden Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered.

Pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications of Facility Operating License No. DPR-19 are hereby changed

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JAN 8 1973

to include a revised inservice inspection program by replacing the existing pages 98, and 100 through 107 with the revised pages 98, and 100 through 107 appended hereto.

Sincerely,

151

Donald J. Skovholt  
Assistant Director for  
Operating Reactors  
Directorate of Licensing

Enclosures:

Revised pages 98, and  
100 through 107

cc w/enclosures:

John W. Rowe, Esquire  
Isham, Lincoln & Beale  
Counselors at Law  
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Chicago, Illinois 60670

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604 Liberty Street  
Morris, Illinois 60451

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OFFICE ▶	L:ORB #2	L:ORB #2	L:ORB #2	L:OR	L:RS	L:OR
SURNAME ▶	RDSilver:sjh	RMDiggs	DLZiemann	DJSkovholt	RMaccary	DJSkovholt
DATE ▶	1/4/73	1/4/73	1/4/73	1/ 73	1/5/73	1/8/73



UNITED STATES  
ATOMIC ENERGY COMMISSION

WASHINGTON, D.C. 20545

January 8, 1973

Docket No. 50-237

Commonwealth Edison Company  
ATTN: Mr. L. D. Butterfield, Jr.  
Nuclear Licensing Administrator  
Post Office Box 767  
Chicago, Illinois 60690

Change No. 19  
License No. DPR-19

Gentlemen:

We have reviewed your Proposed Change No. 17 dated January 31, 1972, as supplemented by letter dated July 31, 1972, requesting changes to Table 4.6.1 to the Technical Specifications for Facility Operating License No. DPR-19 for Dresden Unit 2. It is our understanding that Proposed Change No. 17 supersedes Proposed Change No. 10 dated April 14, 1971. Proposed Change No. 17 would bring certain inservice inspection requirements for Dresden Unit 2 into conformance with those of Section XI of the ASME Boiler and Pressure Vessel Code.

Based on our review of use and inspections of furnace sensitized steel, we have modified your proposed specifications to require a special inservice inspection of the furnace sensitized stainless steel components at the first refueling outage after five years of operation.

We have concluded that the proposed changes, as modified, do not present significant hazards considerations not described or implicit in the Dresden Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered.

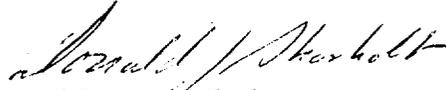
Pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications of Facility Operating License No. DPR-19 are hereby changed

*Handwritten notes:*  
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1/10/73

January 8, 1973

to include a revised inservice inspection program by replacing the existing pages 98, and 100 through 107 with the revised pages 98, and 100 through 107 appended hereto.

Sincerely,



Donald J. Skovholt  
Assistant Director for  
Operating Reactors  
Directorate of Licensing

Enclosures:

Revised pages 98, and  
100 through 107

cc w/enclosures:

John W. Rowe, Esquire  
Isham, Lincoln & Beale  
Counselors at Law  
One First National Plaza  
Chicago, Illinois 60670

Morris Public Library  
604 Liberty Street  
Morris, Illinois 60451

The performance of reactor coolant leakage detection system will be evaluated during the first five years of station operation and the conclusions of this evaluation will be reported to the AEC.

It is estimated that the main steam line tunnel leakage detection system is capable of detecting of the order of 3000 lb/hr. The system performance will be evaluated during the first five years of plant operation and the conclusions of the evaluation will be reported to the AEC.

E. Safety and Relief Valves - Experience in safety valve operation shows that a testing of 50% of the safety valves per refueling outage is adequate to detect failures or deterioration. The tolerance value is specified in Section III of the ASME Boiler and Pressure Vessel Code as  $\pm 1\%$  of design pressure. An analysis has been performed which shows that with all safety valves set 1% higher the reactor coolant pressure safety limit of 1375 psig is not exceeded. Solenoid actuated relief valves are used to avoid activation of the safety valves. In view of the fact that the solenoid activated relief valves are more complicated, it is prudent to test them at each refueling outage. The safety valves are required to be operable above the design pressure (90 psig) at which the core spray sub-systems are not designed to deliver full flow.

