

Docket No. 50-237

Mr. Cordell Reed
Assistant Vice President
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

FEB 27 1979

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Dear Mr. Reed:

The Commission has issued the enclosed Amendment No. 40 to Provisional Operating License No. DPR-19 for Dresden Nuclear Power Station, Unit No. 2. The amendment is in response to your request of February 4, 1979. You were previously notified of this license amendment by telephone and letter on February 4, 1979.

The amendment revises the Technical Specifications to permit operation of the reactor for a period of 24 hours from 1:00 p.m. on February 4, 1979, with a negative pressure of 0.2 inches of water maintained in areas of the Reactor Building below the refueling floor.

Copies of our related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

1. Amendment No. 40 to DPR-2
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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DATE	2/13/79	2/12/79	2/27/79	2/27/79	2/16/79	2/21/79

FEB 27 1979

cc w/enclosures:

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Morris Public Library
604 Liberty Street
Morris, Illinois 60451

Chairman
Board of Supervisors of
Grundy County
Grundy County Courthouse
Morris, Illinois 60450

*Department of Public Health
ATTN: Chief, Division of
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Director, Technical Assessment
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Office of Radiation Programs
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*(w/cy. of incoming dtd. 2/4/79)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION UNIT NO. 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 40
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated February 4, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 40, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

~~Approved by~~
~~Special Agent~~

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: FEB 27 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 40

PROVISIONAL OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Change the Technical Specifications contained in Appendix A by removing Page 120 and inserting the enclosed Page 120. The revised page contains the captioned amendment number and a vertical line indicating the area of change.

3.7 LIMITING CONDITION FOR OPERATION

operation except when all of the following conditions are met.

- a. The reactor is subcritical and Specification 3.3.A is met.
- b. The reactor water temperature is below 212°F and the reactor coolant system is vented.
- c. No activity is being performed which can reduce the shutdown margin below that specified in Specification 3.3.A.
- d. The fuel cask or irradiated fuel is not being moved in the reactor building.

2. The doors of the core spray and LPCI pump compartments shall be closed at all times

4.7 SURVEILLANCE REQUIREMENT

- a. A preoperational secondary containment capability test shall be conducted after isolating the reactor building and placing either standby gas treatment system filter train in operation. Such tests shall demonstrate the capability to maintain a 1/4 inch of water vacuum under calm wind (<5 mph) conditions with a filter train flow rate of not more than 4000 cfm.
- b. Additional tests shall be performed during the first operating cycle under an adequate number of different environmental wind conditions to enable valid extrapolation of the test results.
- c. Secondary containment capability to maintain a 1/4 inch of water vacuum under calm wind (<5 mph) conditions with a filter train flow rate of not more than 4000 cfm, shall be demonstrated at each refueling outage prior to refueling.
- d. For the 24 hour period commencing on February 4, 1979, at 1:00 p.m., reactor operation is permitted provided that a negative pressure of 0.2 inches of water is maintained in the Unit 2 reactor building areas below the refueling floor.

2. Whenever the LPCI and core spray sub-systems are required to be operable, the



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 40 TO PROVISIONAL OPERATING LICENSE NO. DPR-19

COMMONWEALTH EDISON COMPANY

DRESDEN UNIT NO. 2

DOCKET NO. 50-237

Introduction

By letter received on February 4, 1979, Commonwealth Edison Company (CECo) proposed an amendment to the Dresden Unit No. 2 Technical Specifications. The proposed change requested authorization to operate Dresden Unit No. 2 after a negative pressure of 0.2 inches of water was established in the Reactor Building areas below the refueling floor to demonstrate secondary containment integrity. This negative pressure was maintained in the Reactor Building area for 24 hours following initiation at 1:00 p.m. on February 4, 1979.

Evaluation

The existing Dresden Unit No. 2 Specification 4.7.C.1.c requires that the secondary containment be capable of maintaining 0.25 inches of water vacuum at each refueling outage prior to refueling.

On February 2, 1979, several blowout panels on the Reactor Building for Dresden Unit Nos. 2 and 3 became detached as a result of a ventilation system malfunction that pressurized the Reactor Building. The over-pressurization caused the blowout panels to fail, which resulted in a 22' x 40' opening in the Reactor Building superstructure at the refueling floor level.

We reviewed CECo's request and obtained additional related information from licensee representatives by a telephone conversation on February 4, 1979. The following factors related to the proposed change were considered in our review:

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1. The Reactor Building air space has been isolated from the refueling floor air space by temporary barriers. With these barriers in place, CECO is able to maintain a negative pressure of 0.2 inches of water in the Reactor-Building exclusive of the refueling floor by operating the standby gas treatment system.
2. The secondary containment performance requirement (0.25 inches of water negative pressure) was selected to prevent leakage from the Reactor Building caused by localized areas of wind-induced negative pressure. These areas are located in the upper corners of the lee side of the Reactor Building refueling floor during high winds. These areas are now isolated from the Reactor Building volume by temporary barriers. We have determined that a negative pressure of 0.2 inches of water in the balance of the Reactor Building (excluding the refueling floor) is adequate to prevent exfiltration from the building.
3. The accident analysis for Dresden assumes that accidents occur under stagnant meteorological conditions with a low wind velocity. The current wind conditions present at Dresden Station involve 10 to 15 mile per hour winds with turbulent flow.
4. Most of the primary containment penetrations are located in the Reactor Building below the refueling floor and do not communicate directly with the refueling floor air space. Therefore, in the event of a LOCA, the leakage through the containment penetrations (a) will be contained within the portion of the Reactor Building secondary containment that maintains its integrity and (b) will be processed through the standby gas treatment system.
5. CECO has committed to stop all fuel handling and cask handling activities on the refueling floor until secondary containment integrity is restored by repairing the breach in the Reactor Building wall above the refueling floor level.
6. CECO has indicated that they are unable to purchase sufficient replacement power to prevent system voltage reduction and possible subsequent load shedding in the area.

We have considered the preceding factors and have concluded that CECO's procedure to maintain Reactor Building integrity (as previously described) provides an equivalent level of protection against the exfiltration of airborne radioactive material should a LOCA occur within the 24 hours that this Technical Specification is in effect.

We have further concluded that the proposed operation is in the public interest because of possible load shedding in the area that might be required if Dresden Unit No. 2 is not available during this 24-hour period.

The NRC staff has determined that an additional measure of protection will be afforded the public by the continuous operation of the standby gas treatment system during the period that this specification is in effect. We have discussed this action with CECO and they have agreed to operate the system continuously. We have modified the proposed specification to require that the negative pressure be maintained continuously.

In evaluating the above considerations, we have concluded that the proposed change, as modified by the NRC staff, is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: FEB 27 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-237COMMONWEALTH EDISON COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 40 to Provisional Operating License No. DPR-19, issued to the Commonwealth Edison Company (the licensee), which revised the Technical Specifications for operation of Unit No. 2 of Dresden Nuclear Power Station (the facility) located in Grundy County, Illinois. The license amendment is effective as of its date of issuance.

The amendment revises the Technical Specifications to permit operation of the reactor for a period of 24 hours from 1:00 p.m. on February 4, 1979, with a negative pressure of 0.2 inches of water in areas of the Reactor Building below the refueling floor.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 4, 1979, (2) Amendment No. ⁴⁰ to License No. DPR-19, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555 and at the Morris Public Library, 604 Liberty Street, Morris, Illinois 60451. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this FEB 27 1979

FOR THE NUCLEAR REGULATORY COMMISSION

(Signature)
Special Agent in Charge

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors