

Docket No. 50-237

Mr. J. S. Abel  
Director of Nuclear Licensing  
Commonwealth Edison Company  
Post Office Box 767  
Chicago, Illinois 60690

Dear Mr. Abel:

The Commission has issued the enclosed Amendment No. 60 to Provisional Operating License No. DPR-19 for Dresden Station Unit No. 2. This amendment is in response to your application dated March 27, 1981.

The amendment revises the Technical Specifications to reflect removal of 35 hydraulic snubbers (shock suppressors) located within the containment. These snubbers have been replaced by 26 mechanical snubbers. These snubbers are located inside the primary containment and are inaccessible during reactor operation.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

Enclosures:

- 1. Amendment No. 60 to License No. DPR-19
- 2. Safety Evaluation
- 3. Notice of Issuance

cc w/enclosures:  
See next page



FR NOTICE  
AMENDMENT  
ONLY

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OFFICE	DL:ORB #5/PM	DL:ORB #5/LA	OELD	DL:ORB #5/C	DL:AD/SA		
SURNAME	PO'Connor:cd	HSmith	K.R.M.P.	DCrutchfield	GLainas		
DATE	4/30/81	5/6/81	4/30/81	4/30/81	4/30/81		



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

May 6, 1981

Docket No. 50-237  
LS05-81-05-005

Mr. J. S. Abel  
Director of Nuclear Licensing  
Commonwealth Edison Company  
Post Office Box 767  
Chicago, Illinois 60690

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Dennis M. Crutchfield, Chief  
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See next page

May 6, 1981

cc

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN STATION, UNIT NO. 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 60  
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 27, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing

Date of Issuance: May 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 60

PROVISIONAL OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Replace page 91b, 91d, and 91e with the enclosed page 91b and add the following new pages: 91 d-1, 91 d-2, 91 e-1 and 91 e-2

3.6 LIMITING CONDITION FOR OPERATION

I. Shock Suppressors (Snubbers)

1. During all modes of operation except cold shutdown and refuel, all safety-related snubbers listed in Tables 3.6.1a and b shall be operable except as noted in Specification 3.6.I.2 through 3.6.I.4.
2. From and after the time that a snubber is determined to be inoperable, continued reactor operation is permissible only during the succeeding 72 hours unless the snubber is sooner made operable or replaced. Torus ring header snubbers may be inoperable in groups of up to three (3) pairs until September 1, 1980, to facilitate the installation of Mark I Torus Support Modifications.
3. If the requirements of 3.6.I.1 and 3.6.I.2 can not be met, an orderly shutdown shall be initiated and the reactor shall be in cold shutdown or refuel condition within 36 hours.
4. If a snubber is determined to be inoperable while the reactor is in the cold shutdown or refuel mode, the snubber shall be made operable or replaced prior to reactor startup. This requirement does not apply to torus ring header snubbers for the period identified in paragraph 3.6.I.2 above.
5. Snubbers may be added to safety related systems without prior license amendment to Tables 3.6.1a and b provided that a revision to Tables 3.6.1a and b is included with the next license amendment request.

4.6 SURVEILLANCE REQUIREMENT

I. Shock Suppressors (Snubbers)

The following surveillance requirements apply to all hydraulic snubbers listed in Table 3.6.1a.

1. All hydraulic snubbers whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected. This inspection shall include, but not necessarily be limited to inspection of the hydraulic fluid reservoir, fluid connections, and linkage connection to the piping and anchor to verify snubber operability in accordance with the following schedule:

<u>No. of Snubbers Found Inoperable During Inspection Interval</u>	<u>Next Required Inspection Interval</u>
0	18 months $\pm$ 25%
1	12 months $\pm$ 25%
2	6 months $\pm$ 25%
3, 4	124 days $\pm$ 25%
5, 6, 7	62 days $\pm$ 25%
$\geq$ 8	31 days $\pm$ 25%

The required inspection interval shall not be lengthened more than one step at a time.

TABLE 3.6.1a

SAFETY RELATED HYDRAULIC SNUBBERS

SNUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
2	Torus Ring Header 1501-24"	483'	83°			X
3	Torus Ring Header 1501-24"	483'	74°			X
4	Torus Ring Header 1501-24"	483'	38°			X
5	Torus Ring Header 1501-24"	483'	29°			X
7	Torus Ring Header 1501-24"	483'	331°			X
8	Torus Ring Header 1501-24"	483'	322°			X
9	Torus Ring Header 1501-24"	483'	286°			X
10	Torus Ring Header 1501-24"	483'	277°			X
12	Torus Ring Header 1501-24"	483'	218°			X
13	Torus Ring Header 1501-24"	483'	209°			X
15	Torus Ring Header 1501-24"	483'	151°			X
16	Torus Ring Header 1501-24"	483'	142°			X

\*Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.

TABLE 3.6.1a

**SAFETY RELATED HYDRAULIC SNUBBERS**

SHUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
	<b>Isolation Condenser Pipeway Room:</b>					
1	Isolation Condenser Line 1303-12"	558'	180°	X		X
2	Isolation Condenser Line 1303-12"	568'	180°	X		X
3	Isolation Condenser Line 1302-14"	580'	195°	X		X

Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.