F. Structural Integrity - A pre-service inspection of the components listed in Table 4.6.1 will be conducted after site erection to assure the system is free of gross defects and as a reference base for later inspections. Prior to operation, the reactor primary system will be free of gross defects. In addition, the facility has been designed such that gross defects should not occur through life. The inspection program given in Table 4.6.1 was based on Section XI of the ASME Boiler and Pressure Vessel Code, 1971

which was followed except where accessibility for inspection was not provided. The Commonwealth Edison Company recognizes the importance of inspection of those areas which are presently not accessible and will study and implement, if practicable, new means to include those areas within the inspection program. This inspection provides further assurance that gross defects are not occurring after the system is in service. This inspection will reveal problem areas should they occur before a leak develops.

The special inspection of the main feed and steam lines is to provide added protection against pipe whip. The Category J GRP I welds are selected on the basis of an analysis that shows these welds are the highest stress welds and that due to their physical location, a break would result in the least interference and maximum energy upon impact with the drywell. These welds are the only ones which offer any significant risk and are therefore inspected four times as often as the other welds within the drywells.

Category J GRP II welds are selected because without regard for the operating stress levels and interfering equipment, they have sufficient theoretical energy to penetrate and would propel the pipe toward the containment. They are therefore included in first inspection. Upon consideration of impact angle, interfering equipment and distance pipe travels, no substantial risk is involved and no extra inspection is needed.

In addition, extensive visual inspection for leaks will be made periodically on critical systems. The inspection program specified encompasses the major areas of the vessel and piping systems within the drywell. The inspection period is based on the observed rate of growth of defects from fatigue studies sponsored by the AEC.

TABLE 4.6.1

IN-SERVICE INSPECTION REQUIREMENTS FOR DRESDEN UNIT 2

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examinations (1)
A	Longitudinal and Circumferential Shell Welds in Core Region	Volumetric	10% of each longitudinal weld and 5% of each circumferential weld during each 10 year inspection interval	Note: Not applicable with present plant design
B	Longitudinal and Circumferential Welds in Shell (other than those of Category A & C) and meridional and circumferential welds in bottom head and closure head (other than those of Category C)	Volumetric	10% of each longitudinal weld and 5% of each meridional weld during each 10 year inspection interval	<p>Accessible top 10 ft. of vertical vessel weld @ 2, places (100% inspected in 10 years for approximately 2 ft. each year)</p> <p>10% of meridional welds and 5% of circumferential welds in vessel closure head</p> <p>Note: Bottom head closure not applicable with present plant design</p>
C	Vessel-to-flange and head-to-flange-circumferential welds	Volumetric	Cumulative 100% coverage at end of 10 year interval	10% of vessel-to-flange and head-to-flange circumferential weld area each year

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TABLE 4.6.1 (cont)

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examination <sup>(1)</sup>
D	Primary nozzle-to-vessel and nozzle-to-head welds and nozzle-to-vessel and nozzle-to-head inside radiused section	Volumetric	Cumulative 100% coverage at end of 10 year interval	Nozzle Welds: Recirc. Outlet (2) - 1/5 years Recirc. Inlet (10) - 1/year Isolation Condenser Outlet (2) - 1/5 years Core Spray Inlet (2) - 1/5 years Control Rod Drive Return (1) - 1/10 years Standby Liquid Control (1) - 1/10 years Head Instrumentation (2) - 1/5 years Head Spray Inlet (1) - 1/10 years
E-1	Vessel penetrations including instrument connections, control rod drive penetrations and control rod drive pressure boundary welds	Volumetric	Cumulative 25% coverage at end of 10 year interval	Not applicable in accordance with ASME B&PV Code Section I, IS-121
E-2	Vessel penetrations including instrument connections and control rod drive penetration and pressure boundary welds	Visual	Cumulative 25% coverage at end of 10 year inspection interval	Visual examination during hydrostatic test as specified by ASME B&PV Code Section XI, IS-520

TABLE 4.6.1 (cont)

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examinations <sup>(1)</sup>
F	Primary nozzles to safe-end welds	Visual and surface and volumetric	Cumulative 100% coverage at end of 10 year interval	Safe-ended nozzles: Recirc. Outlet (2) - 1/5 years Recirc. Inlet (10) - 1/year Isolation Condenser Outlet (2) - 1/5 years Core Spray Inlet (2) - 1/5 years Control Rod Drive Return (1) - 1/10 years Standby Liquid Control (1) - 1/10 years Head Instrumentation (2) - 1/5 years Head Spray Inlet (1) - 1/10 years
G-1	Closure studs and nuts  Ligaments between threaded stud holes	Volumetric and visual or surface  Volumetric	Cumulative 100% coverage at end of 10 year interval  "	10% of ligaments each year. Examination of bushings, threads and ligaments in base material of flanges may be performed from the face of the flange and are required to be examined only when the connection is disassembled.

