

Distribution:

NOV 12 1980

Docket Nos. 50-237
50-254
50-265

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- NRC PDR
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- R. Purple
- T. Novak
- R. Tedesco
- G. Lainas
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- NSIC
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- Chairman, AEOB

Mr. J. S. Abel
Director of Nuclear Licensing
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

Dear Mr. Abel:

The Commission has issued the enclosed Amendment No. to Provisional Operating License No. DPR-19 for Dresden Station Unit 2 and Amendment Nos. and to Facility Operating License Nos. DPR-29 and DPR-30 for Quad Cities Station Units 1 and 2, respectively. The amendments are in response to your letter of March 19, 1980.

The amendments establish MAPLHGR limits for standard 8x8 (8D250 and 8D262) fuel types for exposure values beyond those currently given in the Technical Specifications.

Your letter dated December 20, 1979 states that it is not your intent to exceed 40,000 MWD/t (short ton) peak pellet exposure for any fuel type for any operating cycle. This constraint will continue to apply and will occur before planar average exposures of 40,000 MWD/t are approached.

You will note that Tables S-3 and S-4 of 10 CFR 51.20 are based on an average fuel burnup of 33,000 MWD/metric ton for irradiated fuel from the reactor. Therefore, even though these amendments establish MAPLHGR limits for fuel burnup out to 40,000 MWD/t, the average level of irradiation of the irradiated fuel from the reactor should not exceed 33,000 MWD/metric ton.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed by
T. A. Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. to DPR-19
2. Amendment No. to DPR-29
3. Amendment No. to DPR-30
4. Safety Evaluation
5. Notice

8012080089

Handwritten: D. Eisenhut
DMC-Wichfield
10/21/80

OELD
Handwritten: Goddard
10/31/80

OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL	DL:OR
SURNAME	T Alexion	R Bevan	S Norris	TA Ippolito	PO' Conner	TM Novak
DATE	10/16/80	10/20/80	10/24/80	10/24/80	10/20/80	10/24/80

Mr. J. S. Abel

- 2 -

November 12, 1980

CC:

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Davenport, Iowa 52801

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Plant Superintendent
Quad Cities Nuclear Power Station
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U. S. Nuclear Regulatory Commission
Resident Inspector's Office
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Mr. William Waters
Chairman, Board of Supervisors
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Grundy County Courthouse
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Director, Technical Assessment Div.
Office of Radiation Programs (AW-459)
US EPA - Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 South Dearborn Street
Chicago, Illinois 60604



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN STATION UNIT NO. 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 50
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 19, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

8012080095

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 12, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 50
PROVISIONAL OPERATING LICENSE NO. DPR-19
DOCKET NO. 50-237

Revise the Appendix "A" Technical Specifications as follows:

Remove

81B

--

81C

81C-1

81C-2

Replace

81B

81B-1

81C

81C-1

81C-2

3.5 LIMITING CONDITION FOR OPERATION

I. Average Planar LHGR

During steady state power operation, the average linear heat generation rate (LHGR) of all the rods in any fuel assembly, as a function of average planar exposure, at any axial location, shall not exceed the maximum average planar LHGR shown in

Figure 3.5-1 dependent on fuel type. If at any time during operation it is determined by normal surveillance that the limiting value for APLHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the APLHGR is not returned to within the prescribed limits within two (2) hours, the reactor shall be brought to the Cold Shutdown condition within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits.

4.5 SURVEILLANCE REQUIREMENT

I. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

J. Local LHGR

During steady state power operation, the linear heat generation rate (LHGR) of any rod in any fuel assembly at any axial location shall not exceed the maximum allowable LHGR as calculated by the following equation.

$$\text{LHGR}_{\text{max}} < \text{LHGR}_d \left[1 - \left(\frac{\Delta P}{P} \right)_{\text{max}} \left(\frac{L}{L_T} \right) \right]$$

LHGR_d - Design LHGR
 = 17.5 kw/ft, 7x7 fuel assemblies
 = 13.4 kw/ft, 8x8 fuel assemblies
 8x8 R fuel assemblies

$\left(\frac{\Delta P}{P} \right)_{\text{max}}$ - Maximum power spiking penalty

- .037 initial core fuel
 - .026 reload 1, 7x7 fuel
 - .022 8x8 fuel
 .000 8x8 R fuel

L_T - Total Core Length - 12 ft.

L - Axial distance from bottom of core

If at any time during operation it is determined by normal surveillance that the limiting value for LHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the LHGR is not returned to within the prescribed limits within two (2) hours, the reactor shall be brought to the Cold Shutdown condition within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits.

J. Local LHGR

The LHGR as a function of core height shall be checked daily during reactor operation at $\geq 25\%$ rated thermal power.

DPR-19

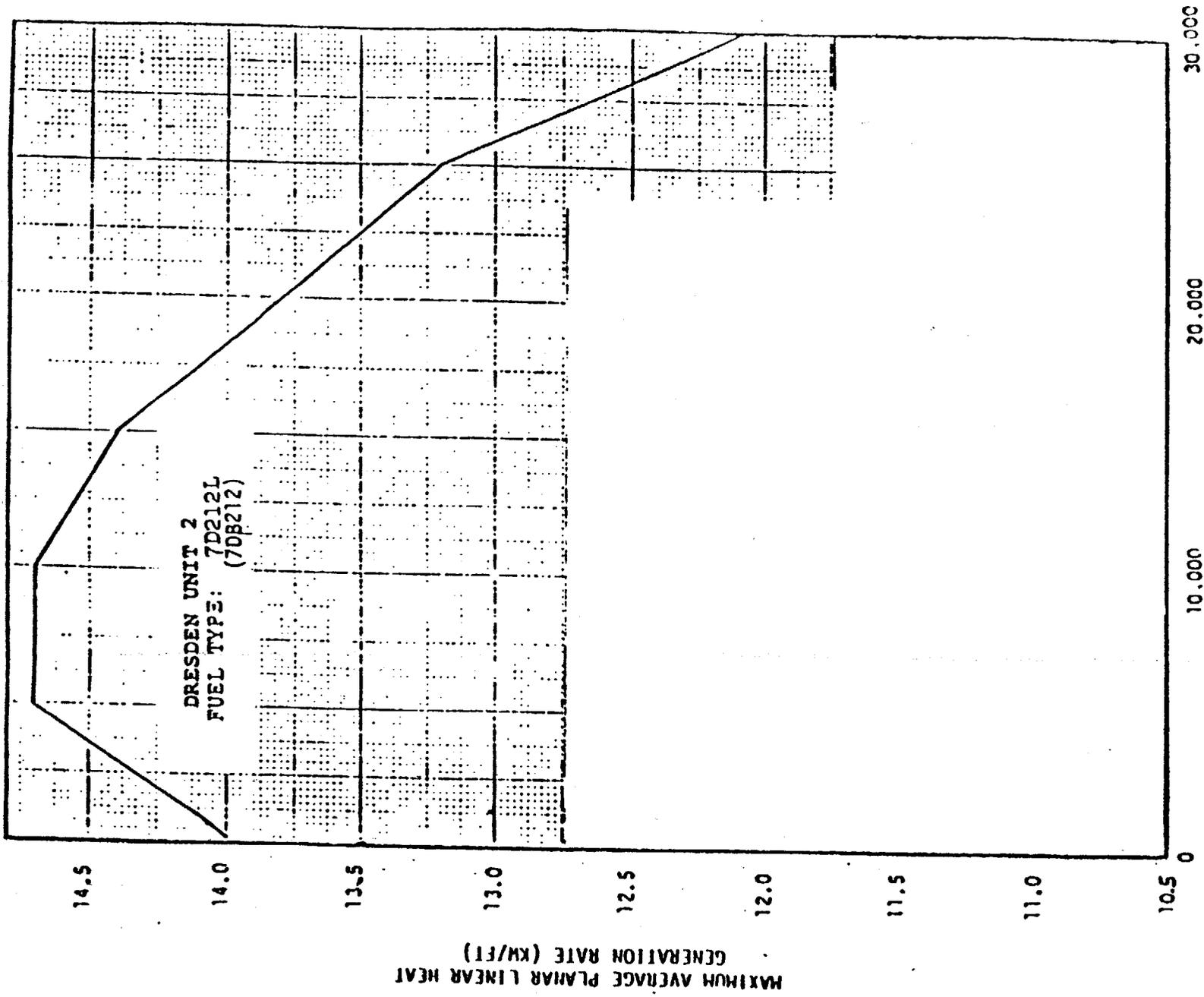


FIGURE 3.5-1
(Sheet 1 of 1)

PLANAR AVERAGE EXPOSURE (MWD/T)

MAXIMUM AVERAGE PLANAR LINEAR
HEAT GENERATION RATE (MW/FT)
VS. PLANAR AVERAGE EXPOSURE

HIC

Maximum Average Planar Linear Heat Generation Rate (kw/ft)

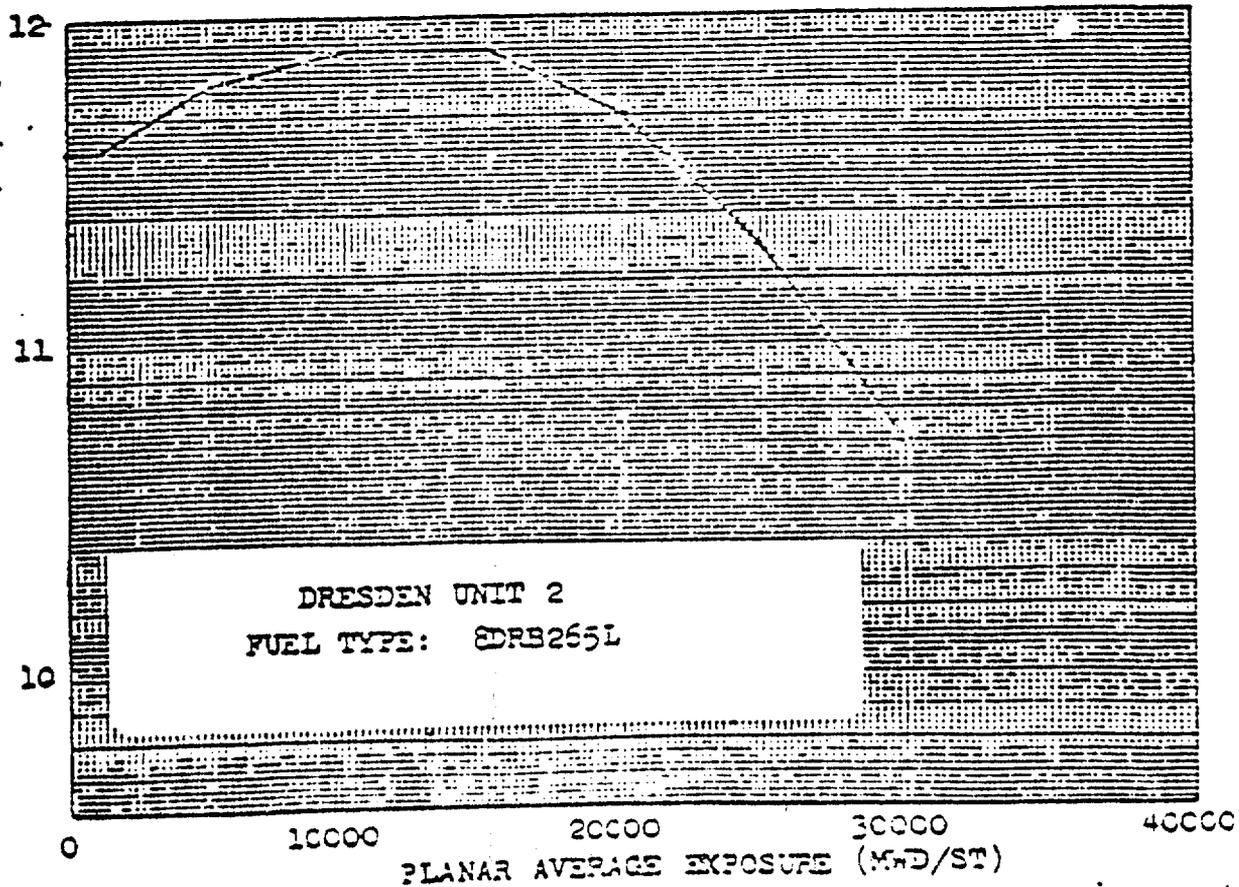
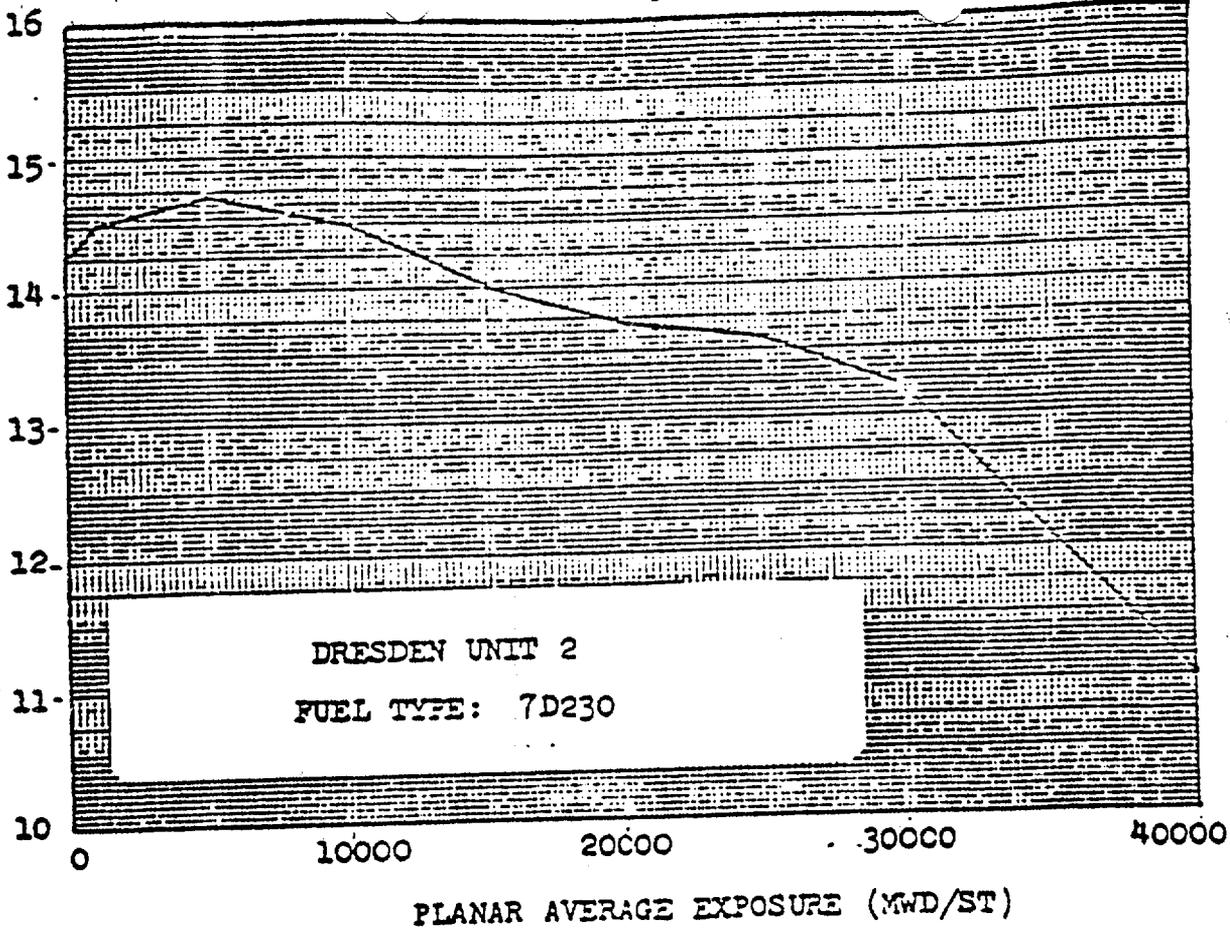


Figure 3.5-1 Maximum Average Planar Linear Heat Generation Rate (MAPLHR) vs. Planar Average Exposure (Sheet 2 of 3)

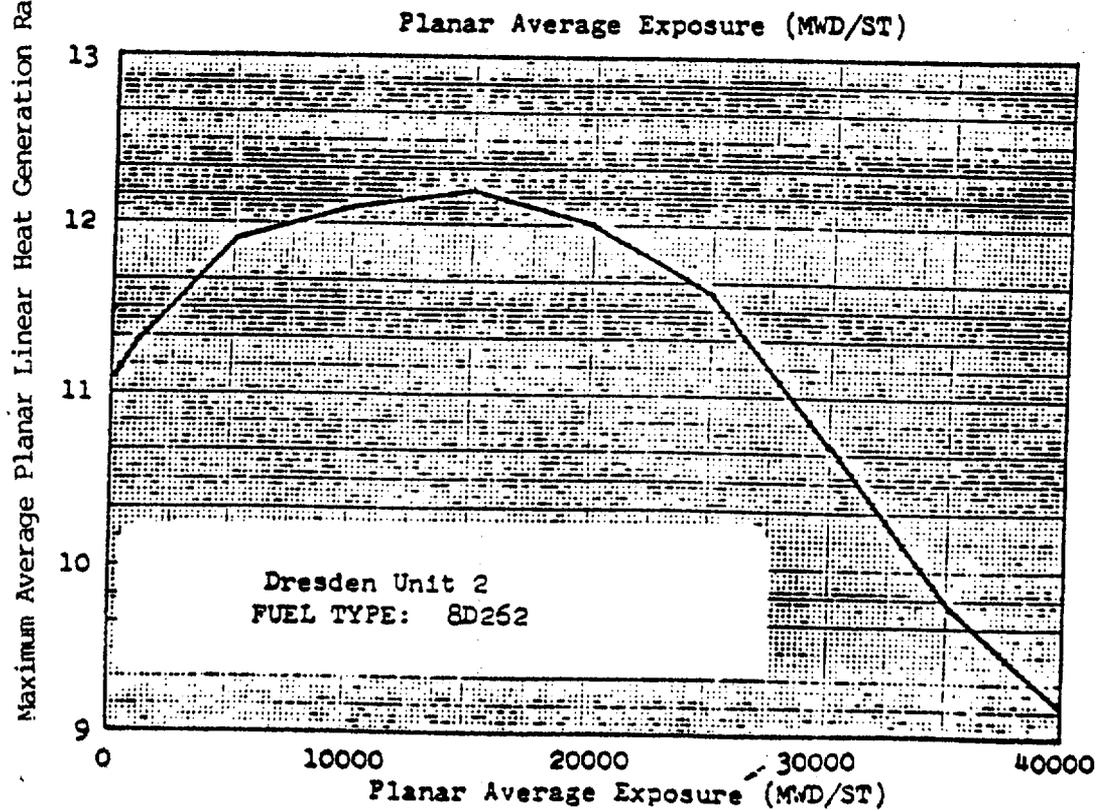
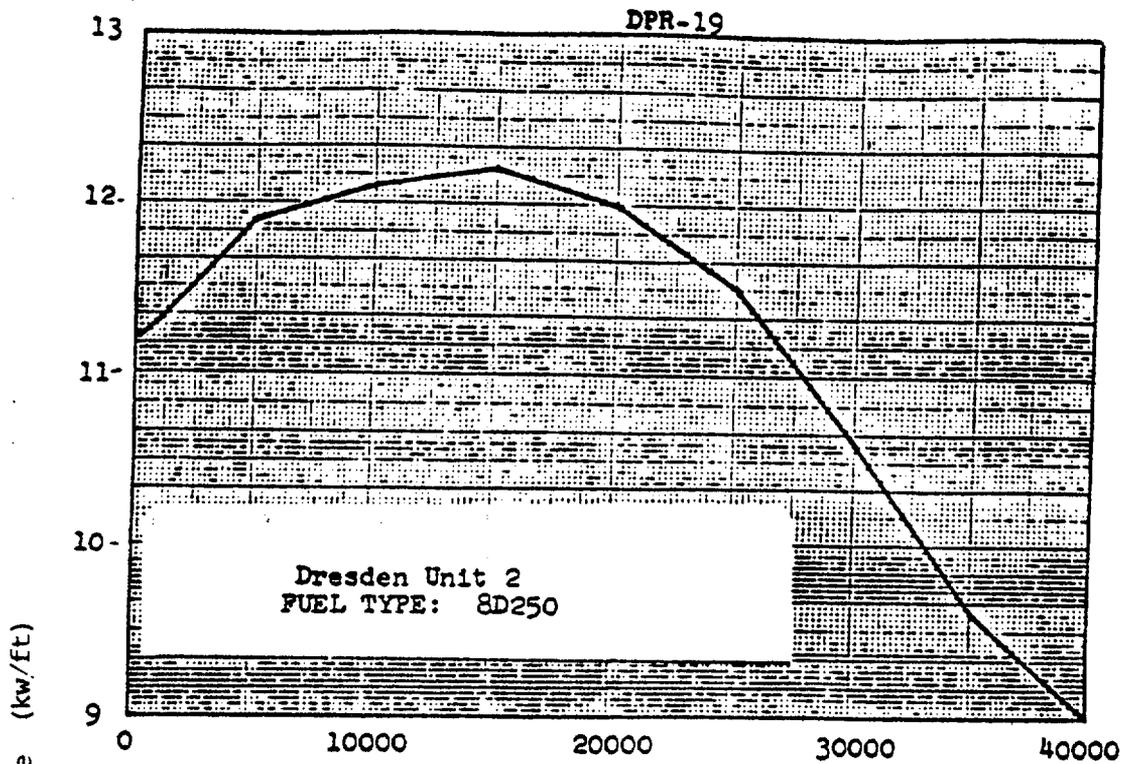


