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APR 1 6 1975

Commonwealth Edison Company ATTN: Mr. J. S. Abel

Nuclear Licensing Administrator -

Boiling Water Reactors

Post Office Box 767 Chicago, Illinois 60690

Gentlemen:

Docket No. 50-237

OI&E (3) NDub e BJones (4) JMMcGough JSaltzman, OMI

RMDiggs RDSilver DLZiemann

SKari BScharf (15) TJCarter PCollins | SVarga CHebron

The Commission has issued the enclosed Amendment No. 6 to Facility License No. DPR-19 for Dresden Unit 2. This amendment includes Change No. 32 to the Technical Specifications and is in response to your request dated April 4, 1975.

The amendment allows the core spray and LPCI systems to remain inoperable under specified conditions when the reactor is placed in the refuel mode from a cold shutdown condition.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Original signed by Dennis L. Ziemann

Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

Enclosures:

- 1. Amendment No. 6 w/Change No. 32
- Safety Evaluation
- 3. Federal Register Notice

cc w/enclosures: see next page

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cc w/enclosures:
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Counselors at Law
One First National Plaza
Chicago, Illinois 60670

Anthony Z. Roisman, Esquire Berlin, Roisman and Kessler 1712 N Street, N. W. Washington, D. C. 20036

Morris Public Library 604 Liberty Street Morris, Illinois 60451

Chairman, Board of Supervisors of Grundy County Grundy County Courthouse Morris, Illinois 60450

cc w/enclosures and filing dtd.
 4/4/75:

Mr. Leroy Stratton
Bureau of Radiological Health
Illinois Department of Public Health
Springfield, Illinois 62706

Mr. Gary Williams Federal Activities Branch Environmental Protection Agency 230 South Dearborn Street Chicago, Illinois 60604

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6 License No. DPR-19

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated April 4, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations, and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-19 is hereby amended to read as follows:

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"3.B Technical Specifications

The Technical Specifications contained in Appendix A, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 33."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by Dennis L. Ziemann

Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

Attachment: Change No. 32 to the Technical Specifications

Date of Issuance: APR 1 6 1975

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ATTACHMENT TO LICENSE AMENDMENT NO. 6

CHANGE NO. 32 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Replace existing page 75 of the Technical Specifications with the attached revised page 75. Changed area on the revised page is reflected by a marginal line:

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- 4.5 SURVEILLANCE REQUIREMENTS
- 4. From and after the date that one of the LPCI pumps is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding 30 days unless such pump is sooner made operable, provided that during such 30 days the remaining active components of the LPCI and containment cooling subsystem and all active components of both core spray subsystems and the diesel generators required for operation of such components if no external source of power were available shall be operable.
- 5. From and after the date that the LPCI subsystem is made or found to be operable for any reason, reactor operation is permissible only during the succeeding 7 days unless it is sooner made operable, provided that during such 7 days all active components of both core spray subsystems, the containment cooling subsystems (including 2 LPCI pumps) and the diesel generators required for operation of such components if no external source of power were available shall be operable.
- 6. Containment cooling spray loops are required to be operable when the reactor water temperature is greater than 212°F except that a maximum of one drywell spray loop may be inoperable for 30 days when the reactor water temperature is greater than 212°F.
- 7. If the requirements of 3.5.A cannot be met, an orderly shutdown of the reactor shall be initiated and the reactor shall be in Cold Shutdown within 24 hours. Subsequently, the reactor may be placed in Refuel, for post maintenance testing of control rod drives only provided no work is being performed which has the potential to drain the reactor vessel.

- 4. When it is determined that one of the LPCI pumps is inoperable, the remaining active components of the LPCI and containment cooling subsystem, both core spray subsystems and the diesel generators required for operation of such components if no external source of power were available shall be demonstrated to be operable immediately and the operable LPCI pumps daily thereafter.
- 5. When it is determined that the LPCI subsystem is inoperable, both core spray subsystems, the containment cooling subsystem, and the diesel generators required for operation of such components if no external source of power were available shall be demonstrated to be operable immediately and daily thereafter.
- 6. During each five-year period, an air test shall be performed on the drywell spray headers and nozzles.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 6 TO LICENSE NO. DPR-19

(CHANGE NO. 32 TO THE TECHNICAL SPECIFICATIONS)

COMMONWEALTH EDISON COMPANY

DRESDEN UNIT 2

DOCKET NO. 50-237

INTRODUCTION

By letter dated April 4, 1975, Commonwealth Edison Company (CE) requested a change to the Technical Specifications appended to Facility Operating License No. DPR-19 for Dresden Nuclear Power Station Unit 2. The proposal would revise Section 3.5.A.7. regarding low pressure, emergency core cooling system limiting conditions of operation.

DISCUSSION

Section 3.5.A. of the Technical Specifications requires that, except under certain specified conditions, the core spray and low pressure coolant injection (LPCI) systems must be operable whenever irradiated fuel is in the reactor. This section states that if the specified requirements cannot be complied with, an orderly shutdown shall be initiated and the reactor shall be in the cold shutdown condition within 24 hours. Once in the cold shutdown condition, all low pressure core and containment cooling subsystems (which includes LPCI and core spray) may be inoperable provided no work is being done which has the potential for draining the reactor vessel. The proposition would allow the core spray and LPCI systems to remain inoperable when the reactor is placed in the refuel mode from the cold shutdown condition provided, again, that no work is being performed which has the potential for draining the reactor vessel.

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The purpose of this change is to allow CE to perform post maintenance testing of control rod drives while simultaneously performing maintenance work on the low pressure, emergency core cooling system. The performance of these tests in the cold shutdown condition is prevented by interlocks. These tests can be performed in the refuel mode. Permitting these simultaneous functions does not result in a decrease in the safety margin because performance of the control rod tests in the refueling mode does not increase the probability of draining the reactor vessel. Furthermore, interlocks operable in the refuel mode prevent conditions which would cause pressurization of the reactor vessel or reactor containment. For post maintenance testing of control rod drives, the LPCI and core spray perform no special safety function different from its standby function in the cold shutdown mode. Therefore, there is no greater need for the LPCI and core spray in the refuel mode than in cold shutdown condition for the proposed operations, such as control rod testing, which do not affect the potential for draining the core. However, since CE's analysis does not include the entire range of activity that could be carried out in the refuel mode, the proposed technical specification will be limited to allow only post maintenance testing of control rod drives when the LPCI and core spray systems are inoperable.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: APR 1 6 1975

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Form AEC-318 (Rev. 9-	53) AECM 0240	☆:	J. S. GOVERNMENT PRIN	TING OFFICE 1021		

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-237

COMMONWEALTH EDISON COMPANY

NOTICE OF ISSUANCE OF FACILITY LICENSE AMENDMENT

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. DPR-19 to the Commonwealth Edison Company (the licensee) which revised Technical Specifications for operation of the Dresden Nuclear Power Station Unit 2 located in Grundy County, Illinois. The amendment is effective as of its date of issuance.

The amendment allows the core spray and low pressure coolant injection (LPCI) systems to remain inoperable under specified conditions when the reactor is placed in the Refuel mode from a cold shutdown condition, in accordance with the licensee's application dated April 4, 1975.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

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For further details with respect to this action, see (1) the application for amendment dated April 4, 1975, (2) Amendment No. 6 to License No. DPR-19 with Change No. 32, and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Morris Public Library at 604 Liberty Street in Morris, Illinois, 60451. A single copy of items (2) and (3) may be obtained upon request addressed to the Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 16th day of April 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by Dennis L. Ziemann

Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

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