TABLE 4.6.1 (cont)

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examinations <sup>(1)</sup>
G-1(cont)	Closure washers, bushings	Visual	Cumulative 100% coverage at end of 10 year interval	Not applicable with present plant design
	Pressure-retaining bolting $\geq 2$ " in. diameter	Visual and Volumetric	"	10% of recirculating pump bolts each year
G-2	Pressure-retaining bolting $< 2$ " in diameter which is not excluded from examination in accordance with ASME B&PV Section XI, IS-121	Visual	"	Bolting will be examined when bolting is removed or when the bolted connection is broken or disassembled. For bolting which is not removed, or the bolted connection is not broken, the inspection will consist of a visual exam to detect signs of distress or evidence of leaking
H	Integrally welded vessel supports	Volumetric	During 10 year interval	10% (approximately 8 ft.) of lineal ft. or vessel support skirt welding in 10th year
I	Closure head cladding	Visual and surface or volumetric	During 10 year interval	During the 10 year interval, at least 6 patches (each 36 sq. in.) evenly distributed in the closure head.

TABLE 4.6.1 (cont)

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examinations <sup>(1)</sup>																																	
I (cont)	Vessel Cladding	Visual	During 10 year interval	6 patches (each 36 sq. in.) evenly distributed in the accessible sections of the vessel shell shall be examined.																																	
J(2)	Circumferential and longitudinal pipe welds	Visual and Volumetric	<p>Cumulative 25% of all weld joints (selectively distributed among the higher stress joints in entire system) every 10 years</p> <p>Group I and Group II welds (see note (2) for breakdown) on main feed and main steam lines shall be inspected in 10 years during the first interval. At least 25% of the welds shall be inspected at approximately each 2½ year interval. Group I welds shall be inspected during each 10 year interval thereafter.</p>	<table border="1"> <thead> <tr> <th data-bbox="1459 690 1564 722">System</th> <th data-bbox="1816 657 1921 722">Pipe Sizes</th> <th data-bbox="1942 657 2026 722">Total Welds</th> </tr> </thead> <tbody> <tr> <td data-bbox="1459 755 1774 787">Shutdown cooling</td> <td data-bbox="1837 755 1900 820">14" 16"</td> <td data-bbox="1963 755 2005 787">68</td> </tr> <tr> <td data-bbox="1459 820 1732 852">Isolation Con.</td> <td data-bbox="1837 820 1900 885">12" 14"</td> <td data-bbox="1963 820 2005 852">39</td> </tr> <tr> <td data-bbox="1459 885 1711 950">Reactor Water Cleanup</td> <td data-bbox="1837 885 1900 950">8" 10"</td> <td data-bbox="1963 885 2005 917">36</td> </tr> <tr> <td data-bbox="1459 950 1753 982">CRD Hyd. System</td> <td data-bbox="1837 950 1900 982">4"</td> <td data-bbox="1963 950 2005 982">12</td> </tr> <tr> <td data-bbox="1459 982 1774 1047">L.P.Coolant Inj.</td> <td data-bbox="1837 982 1900 1047">14" 16" 18"</td> <td data-bbox="1963 982 2005 1015">43</td> </tr> <tr> <td data-bbox="1459 1079 1795 1112">Core Spray Piping</td> <td data-bbox="1837 1079 1900 1112">10"</td> <td data-bbox="1963 1079 2005 1112">74</td> </tr> <tr> <td data-bbox="1459 1112 1774 1144">H.P.Coolant Inj.</td> <td data-bbox="1837 1112 1900 1144">10"</td> <td data-bbox="1963 1112 2005 1144">22</td> </tr> <tr> <td data-bbox="1459 1144 1669 1177">Feed Piping</td> <td data-bbox="1837 1144 1900 1209">12" 18"</td> <td data-bbox="1963 1144 2005 1177">61</td> </tr> <tr> <td data-bbox="1459 1209 1711 1242">Recirculation</td> <td data-bbox="1837 1209 1900 1339">4" 12" 22" 28"</td> <td data-bbox="1963 1209 2005 1242">123</td> </tr> <tr> <td data-bbox="1459 1339 1648 1372">Main Steam</td> <td data-bbox="1837 1339 1900 1372">8"</td> <td data-bbox="1963 1339 2005 1372">129</td> </tr> </tbody> </table>	System	Pipe Sizes	Total Welds	Shutdown cooling	14" 16"	68	Isolation Con.	12" 14"	39	Reactor Water Cleanup	8" 10"	36	CRD Hyd. System	4"	12	L.P.Coolant Inj.	14" 16" 18"	43	Core Spray Piping	10"	74	H.P.Coolant Inj.	10"	22	Feed Piping	12" 18"	61	Recirculation	4" 12" 22" 28"	123	Main Steam	8"	129
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TABLE 4.6.1 (cont)