Figure 3.5-1 Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) vs. Planar Average Exposure (Sheet 3 of 3)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated March 19, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-29 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 59, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 12, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix "A" Technical Specifications as follows:

Remove

3.5/4.5-9
Fig. 3.5-1 (Sheet 1 of 3)
Fig. 3.5-1 (Sheet 2 of 3)
Fig. 3.5-1 (Sheet 3 of 3)
--

Replace

3.5/4.5-9
Figure 3.5-1 (Sheet 1 of 4)
Figure 3.5-1 (Sheet 2 of 4)
Figure 3.5-1 (Sheet 3 of 4)
Figure 3.5-1 (Sheet 4 of 4)

cycle by assuring that water can be run through the drain lines and actuating the air-operated valves by operation of the following sensors:

- 1) loss of air
 - 2) equipment drain sump high level
 - 3) vault high level
- d. The condenser pit 5-foot trip circuits for each channel shall be checked once a month. A logic system functional test shall be performed during each refueling outage.

I. Average Planar Linear Heat Generation Rate (APLHGR)

During steady-state power operation, the average linear heat generation rate (APLHGR) of all the rods in any fuel assembly, as a function of average planar exposure, at any axial location, shall not exceed the maximum average planar LHGR shown in Figure 3.5-1.

If at any time during operation it is determined by normal surveillance that the limiting value for APLHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the APLHGR is not returned in within the prescribed limits within 2 hours, the reactor shall be brought to the cold shutdown condition within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits.

J. Local LHGR

During steady-state power operation, the linear heat generation rate (LHGR) of any rod in any fuel assembly at any axial location shall not exceed the maximum allowable LHGR as calculated by the following equation. If at any time during operation it is determined by normal surveillance that the limiting value for LHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the LHGR is not returned to within the prescribed limits within

I. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at $\geq 25\%$ rated thermal power.

J. Local LHGR

Daily during steady-state power operation above 25% of rated thermal power, the local LHGR shall be checked.

DPR-29

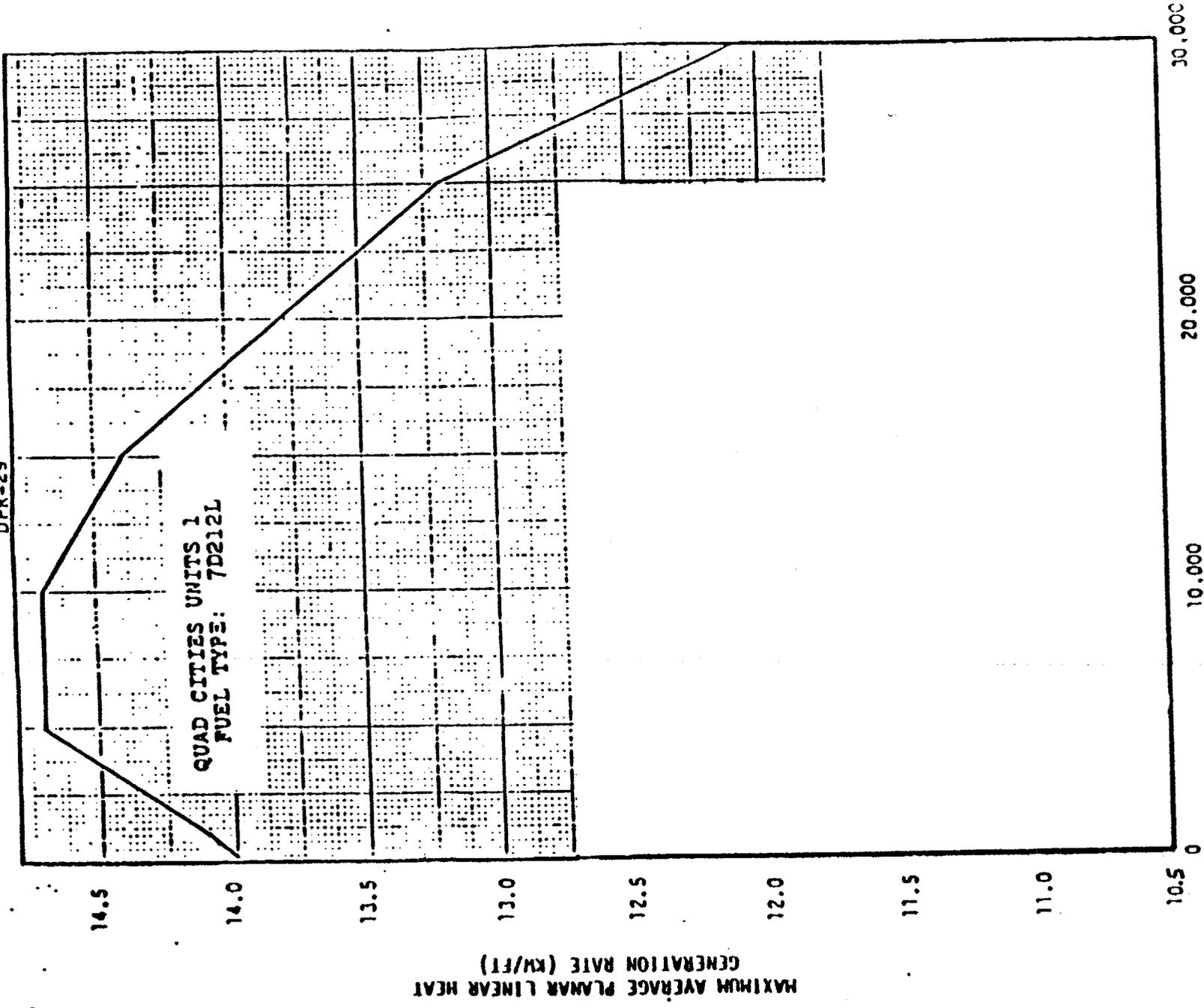


FIGURE 3.5-1
(Sheet 1 of 4)

PLANAR AVERAGE EXPOSURE (mD/T)
MAXIMUM AVERAGE PLANAR LINEAR
HEAT GENERATION RATE (MW/FT)
VS. PLANAR AVERAGE EXPOSURE

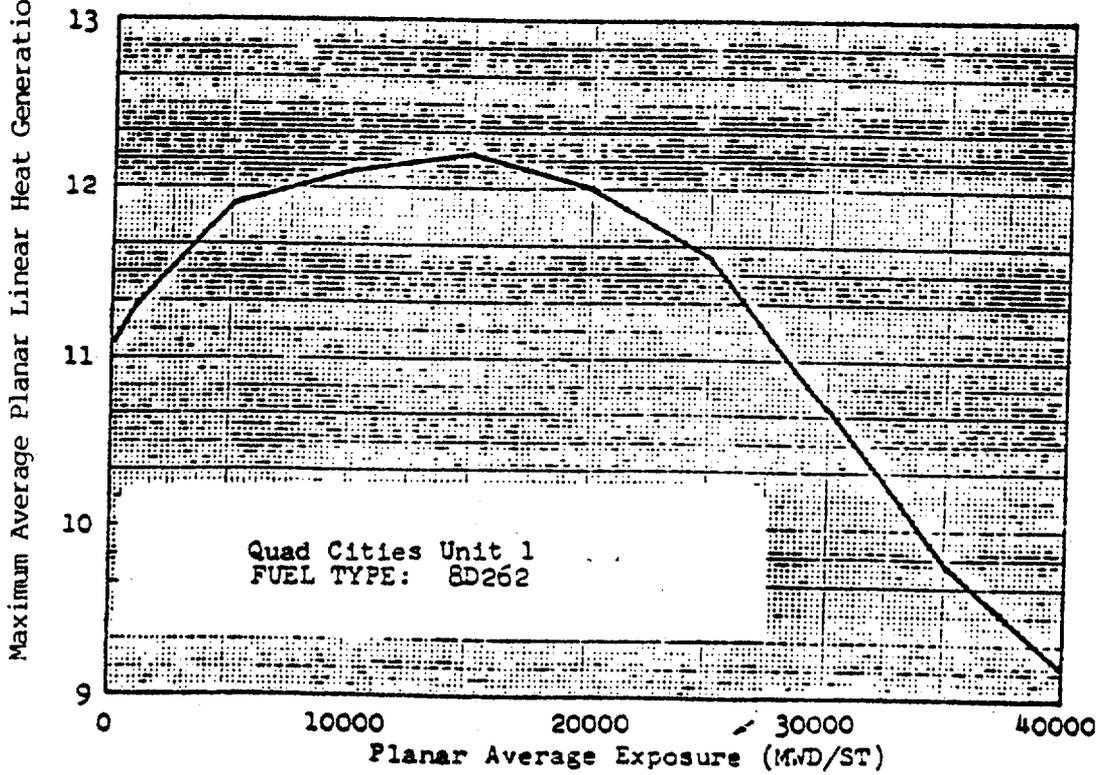
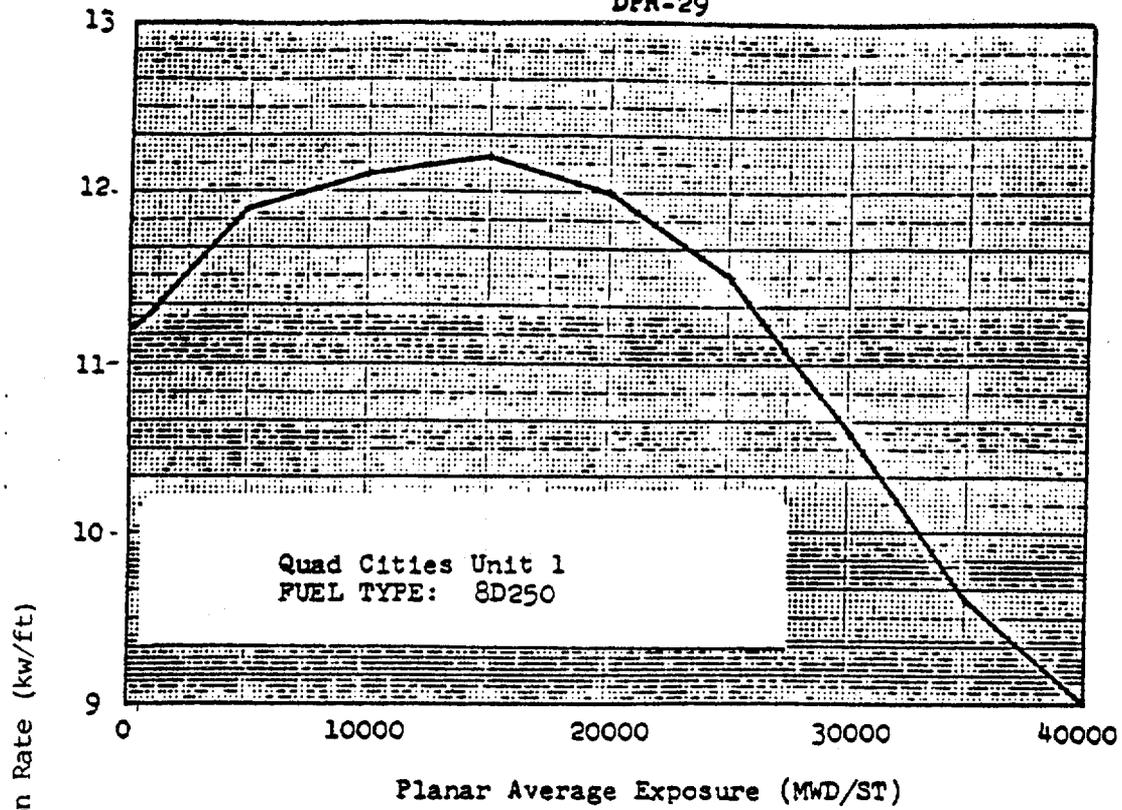


Figure 3.5-1 Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) vs. Planar Average Exposure (Sheet 4 of 4)

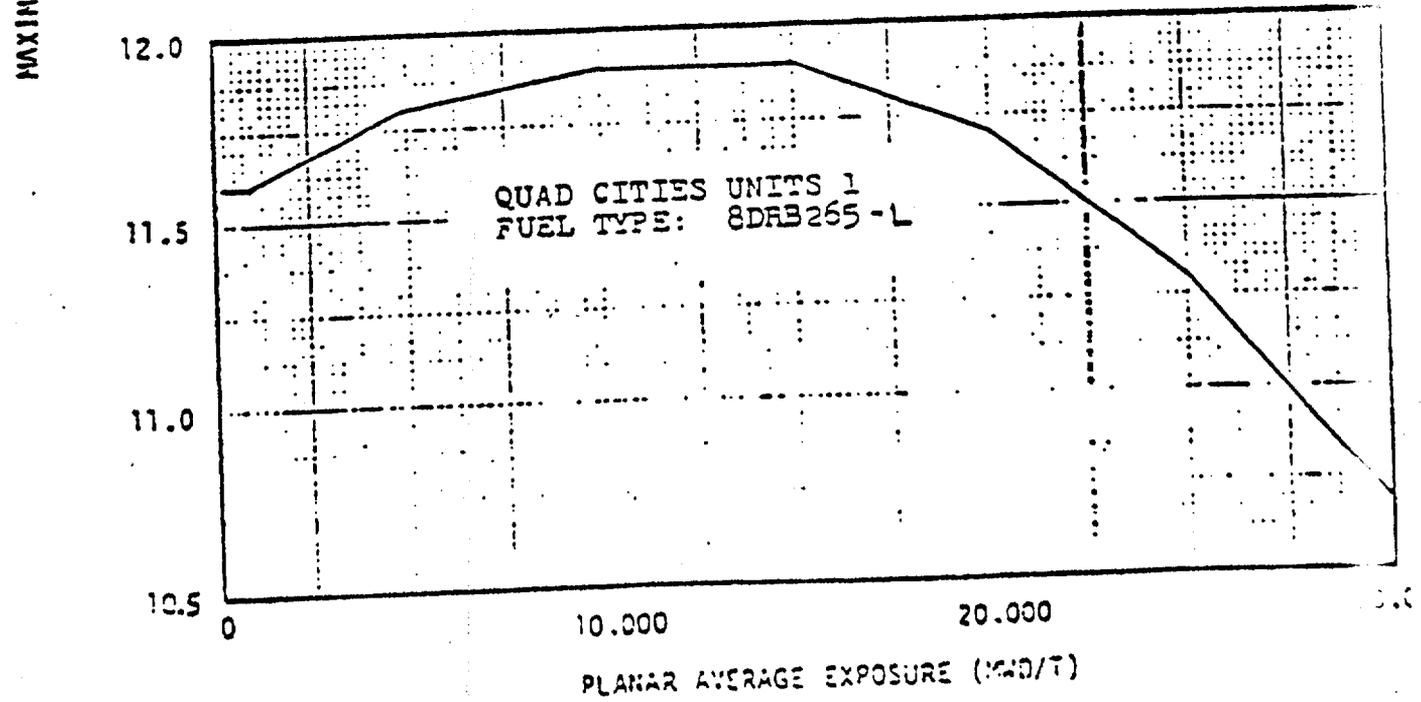
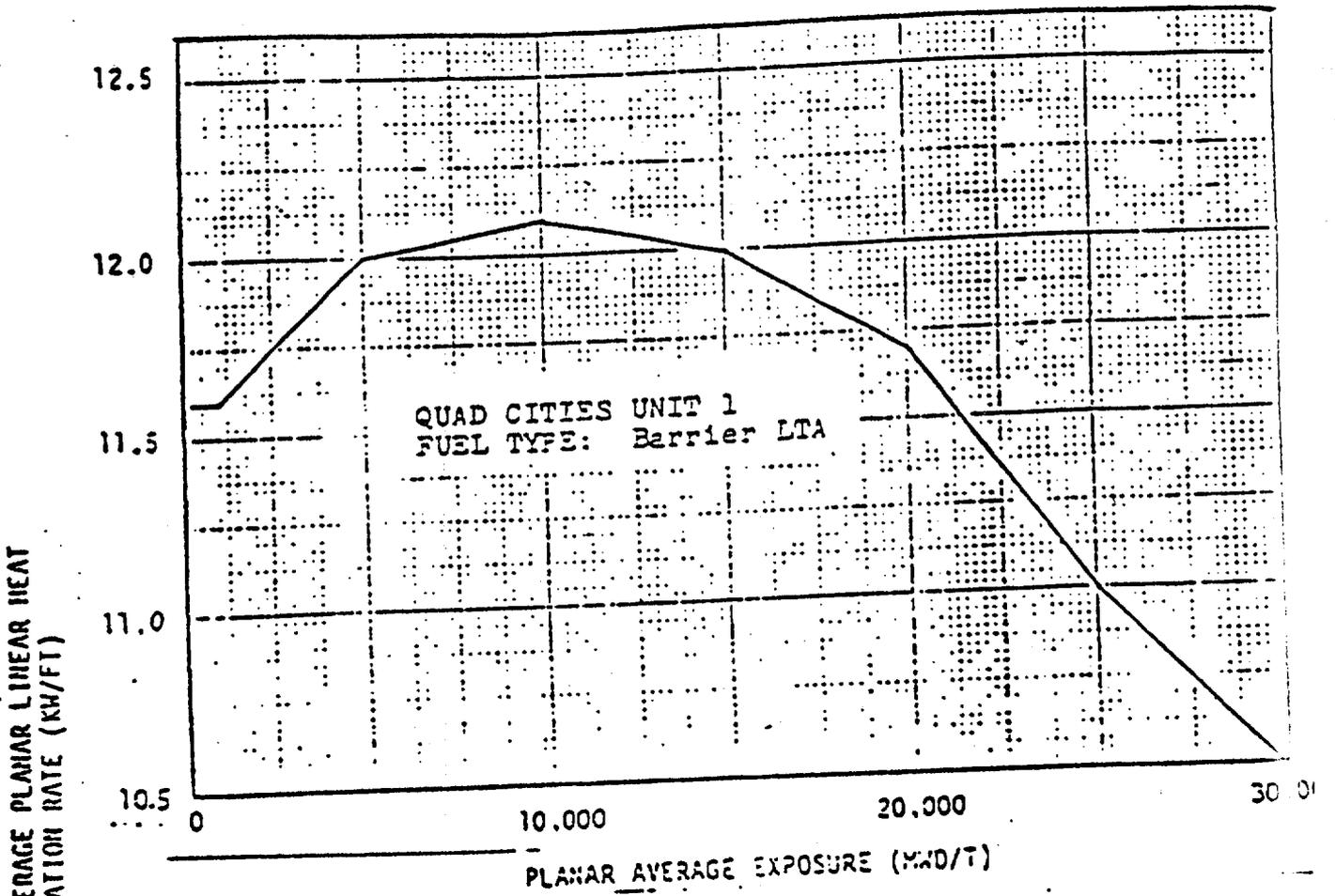


FIGURE 3.5-1
(Sheet 2 of 4)

MAXIMUM AVERAGE PLANAR LINEAR
HEAT GENERATION RATE (MAPLHGR)
VS. PLANAR AVERAGE EXPOSURE

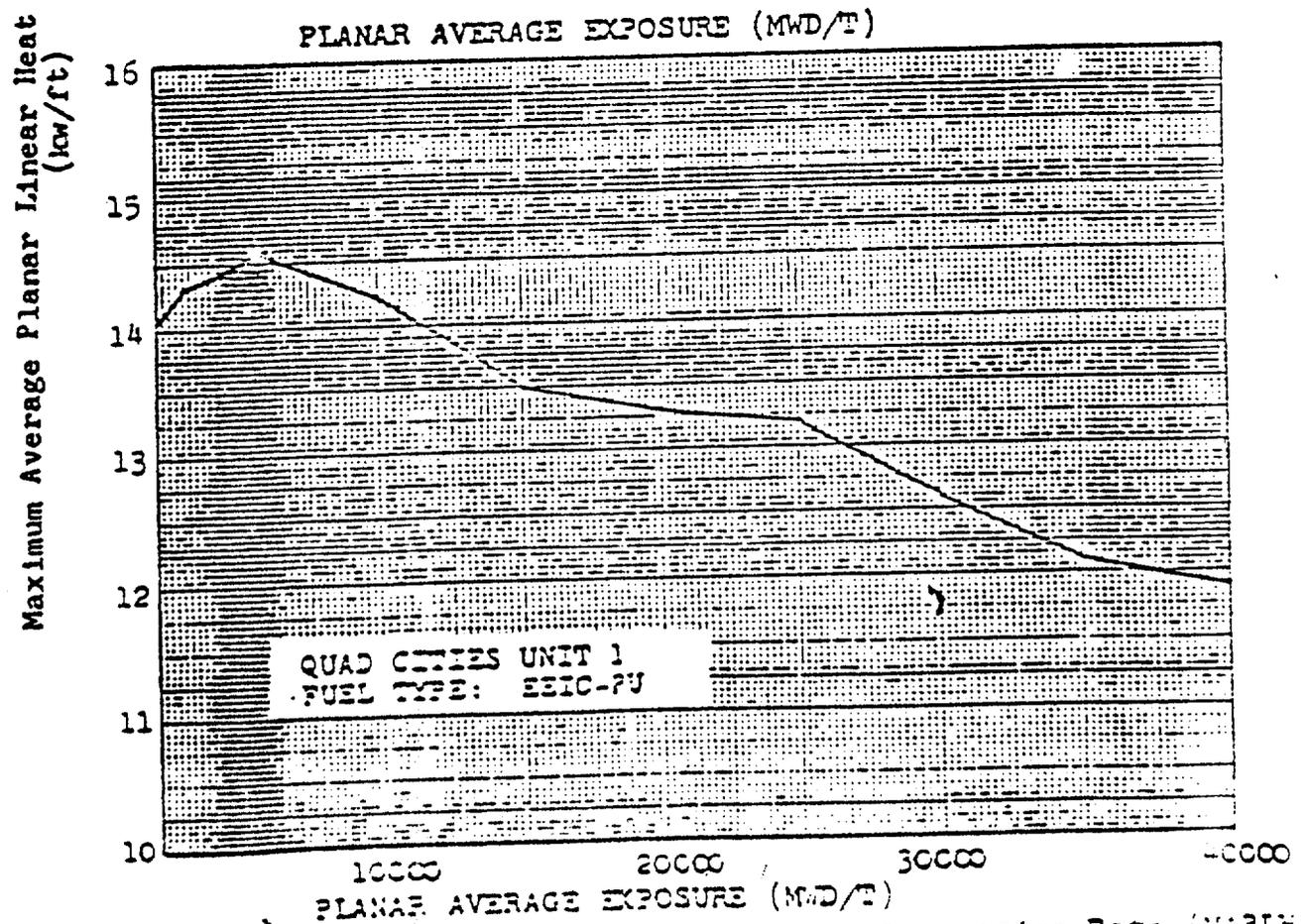
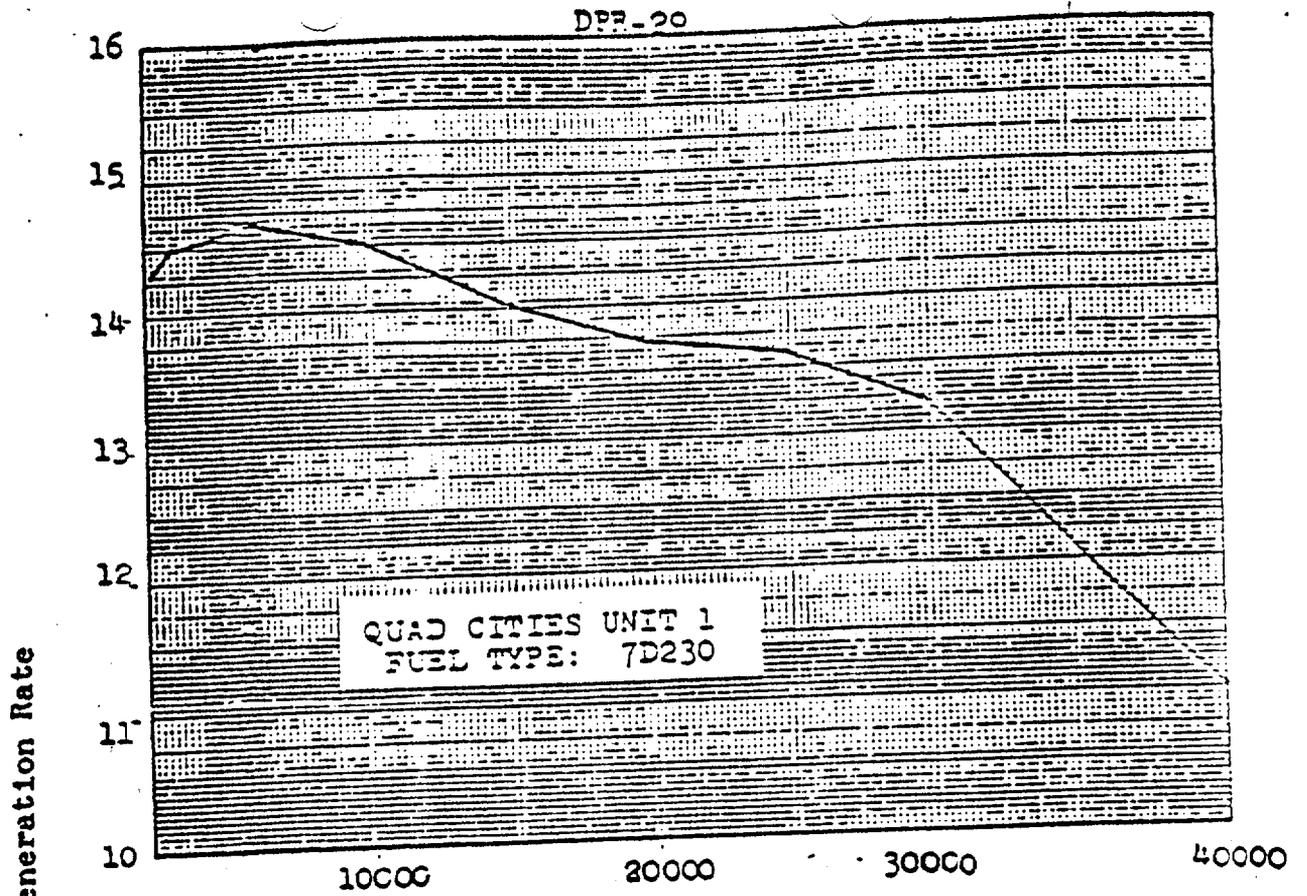


Figure 3.5-1 Maximum Average Planar Linear Heat Generation Rate (MAPLR) vs. Planar Average Exposure (Sheet 3 of 4)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-30

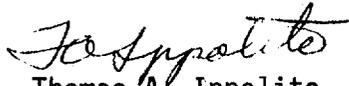
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated March 19, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 12, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix "A" Technical Specifications as follows:

Remove

Figure 3.5-1 (Sheet 1 of 4)
Figure 3.5-1 (Sheet 2 of 4)
Figure 3.5-1 (Sheet 3 of 4)
Figure 3.5-1 (Sheet 4 of 4)

--

Replace

Figure 3.5-1 (Sheet 1 of 5)
Figure 3.5-1 (Sheet 2 of 5)
Figure 3.5-1 (Sheet 3 of 5)
Figure 3.5-1 (Sheet 4 of 5)
Figure 3.5-1 (Sheet 5 of 5)

DFR-30

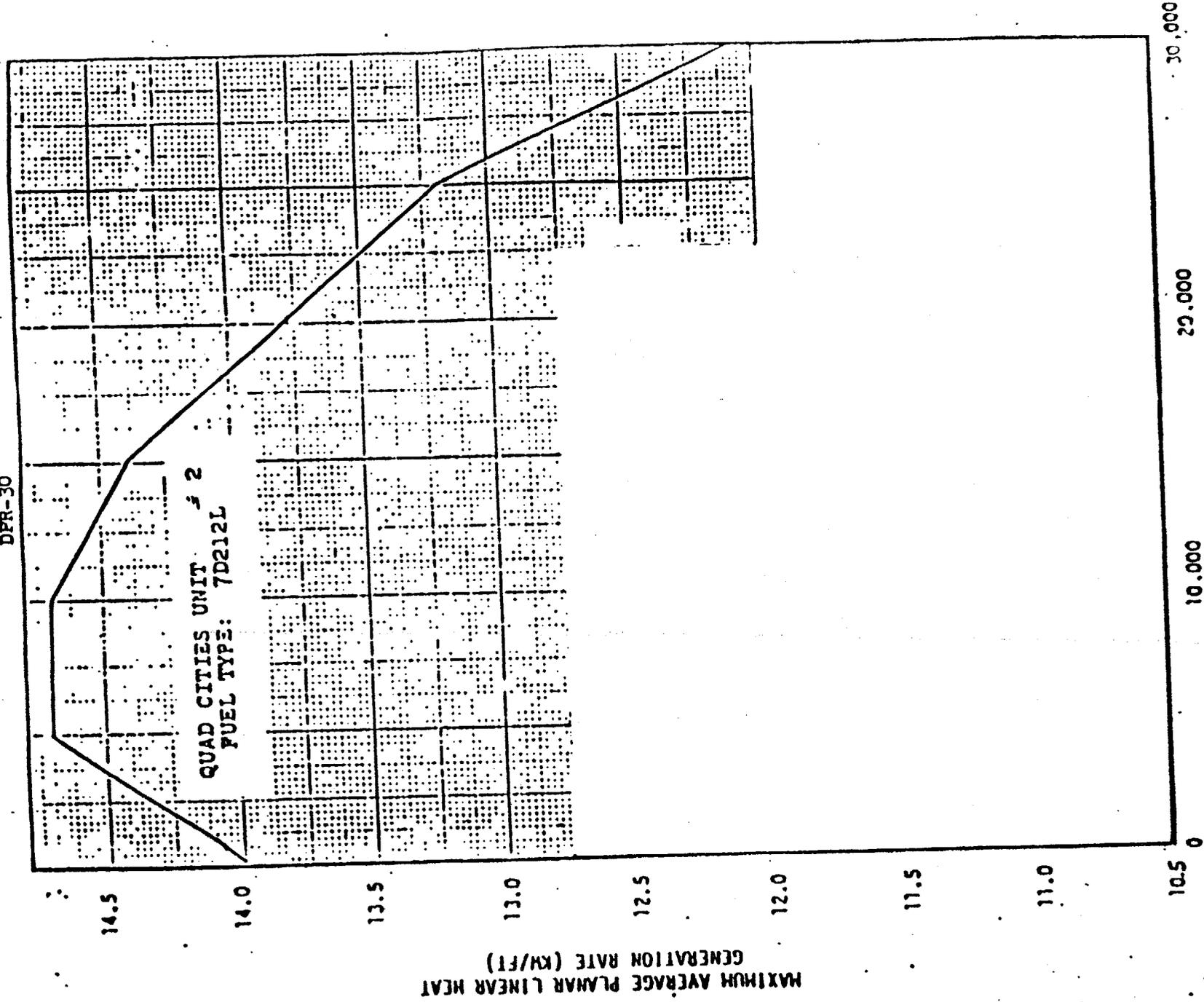


FIGURE 3.5-1
(Sheet 1 of 5)

PLANAR AVERAGE EXPOSURE (MWD/T)

MAXIMUM AVERAGE PLANAR LINEAR
HEAT GENERATION RATE (MWLHGR)
VS. PLANAR AVERAGE EXPOSURE

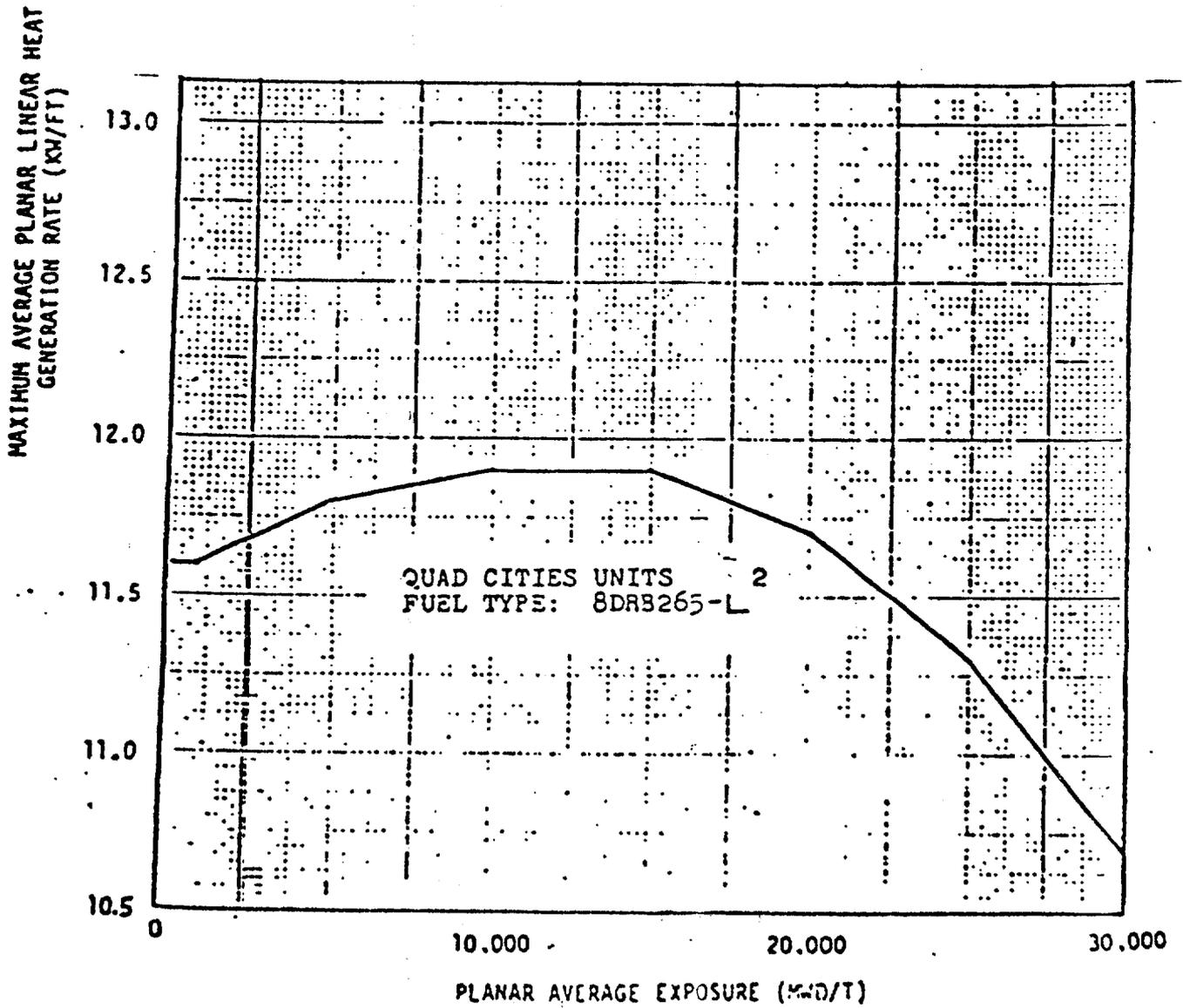


FIGURE 3.5-1
(Sheet 2 of 5)