TABLE 3.6.1b

## SAFETY RELATED MECHANICAL SNUBBERS

SNUBBER NO.	LOCATION	ELEVATION	AZINUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
1	Drywell Recirc. Motor 2B-202	524'	328°	X	X	
2	Drywell Recirc. Motor 2B-202	524'	302°	X	X	
3	Drywell Recirc. Motor 2B-202	524'	315°	X	X	
4	Drywell Recirc. Motor 2A-202	524'	148°	X	X	
5	Drywell Recirc. Motor 2A-202	524'	122°	X	X	
6	Drywell Recirc. Motor 2A-202	524'	135°	X	X	
7	Drywell Recirc. Pump 2B-202	512'	326°	X	X	
8	Drywell Recirc. Pump 2B-202	512'	304°	X	X	
9	Drywell Recirc. Pump 2B-202	517'	315°	X	X	
10	Drywell Recirc. Pump 2A-202	512'	124°	X	X	
11	Drywell Recirc. Pump 2A-202	512'	146°	X	X	
12	Drywell Recirc. Pump 2A-202	507'	135°	X	X	
13-16	Removed					
17	Drywell Recirc Header 201B-22"	533'6"	195°	X	X	
18-20	Removed					
21	Drywell Recirc Header 201A-22"	533'6"	22°	X	X	
22-23	Removed					
24	Drywell Feedwater Line 3204D-12"	538'	108°	X	X	
25-29	Removed					
30	Drywell Core Spray Line 1403-10"	575'	336°	X	X	
31	Drywell Core Spray Line 1404-10"	562'	231°	X	X	
32	Drywell Target Rock Valve 203-3A	542'6"	16°	X	X	
33	Drywell Target Rock Valve 203-3A	542'4"	31°	X	X	
34	Drywell Target Rock Valve 203-3A	540'0"	19°	X	X	
35	Drywell Target Rock Valve 203-3A	540'3"	34°	X	X	
36	Drywell Recirc. Line 201B-28"	518'	270°	X	X	

\*Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.

**TABLE 3.6.1b**

**SAFETY RELATED MECHANICAL SNUBBERS**

SNUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
37	Drywell Recirc. Line 201A-28"	518'	90°	X	X	
38	Drywell Shutdown Cooling Line 1001A-16"	523'	0°	X	X	
39	Drywell Rx Water Cleanup Line 1201-8"	533'	316°	X	X	
40	Drywell Rx Water Cleanup Line 1201-8"	533'	301°	X	X	

\*Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 60 TO PROVISIONAL OPERATING LICENSE NO. DPR-19  
COMMONWEALTH EDISON COMPANY  
DRESDEN 2  
DOCKET NO. 50-237

1.0 INTRODUCTION

By letter dated March 27, 1981, Commonwealth Edison Company (CECo) informed NRC that they had reanalyzed the seismic design of safety related piping, at Dresden Unit 2, in response to IE Bulletin 79-14. The reanalysis identified piping locations where existing safety related hydraulic snubbers had to be removed to prevent the occurrence of stresses in excess of the original FSAR requirements. In addition, CECo also replaced the remaining hydraulic snubbers located in inaccessible areas with mechanical snubbers.

Because the hydraulic snubbers were identified in the Technical Specifications, CECo has proposed an amendment to the Technical Specifications which would reflect the removal of the hydraulic snubbers and the replacement of some of them by mechanical snubbers.

2.0 DISCUSSION AND EVALUATION

The CECo analyses completed in response to IE Bulletin 79-14, using as-built piping configurations, identified nine (9) piping locations where the stress was calculated to be in excess of the allowable stresses. CECo corrected these overstress conditions by relocating and removing a number of hydraulic shock suppressors that were contributing to the stresses on the piping.

We have reviewed CECo's new snubber configurations and concur with their determination that they have acceptably eliminated the overstressed points by removing and relocating snubbers. The reconfigured piping stresses no longer violate the allowable stress limits and are therefore acceptable to the staff.

In addition, CECo has replaced all remaining inaccessible hydraulic snubbers on safety related piping with mechanical snubbers. These mechanical snubbers will provide an equivalent level of safety and restraint as was previously provided by the hydraulic snubbers and their use is acceptable to the NRC staff.

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Based on the above, we conclude that the Technical Specifications should be changed to delete the hydraulic snubbers which have been replaced by mechanical snubbers and to add the mechanical snubbers to the list of snubbers required for Dresden 2 operation.

### 3.0 ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 6, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-237COMMONWEALTH EDISON COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO  
PROVISIONAL OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 60 to Provisional Operating License No. PPR-19 issued to Commonwealth Edison Company, which revised the Technical Specifications for operation of the Dresden Nuclear Power Station, Unit No. 2, located in Grundy County, Illinois. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications to allow removal of thirty-five hydraulic shock suppressors (snubbers) located within the containment and their replacement by twenty-six mechanical snubbers.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

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For further details with respect to this action, see (1) the application for amendment dated March 27, 1981, (2) Amendment No. 60 to License No. DPR-19 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Morris Public Library, 604 Liberty Street, Morris, Illinois. A copy of items (2) (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 6th day of May, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Dennis M. Crutchfield, Chief  
Operating Reactors Branch #5  
Division of Licensing