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examinations (1)
K-1	Integrally-welded external support attachments for piping, valve and pumps	Visual and Volumetric	100% cumulative in first 10 years. 25% cumulative in each following 10-year inspection interval..	Welds to the pressure-containing boundary, the base metal beneath the weld zone and along the support attachment member for a distance of two base metal thicknesses.
K-2	Support members and structures for piping, valves and pumps whose structural integrity is relied upon to withstand design loads and seismic-induced displacements.	Visual	100% cumulative during each 10-year inspection interval.	Support settings of constant and variable spring type hangers, snubbers and shock absorbers shall be inspected to verify proper distribution of design loads among the associated support components.
L-1	Pump casing welds	Visual and Volumetric	One pump of each type during 10 year interval.	Not applicable with present plant design.
L-2	Pump casings	Visual	One pump of each type during 10 year interval.	One recirculating pump in 10 years.

TABLE 4.6.1 (cont)

Category	Component Parts to be Examined	Exam Method	Frequency of Examination	Extent of Examinations <sup>(1)</sup>
M-1	Welds in valve bodies 3" and above	Visual and Volumetric	One valve of each type during 10 year interval	Not applicable with present plant design
M-2	Valve bodies 3" and above.	Visual	One valve of each type during 10 year interval	One disassembled valve (with or without welds and 3" over in normal size) in each category and type shall be subject to visual examination. Individual examinations shall cover 100% of the pressure boundary welds and may be performed at or near the end of the 10 year interval.
N	Interior surfaces and internals & integrally welded internal supports of the reactor vessel, including core spray spargers, core spray nozzles, and upper portions of jet pumps.	Visual	During first refueling outage and during subsequent refueling outages at approximately 3-year intervals.	Interior surfaces and internal components of the reactor vessel, including the space at the bottom head, and internal attachments which are welded to the vessel, made accessible by the removal of components during normal refueling operations. All internal attachments, whose failure may adversely affect core integrity, shall be examined.

Notes to Table 4.6.1:

(1) Examinations which reveal unacceptable structural defects in a category shall be extended to include an additional number (or areas) of system components or piping in the same category approximately equal to that initially examined. In the event further unacceptable structural defects are revealed, all remaining system components or piping in the category shall be examined to the extent specified in that examination category.

\*

(2)

Category J Weld Breakdown:

Main Steam Line

Group I Welds

<u>Line</u>	<u>Weld Identification</u>
3001A	K-6
3001B	K-5A
3001C	K-6
3001D	K-6

Feedwater Line

Group I Welds

<u>Line</u>	<u>Weld Identification</u>
3204A	K-3
3204B	K-3

Group II Welds

<u>Line</u>	<u>Weld Identification</u>
3001A	4, 5, K-10, K-11, K-12
3001B	4, 5, K-10, K-11, K-12
3001C	4, 5, K-6A, K-10, K-11, K-12
3001D	4, 5, K-10, K-11, K-12

Group II Welds

<u>Line</u>	<u>Weld Identification</u>
3204A	K-4, K-5, 9, K-2
3204B	K-4, K-5, K-6, 11, K-2
3204C	10
3204F	12, 1

Note: The following component parts shall be examined at the first refueling outage occurring after five years of operation.

- a. All of the furnace sensitized wrought stainless steel safe ends and their welds.
- b. The sensitized heat affected zones adjacent to the welds and the welds in the stainless steel reactor coolant piping will be PT and UT examined in accordance with a program which will provide for examination of 10% of the welds.
- c. The furnace sensitized stainless steel internal brackets and their attaching welds inside the reactor pressure vessel, including the jet pump riser supports, shall be visually inspected.

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UNITED STATES  
ATOMIC ENERGY COMMISSION

WASHINGTON, D.C. 20545

JAN 8 1973

Files (Docket No. 50-237) *Ma for*  
THRU: D. L. Ziemann, Chief, ORB #2, L

SAFETY EVALUATION FOR TECHNICAL SPECIFICATION CHANGE NO. 19 FOR DRESDEN  
UNIT 2 (COMMONWEALTH EDISON COMPANY)

Commonwealth Edison proposed, by letter dated January 31, 1972, that changes be made to the inservice inspection program for Dresden Unit 2 to bring this program into conformance with Section XI of the ASME Boiler and Pressure Vessel Code. Technical Review evaluated these changes and reported their conclusions and recommendations in a memo from R. R. Maccary to R. L. Tedesco dated March 17, 1972 (copy enclosed). The Regulatory staff had previously concluded that the margin of safety in the design of the safe ends is adequate and recommended that an augmented inservice inspection program for sensitized stainless steel components be continued through the second refueling outage. Commonwealth Edison performed the augmented inspection recommended in that memo during the second refueling outage in the spring of 1972 and reported the results by letter dated July 31, 1972. According to Commonwealth Edison's letter, the inspection revealed "only very minor, acceptable surface defects, which were removed with a light polishing" and a defect in a jet pump instrumentation nozzle which, according to CE, appeared to be a surface defect. Technical Review reviewed the results of this inspection and has concluded that the inservice inspection program proposed by Commonwealth Edison is acceptable with the exception that a special inservice inspection of the furnace sensitized stainless steel components should be performed by CE during the first refueling outage after five years of operation and the results should be evaluated by the AEC.

On the basis of the Technical Review evaluation, we conclude that the proposed changes to the Technical Specifications as modified to provide a special inservice inspection of the furnace sensitized stainless steel components at the first refueling outage after five years of operation (about January 1975) do not present significant hazards

*207.1*

JAN 8 1973

considerations not described or implicit in the Dresden Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered.



Richard D. Silver  
Operating Reactors Branch #2  
Directorate of Licensing

Enclosure:

Memo, Maccary to Tedesco,  
dtd 3/17/72

cc w/enclosure:

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Local PDR

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RTedesco, L:CS (2)  
DJSkovholt, L:OR  
RO (3)  
TJCarter, L:OR  
DLZiemann, L:ORB #2  
RDSilver, L:ORB #2  
MJinks, DRA (2)  
RMDiggs, L:ORB #2  
SPawlicki, L:ME  
RMaccary, L:RS

2007

JAN 8 1973

Files (Docket No. 50-237)

THRU: D. L. Ziemann, Chief, ORB #2, L 151 F. D. Anderson for

**SAFETY EVALUATION FOR TECHNICAL SPECIFICATION CHANGE NO. 19 FOR DRESDEN UNIT 2 (COMMONWEALTH EDISON COMPANY)**

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On the basis of the Technical Review evaluation, we conclude that the proposed changes to the Technical Specifications as modified to provide a special inservice inspection of the furnace sensitized stainless steel components at the first refueling outage after five years of operation (about January 1975) do not present significant hazards

D/L  
7/2/72

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*Richard D Silver*

Richard D. Silver  
Operating Reactors Branch #2  
Directorate of Licensing

Enclosure:  
Memo, Maccary to Tedesco,  
dtd 3/17/72

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AEC PDR  
Local PDR

cc w/o enclosure:  
RTedesco, L:CS (2)  
DJSkovholt, L:OR  
RO (3)  
TJCarter, L:OR  
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DATE ▶	1/4/73	1/4/73	1/5/73	1/5/73		