MAXIMUM AVERAGE PLANAR LINEAR
HEAT GENERATION RATE (MAPLHGR)
VS. PLANAR AVERAGE EXPOSURE

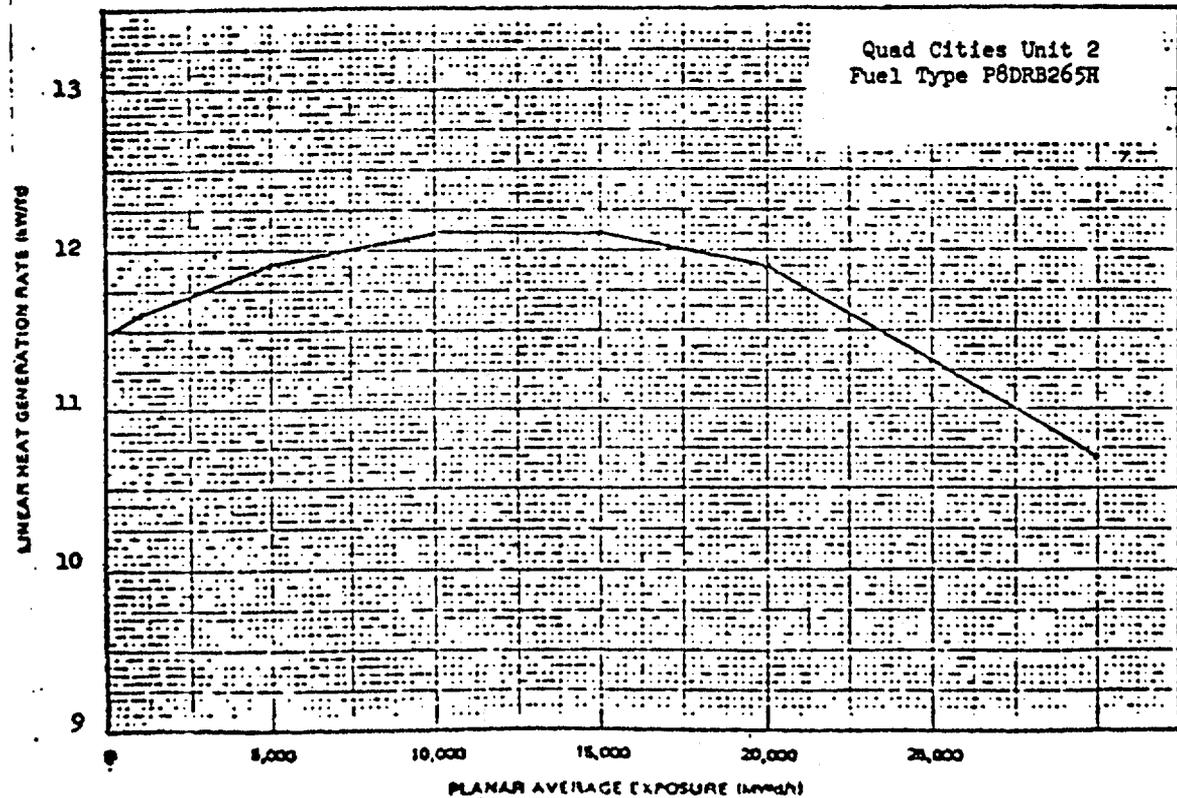
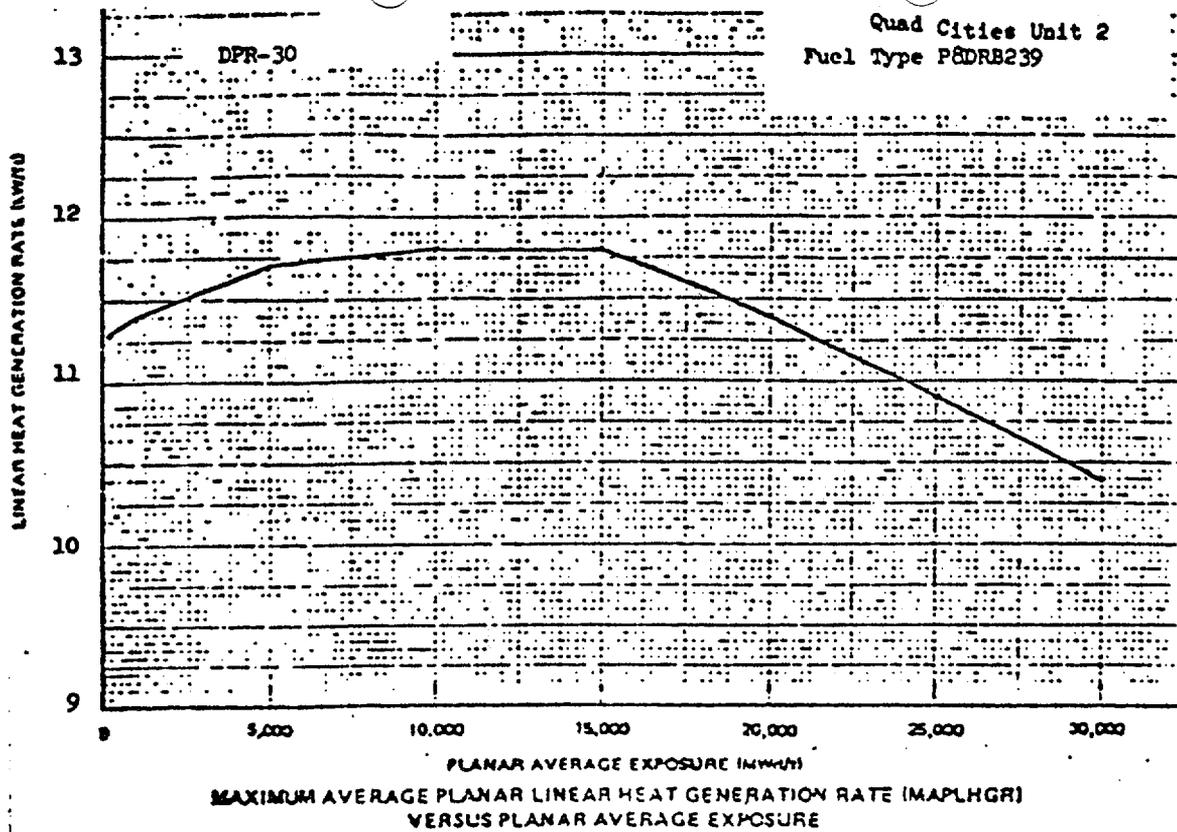
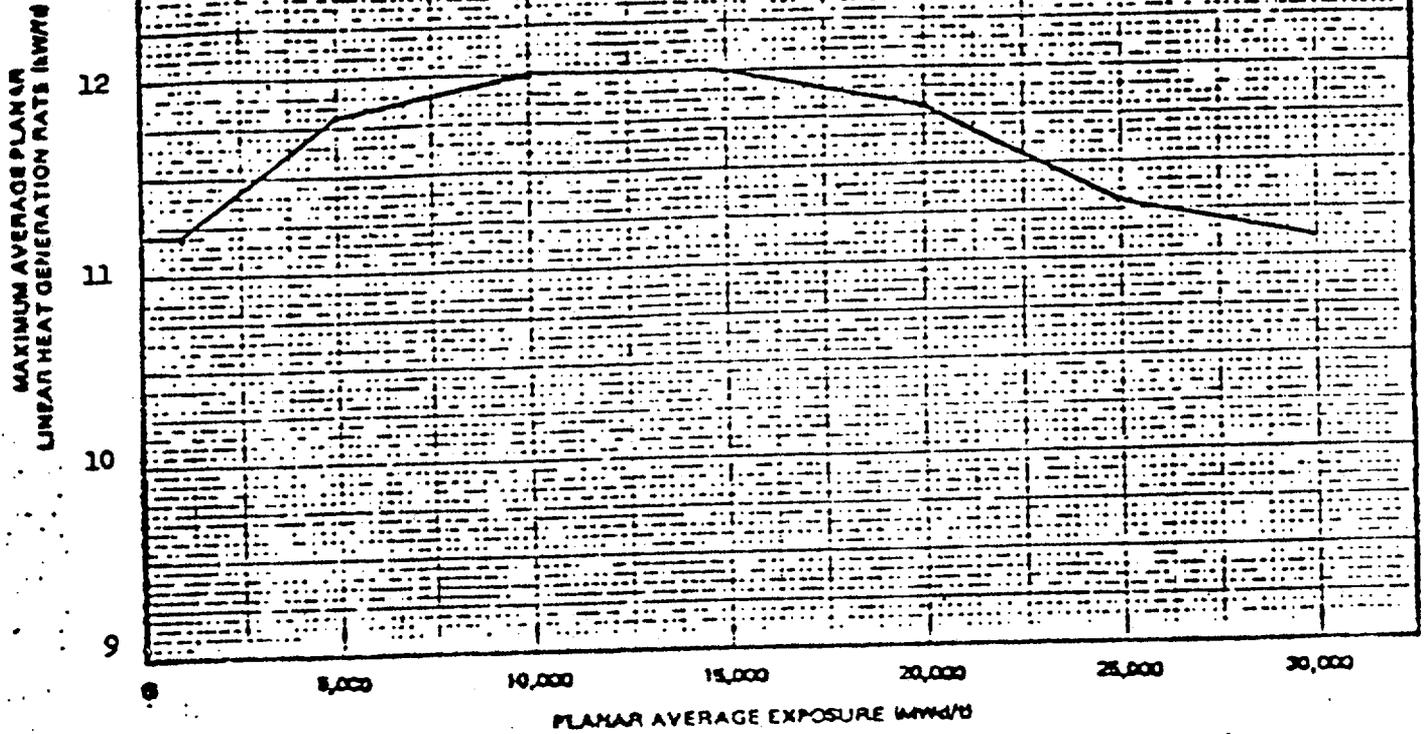


Figure 3.5-1 Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) vs. Planar Average Exposure
(Sheet 3 of 5)

Quad Cities Unit 2
Fuel Type P 8DRB282



MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR)
VERSUS PLANAR AVERAGE EXPOSURE

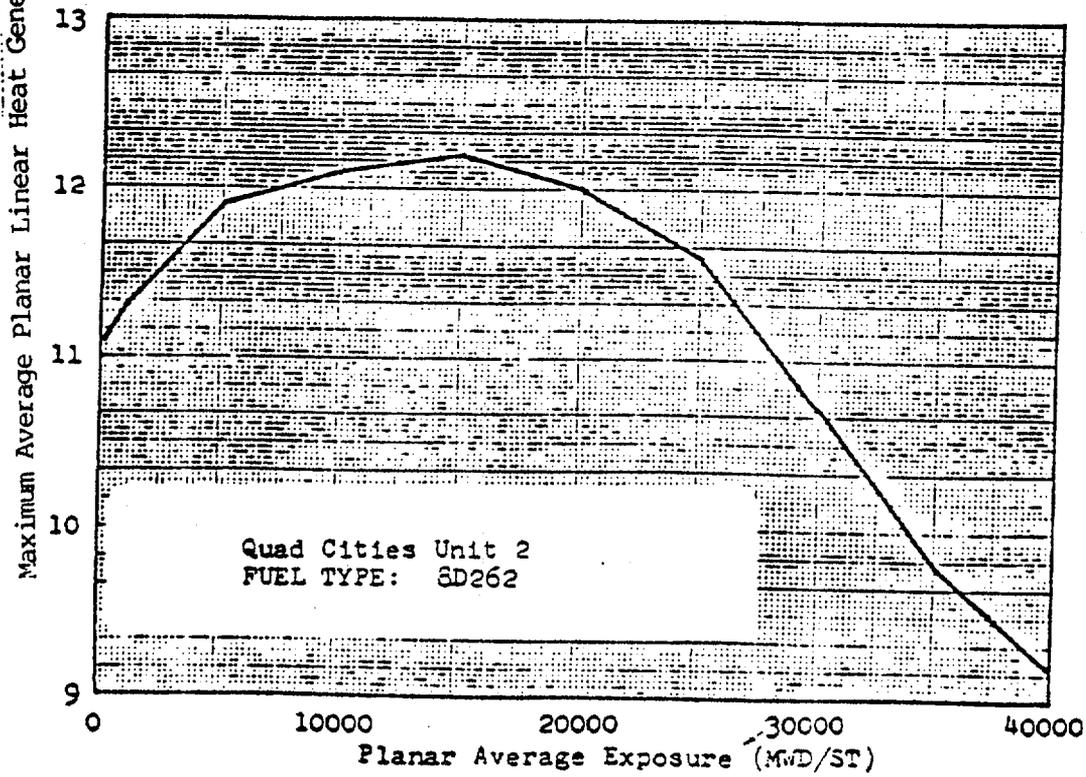
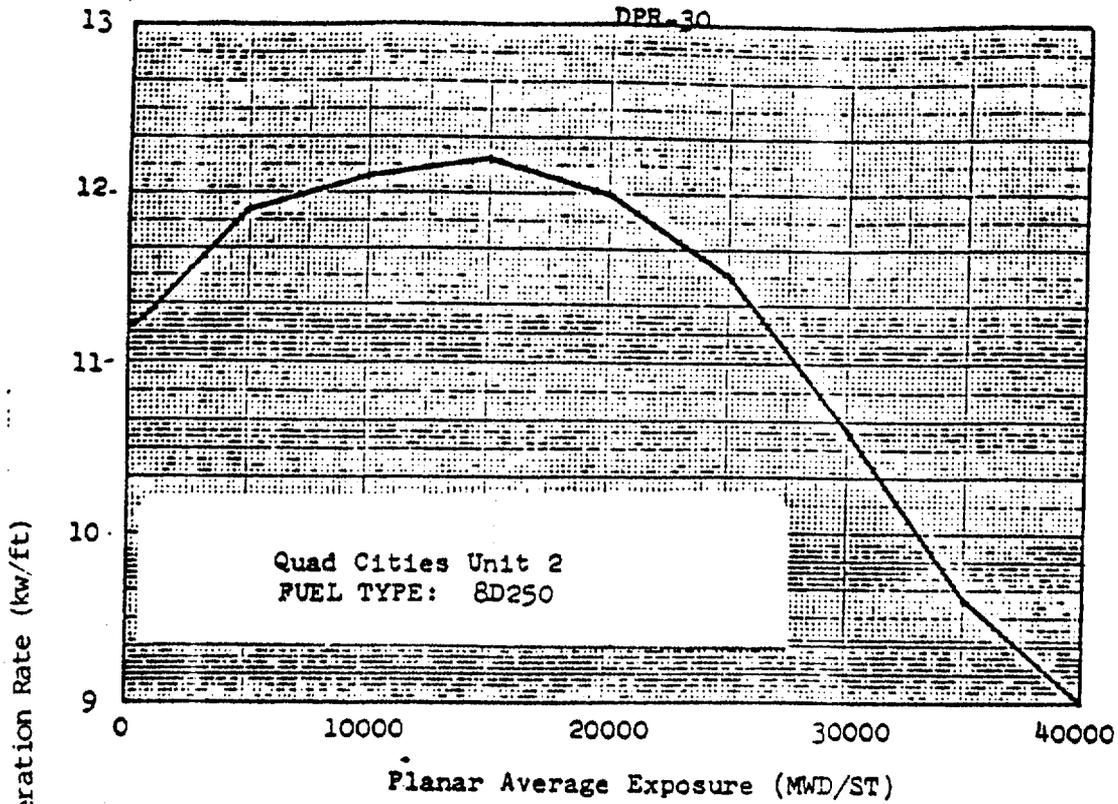


Figure 3.5-1 Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) vs. Planar Average Exposure (Sheet 5 of 5)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 50 TO PROVISIONAL OPERATING LICENSE NO. DPR-19,
AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-29, AND
AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DRESDEN STATION UNIT NO. 2

QUAD CITIES STATION UNIT NO. 1

QUAD CITIES STATION UNIT NO. 2

DOCKET NOS. 50-237, 50-254, 50-265

Introduction

By letter dated March 19, 1980 (Ref. 1), Commonwealth Edison Company (CECo, the licensee) proposed amendments to Appendix A, Technical Specifications for Operating Licenses DPR-19, DPR-29 and DPR-30 for Dresden Unit 2 and Quad Cities Units 1 and 2, respectively. The amendments would extend the Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) curves for standard 8x8 (8D250 and 8D262) fuel types to exposure values beyond those currently given in the Technical Specifications.

Evaluation

The extension of the MAPLHGR curves by addenda to Reference 2 has been proposed by the licensee. The analysis methods for the MAPLHGR calculations of Reference 2 have been previously accepted and approved by our letter of February 3, 1979 (Ref. 3) and December 28, 1979 (Ref. 4). The licensee's letter of March 19, 1980 presents the results of an analysis that extends MAPLHGR curves to 40,000 MWD/t (short ton) for standard 8x8 (8D250 and 8D262) fuel types. These results are presented as a revision to NEDO 24146A (Ref. 2).

8012080102

In our review of this proposed change, as in an earlier review (Ref. 4) we identified two areas having safety implications which require consideration. One relates to the 1% plastic strain criterion of the Zircaloy fuel rod cladding as the safety limit below which fuel damage due to overstraining is not expected to occur. At extended exposures, i.e., beyond 40,000 MWD/t peak pellet exposure, this safety limit had not been calculated. As in an earlier like case (Ref. 5) the licensee has agreed with us in that operation beyond this exposure is restricted and would require additional analyses. Also, the probability of a high exposure bundle achieving power levels that would challenge the 1% plastic strain limit is extremely small. This probability is based on reactor operation vs. fuel exposure characteristics and the licensee documentation of operational margin of linear power density in Reference 5.

Another area of consideration identified in our review is the enhancement of fission gas release at extended exposures. The licensee has confirmed by Reference 5 that, because APLHGR for high burnup rods is only 80 to 90% of MAPLHGR limits, there are no significant effects of enhanced fission gas release at extended exposures. This confirmation is in agreement with our generic evaluation (Ref. 6).

Exposure restrictions due to the 1% plastic strain limit and enhanced fission gas release at extended exposures are subjects of ongoing generic review by the staff. Although their treatment for the current application is acceptable, future generically related changes may be necessary. We have reviewed the licensee's proposal for extending the MAPLHGR curves to 40,000 MWD/t for standard 8x8 fuel types and have found it to be acceptable.

Environmental Considerations

In addition to the two areas of consideration having safety implications, the staff considered the proposed changes in the light of Tables S-3 and S-4 of 10 CFR Part 51, which address uranium fuel cycle and fuel transportation environmental impacts. The assumed maximum average level of irradiation of the irradiated fuel from the reactor is 33,000 MWD/MT (megawatt days per metric ton) or about 29,900 MWD/t (megawatt days per short ton). Although these amendments establish MAPLHGR limits for fuel burnup out to 40,000 MWD/t, these amendments will not cause the average fuel burnup of 33,000 MWD/MT (29,900 MWD/t) for irradiated fuel from the reactor to be exceeded.

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that these amendments involve an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 12, 1980

REFERENCES

1. March 19, 1980 - Letter from D. L. Peoples (Commonwealth Edison) to the Director of Nuclear Reactor Regulation (USNRC), "Dresden Station Unit 2, Quad Cities Station Units 1 and 2, Proposed Amendment to Appendix A, Technical Specifications to include Extended Exposure MAPLHGR Curves, NRC Docket Nos. 50-237 and 50-254/265," March 19, 1980.
2. "Loss-of-Coolant Accident Analysis Report for Dresden Units 2, 3 and Quad Cities Units 1, 2 Nuclear Power Stations," NEDO-24146A, April 1979.
3. Letter from T. A. Ippolito (USNRC) to C. Reed (CECo), February 3, 1979.
4. Letter from T. A. Ippolito (USNRC) to D. L. Peoples (CECo), December 28, 1979.
5. Letter from R. F. Janacek (CECo) to H. R. Denton (USNRC), December 20, 1979.
6. Memorandum from P. S. Check to A. Schwencer, et al., "Enhanced Fission Gas Release," October 13, 1978.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-237, 50-254 AND 50-265COMMONWEALTH EDISON COMPANYANDIOWA-ILLINOIS GAS AND ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 50 to Provisional Operating License No. DPR-19, issued to Commonwealth Edison Company, which revised the Technical Specifications for operation of the Dresden Nuclear Power Station, Unit No. 2, located in Grundy County, Illinois. The Commission has also issued Amendment No. 59 to Facility Operating License No. DPR-29, and Amendment No. 54 to Facility Operating License No. DPR-30, issued to Commonwealth Edison Company and Iowa-Illinois Gas and Electric Company, which revised the Technical Specifications for operation of the Quad-Cities Nuclear Power Station, Unit Nos. 1 and 2, located in Rock Island County, Illinois. The amendments are effective as of the date of issuance.

The amendments revise the technical specifications to establish MAPLHGR limits for standard 8x8 (8D250 and 8D262) fuel types for exposure values beyond those currently given in the technical specifications. This is appropriate for extended operating cycle operation and results in more effective use of fuel and reduced spent fuel generation.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate

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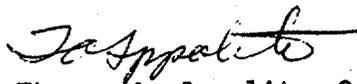
findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated March 19, 1980, (2) Amendment No. 50 to License No. DPR-19, Amendment No. 59 to License No. DPR-29, and Amendment No. 54 to License No. DPR-30, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C., and at the Morris Public Library, 604 Liberty Street, Morris, Illinois, for Dresden 2 and at the Moline Public Library, 504 - 17th Street, Moline, Illinois, for Quad Cities 1 and 2. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 12th day of November, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing