

DEC 1 6 1983

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Docket No. 50-374

Mr. Dennis L. Farrar  
Director of Nuclear Licensing  
Commonwealth Edison Company  
Post Office Box 767  
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: La Salle County Station, Unit 2, Issuance of Facility  
Operating License

The U. S. Nuclear Regulatory Commission (NRC) has issued the enclosed Facility Operating License NPF-18 to Commonwealth Edison Company for La Salle County Station, Unit 2, located in Brookfield Township, La Salle County, Illinois. License No. NPF-18 authorizes operation of the La Salle County Station, Unit 2, at reactor core power levels not in excess of 3323 megawatts thermal (100% power). Pending Commission approval, operation is restricted to power levels not to exceed 5 percent of full power (166 megawatts thermal).

Also enclosed is a copy of a related Federal Register notice which has been forwarded to the Office of the Federal Register for publication.

Two signed copies of Amendment No. 4 to Indemnity Agreement No. B-84 which covers the activities authorized under License No. NPF-18 are also enclosed. Please sign and return one copy to this office.

Supplement No. 7 to the Safety Evaluation Report for La Salle County Station, Units 1 and 2 has been issued. Two copies are enclosed; 18 additional copies will be forwarded to you in about a week.

Sincerely,

Original signed by:

Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

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Enclosures:

1. Facility Operating License No. NPF-18
2. Federal Register Notice
3. Amendment No. 4 to Indemnity Agreement B-84
4. Supplement No. 7 to the SER (2)

DL:DJR  
DGE:enhut  
12/15/83

cc w/enclosures: \*SEE PREVIOUS CONCURRENCES

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DATE		12/02/83	12/02/83	12/13/83	12/02/83	12/02/83

December 16, 1983

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La Salle County Station, Unit 2

Docket No. 50-374

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\* w/Technical Specifications

La Salle

Mr. Dennis L. Farrar  
Director of Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

cc: Philip P. Steptoe, Esquire  
Suite 4200  
One First National Plaza  
Chicago, Illinois 60603

Dean Hansell, Esquire  
Assistant Attorney General  
188 West Randolph Street  
Suite 2315  
Chicago, Illinois 60601

William G. Guldemon, Resident Inspector  
La Salle, NPS, U.S.N.R.C.  
P. O. Box 224  
Marseilles, Illinois 61364

Chairman  
La Salle County Board of Supervisors  
La Salle County Courthouse  
Ottawa, Illinois 61350

Attorney General  
500 South 2nd Street  
Springfield, Illinois 62701

Department of Public Health  
Attn: Chief, Division of Nuclear Safety  
535 West Jefferson  
Springfield, Illinois 62761

The Honorable Tom Corcoran  
United States House of Representatives  
Washington, D. C. 20515

Chairman  
Illinois Commerce Commission  
Leland Building  
527 East Capitol Avenue  
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager  
Nuclear Facility Safety  
Illinois Department of Nuclear Safety  
1035 Outer Park Drive, 5th Floor  
Springfield, Illinois 62704



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LA SALLE COUNTY STATION, UNIT 2

FACILITY OPERATING LICENSE

License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for license filed by the Commonwealth Edison Company (licensee), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the La Salle County Station, Unit 2 (facility) has been substantially completed in conformity with Construction Permit No. CPPR-100 and the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission (except as exempted from compliance in Section 2.D below);
  - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - E. The Commonwealth Edison Company is technically qualified to engage in the activities authorized by this license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
  - F. The Commonwealth Edison Company has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;
  - G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;

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- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of this Facility Operating License No. NPF-18, subject to the conditions for protection of the environment set forth in the Environmental Protection Plan attached as Appendix B, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Based on the foregoing findings regarding this facility, Facility Operating License NPF-18 is hereby issued to the Commonwealth Edison Company (the licensee) to read as follows:
- A. This license applies to the La Salle County Station, Unit 2, a boiling water nuclear reactor and associated equipment, owned by the Commonwealth Edison Company. The facility is located in Brookfield Township, La Salle County, Illinois, and is described in the licensee's "Final Safety Analysis Report", as supplemented and amended, and in the licensee's Environmental Report, as supplemented and amended.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses Commonwealth Edison Company:
    - (1) Pursuant to Section 103 of the Act and 10 CFR Part 50, to possess, use, and operate the facility at the designated location in Brookfield Township, La Salle County, Illinois, in accordance with the procedures and limitations set forth in this license;
    - (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
    - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
    - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3323 megawatts thermal (100% power) in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational tests, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license. Pending Commission approval, this license is restricted to power levels not to exceed 5 percent of full power (166 megawatts thermal).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Conduct of Work Activities During Fuel Load and Initial Startup

The licensee shall review by committee all Unit 2 Preoperational Testing and System Demonstration activities performed concurrently with Unit 2 initial fuel loading or with the Unit 2 Startup Test Program to assure that the activity will not affect the safe performance of the Unit 2 fuel loading or the portion of the Unit 2 Startup Program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security and health physics, with respect to performance of the activity concurrently with the Unit 2 fuel loading or the portion of the Unit 2 Startup Program being performed. The committee for the review shall be composed of at least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by section 4.4 of ANSI N18.7-1971. At least one of these three shall be a senior member of the Assistant Superintendent of Operation's staff.

(4) Inservice Testing of Pumps and Valves (Section 3.9.6, SER\*)

Pursuant to 10 CFR Part 50.55a, the relief identified in the submittals dated February 18, 1983, August 2, 1983, and November 12, 1983 that the licensee has requested from the pump and valve testing requirements of 10 CFR Part 50, Section 55.55a(g)(2) and (g)(4)(i) is granted for that portion of the initial 120-month period during which the staff completes its review.

(5) Environmental Qualifications (Section 3.11, SER, SSER #1, SSER #2)

Prior to startup after the first refueling outage but not later than March 1985, the licensee shall environmentally qualify all electrical equipment as required by 10 CFR 50.49.

(6) Surveillance of Control Blade (Section 4.2.3.14, SER)

IE Bulletin No. 79-26, Revision 1, "Boron Loss from BWR Control Blades," describes certain actions to be taken by licensees to determine boron loss from BWR control blades. The licensee shall comply with items 1, 2 and 3 of this bulletin and submit a written response on item 3 within 30 days after plant startup following the first refueling outage.

(7) Low Pressure in Pump Discharge of the Control Rod Drive (Section 4.6.2, SSER #2, and Section 7.2.3.2, SSER #7)

Prior to completion of the startup test program, the licensee shall install instrumentation that would automatically shut down the reactor in the event of low control rod drive pump discharge pressure. This automatic scram shall be activated during startup and refueling modes only.

(8) Containment Isolation System (Section 6.2.1.1, SSER #2 and Section 3.9.3.1, SSER #5, SSER #6)

Prior to startup after the first refueling outage, the licensee shall replace the eight 26-inch and two 8-inch vent and purge isolation valves with valves that can close in 10-seconds or less and that do not require AC power to close.

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\*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

(9) Standby Liquid Control System Cable Terminations  
(Section 7.2.3.2, SSER #7)

Prior to initial criticality, the licensee shall provide physical separation of at least six inches between terminations at terminal blocks for redundant standby liquid control system circuits in both local and control room panels.

(10) Cable Separation Concerns (Section 7.3.3.2, SSER #7)

Prior to startup after the first refueling outage, the licensee shall resolve the deficiencies described in Appendix D to SSER #7 regarding separation between redundant cables.

(11) Low and/or Degraded Grid Voltage (Section 8.2.2.2, SER)

Prior to startup after the first refueling outage, the licensee shall install a second level of undervoltage protection on Divisions 1 and 2.

(12) Reliability of Diesel-Generators (Sections 8.3.1.1 and 9.6.3.4, SER, SSER #5)

Prior to startup after the first refueling outage, the licensee shall implement the following design modifications with respect to diesel-generator reliability:

(a) The controls and monitoring instrumentation shall be removed from the engine and engine skid, except instruments qualified for this location. The non-qualified control and monitoring instruments shall be installed on a free standing floor mounted panel in such a manner (including the use of vibration isolation mounts if necessary) that any induced vibrations will not result in cyclic fatigue failure for the expected life of the instrument. The licensee shall submit an evaluation for NRC staff approval that demonstrates this design objective has been achieved.

(b) The EMD MI 9644 manufacturer's modification.

(13) Direct Current Power Systems (Section 8.3.1.2, SER)

Prior to completion of the startup test program, the following additional instrumentation shall be provided in the control room for the 125 and 250-volt direct current systems for Divisions 1 and 2 and the 125-volt Division 3 direct current system:

(1) battery current (ammeter-charge/discharge), (2) battery charger output voltage (voltmeter), (3) battery charger output current (ammeter), (4) battery high discharge rate alarm, and (5) battery charger trouble alarm. In the interim, the licensee shall implement approved procedures to monitor battery current, battery charger output voltage, and battery charger output current at the local panels at least once per eight-hour shift.

(14) Control of Heavy Loads (Section 9.1, SSER #1, SER #5)

Prior to startup after the first refueling, the licensee shall have made commitments acceptable to the NRC regarding the guidelines of Sections 5.1.2 through 5.1.6 of NUREG-0612.

(15) Fire Protection Program  
(Section 9.5, SER, SSER #1, SSER #2, SSER #3, SSER #5, SSER #7)

- (a) The licensee shall maintain in effect all provisions of the approved fire protection program.
- (b) Prior to initial criticality, the licensee shall replace the B diesel fire pump engine and perform a test in accordance with Sections 11-2.3, 11-2.4 and 11-2.5 of NFPA-20/1983.
- (c) Prior to initial criticality, the licensee shall revise the periodic fire pump surveillance tests to be in accordance with Section 11.3 of NFPA-20/1983 and be conducted on 18-month intervals.
- (d) Prior to initial criticality, the licensee shall revise the fire protection loop flow test in accordance with Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition published by the National Fire Protection Association.
- (e) Prior to exceeding five percent power, the licensee shall provide a detailed computer analysis of the fire protection water system to demonstrate system adequacy.
- (f) Prior to exceeding five percent power, the licensee shall provide a history of the deviations observed in operation of the diesel fire pumps.
- (g) Prior to exceeding five percent power, the licensee shall provide the results of an analysis of the service water system's capability to perform as a backup water supply for the fire protection. This analysis will include a description of the surveillance procedures for the service water pump system when used as a fire protection water supply, and an evaluation of limitations on the service water system due to the use of these pumps as a fire water supply.

- (h) Prior to exceeding five percent power, the licensee shall provide a surveillance program for NRC staff approval to ensure operability of the fire dampers. This program will include a periodic operability test of a sample population of accessible dampers.
- (i) Prior to exceeding five percent power, the licensee shall assure, in a manner acceptable to the NRC staff, that a fire in a single fire zone will not result in the inadvertent opening of all three high/low pressure interface valves between the reactor core isolation cooling and residual heat removal systems.
- (j) Prior to startup after the first refueling outage, the licensee shall provide the suppression pool level and temperature monitoring instrumentation at the remote shutdown panel which are electrically isolated from the control room, and assure the instrumentation are operable with the necessary surveillance procedures in place.
- (k) Prior to startup after the first refueling outage, the licensee shall assure, in a manner acceptable to the NRC staff, that a fire in any single fire zone will not affect the control of the fuel oil transfer pumps for diesel generators "0" and "2A".

(16) Industrial Security (Section 13.6, SER, SSER#3, SSER #5)

The licensee shall maintain in effect and fully implement all provisions of the approved physical security plan, guard training and qualification plan, and contingency plan, including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plans which contain safeguard information are collectively entitled: "La Salle County Station Security Plan Units 1 and 2", Revision 13, dated November 1982 (transmitted by letter dated November 29, 1982); "La Salle Station Contingency Plan", Revision 6, dated May 1980 (transmitted by letter dated May 2, 1980); Revision 7, dated June 1980 (transmitted by letter dated June 27, 1980); Revision 13, dated November 1982 (transmitted by letter dated November 29, 1982); "La Salle Station Guard Training and Qualification Plan", Revision 6, dated May 1980 (transmitted by letter dated May 2, 1980); Revision 8, dated August 1980 (transmitted by letter dated August 11, 1980); Revision 10, dated July 1981 (transmitted by letter dated July 10, 1981); Revision 11, dated December 1981 (transmitted by letter dated December 24, 1981); and "La Salle Nuclear Power Station Contingency Plan", dated March 24, 1980, as revised by letters dated June 27, 1980 and November 29, 1982.

The licensee is exempted from the provisions of 10 CFR 73.55(d)(9), but shall meet all other commitments of the Physical Security Plan and the following additional items.

- (a) Change all keys, locks, and combinations and related equipment used to control access to protected areas and vital areas at least every 12 months.
- (b) Issue keys, locks, combinations, and other access control devices to protected and vital areas only to those individuals who possess access authorization to those areas.
- (c) Change keys, locks, combinations, and related equipment to which an individual had access within 5 days and immediately for card keys after access authorization is withdrawn due to lack of trustworthiness, reliability, or inadequate work performance.
- (d) Change keys, locks, combinations, and related equipment within five (5) days, and immediately for card keys when there is evidence that such equipment or devices have been lost or compromised.

(17) Initial Test Program (Section 14, SER, SSER #7)

The licensee shall conduct the initial startup test program (set forth in Section 14 of the licensee's Final Safety Analysis Report, as amended) without making any modifications of this program unless such modifications are in accordance with the provisions of 10 CFR Section 50.59. In addition, the licensee shall not make any major modifications to this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- (a) Elimination of any test identified in Section 14 of the licensee's Final Safety Analysis Report, as amended, as being essential;
- (b) Modification of test objectives, methods or level 1 acceptance criteria for any test identified in Section 14 of the licensee's Final Safety Analysis Report, as amended, as being essential;
- (c) Performance of any test at a power level different from that described in the program; and
- (d) Failure to complete any tests included in the described program (planned or scheduled) for power levels up to the authorized power level.

(18) NUREG-0737 Conditions (Section 22.2)

The licensee shall complete the following conditions to the satisfaction of the NRC. These conditions reference the appropriate items in Section 22.2, "TMI Action Plan Requirements for Applicants for Operating Licenses," in the Safety Evaluation Report and Supplements 1, 2, 3, 4, 5, and 7, NUREG-0519.

(a) Emergency Response Capability (I.D.1, SER, SSER #1, SSER #5)

The licensee shall correct the design deficiencies for the control room and complete the other related emergency response capabilities as required by Attachment 2 of this license.

(b) Direct Indication of Safety/Relief Valve Position (II.D.3, SER, SSER #2 and Section 3.10, SSER #7)

Prior to completion of the startup test program, the licensee shall qualify the safety/relief valve position indicator.

(c) Instrumentation for Detection of Inadequate Core Cooling (II.F.2, SER, SSER #1, SSER #5)

The licensee shall implement the NRC staff's requirements regarding upgrading the liquid level instrumentation or inclusion of additional instrumentation for detecting inadequate core cooling based on the NRC staff's review of the BWR Owners Group Reports SLI-8211 and SLI-8218 and the licensee's plant-specific evaluation report addressing the recommendations of the BWR Owners Group reports. Any required modifications shall be completed on a schedule acceptable to the NRC staff.

(d) Modification of Automatic Depressurization System Logic - Feasibility for Increased Diversity for Some Event Sequences (II.K.3.18, SER, SER #1, SSER #3, SSER #5)

Prior to startup after the first refueling outage, the licensee shall:

- (i) Install modifications to the Automatic Depressurization system described in the licensee's letter dated July 1, 1983. The final circuit diagrams and an analysis of the bypass timer time delay shall be submitted for NRC staff review and approval prior to installation.
- (ii) Incorporate into the Plant Abnormal Procedures the usage of the inhibit switch; and
- (iii) Modify the Technical Specifications to provide the bypass timer and manual inhibit switch.

- D. Exemptions from certain requirements of Appendices G, H, and J to 10 CFR Part 50, and to 10 CFR Part 73 are described in the Safety Evaluation Report and Supplement No. 1, No. 2, No. 3, and No. 5 to the Safety Evaluation Report. In addition, an exemption was requested until completion of the first refueling from the requirements of 10 CFR §70.24. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, these exemptions are hereby granted pursuant to 10 CFR 50.12. With the granting of this exemption the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.
- E. Before engaging in additional construction or operational activities which may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement and its Addendum, the licensee shall provide a written notification to the Director of the Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.
- F. With the exception of Section 2, Item C(2), the licensee shall report any violations of the requirements contained in Section 2.C and 2.E of this license within 24 hours by telephone and confirm by telegram, mailgram, or facsimile transmission to the NRC Regional Administrator, Region III, or that administrator's designee, no later than the first working day following the violation, with a written followup report within 14 days.
- G. The licensee shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- H. The licensee shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

1. This license is effective as of the date of issuance and shall expire at Midnight on December 16, 2023.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

*D.G. Eisenhower*

Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

Attachments/Appendices:

1. Attachment 1
2. Attachment 2
3. Appendix A - Technical Specifications (NUREG-1013)
4. Appendix B - Environmental Protection Plan

Date of Issuance: **DEC 16 1983**

*[Signature]*  
HR Denton  
12/15/83

\*SEE PREVIOUS CONCURRENCES

OFFICE	DL:LB#2/PM	DL:LB#2/LA	DL:LB#2/BC	OELD	DL:AD/L	DL:OED	NRR
SURNAME	ABournia*:kw	EHylton*	ASchwencer*	CWoodhead*	TMNovak*	<del>DGEisenhut</del>	EGCase
DATE	12/12/83	12/12/83	12/12/83	12/13/83	12/2/83	12/15/83	12/ /83

ATTACHMENT 1 TO LASALLE COUNTY UNIT 2 OPERATING LICENSE NPF-18

This attachment identifies certain preoperational tests, system demonstrations and other items which must be completed to the Commission's satisfaction in accordance with the operational plateaus as identified below.

- A. The following items must be completed prior to proceeding to operational Mode 2 (initial criticality).
1. The following Preoperational Tests shall be completed, including all reviews:
    - a. Containment Monitoring System (PT-CM-201)
    - b. Post LOCA Containment Monitoring System (PT-CM-202)
    - c. Main Steam Isolation Valve Leakage Control System (PT-MS-201A)
    - d. Main Steam Isolation Valves and Main Steam System Instrumentation (PT-MS-201B)
    - e. Off-Gas System (PT-OG-201)
    - f. Reactor Recirculation System (PT-RR-201)
    - g. Post LOCA Hydrogen Recombiners (PT-VP-202)
    - h. Transverse Incore Probe System (PT-NR-202)
  2. Personnel emergency equipment shall be installed in the Reactor Building.
- B. The Preoperational Test for the Containment Ventilation System (PT-VP-203) shall be completed, including all reviews prior to exceeding a primary system temperature of 200°F in Operational Mode 2.
- C. The Preoperational Test for the Automatic Depressurization System and Main Steam System Safety/Relief Valves (PT-MS-201C) shall be completed prior to exceeding a primary system pressure of 350 psig in Operational Mode 2. All reviews of this test shall be completed within one week of completed testing.
- D. The System Demonstration for Pipe Vibration Monitoring (SD-SI-201) shall be completed, including all reviews prior to exceeding 5% of rated full power operation.

- E. The following items must be completed for the Fire Protection Program prior to exceeding 5% of rated full power operation:
1. The licensee shall conduct a review of the adequacy of its fire detector system installation. This evaluation shall specifically address the number and location of fire detectors, and ventilation effects on the ability of the detectors to sense a fire. The licensee shall initiate compensatory measures in areas found deficient to assure adequate fire detection capability. These compensatory measures will remain in place until permanent modifications to correct any deficiencies are made to the fire detection system.
  2. The licensee shall demonstrate that Emergency Lighting Units with at least an 8-hour battery power supply have been installed and are functionally operable in the Control Room, in the Auxiliary Electric Equipment Room at the Remote Shutdown Panel, and on a dedicated access route from the Control Room to the Auxiliary Electrical Equipment Room which will include two egress doors from the Control Room.
  3. The licensee shall have in place approved procedures defining egress routes for control room evacuation to the remote shutdown panel.
  4. The licensee shall install the rollup door in the Unit 2 Auxiliary Electric Equipment Room.
  5. The licensee shall have in place approved procedures for manual override of those ventilation dampers identified in LaSalle Unit 2 ECN-M-763-LA and LaSalle Unit 1 Modification M-1-1-82-311 as requiring such operation during certain fires.
  6. The licensee shall meet the breathing air supply requirements specified in 10 CFR 50 Appendix R, Section III.H.
  7. The licensee shall provide a redundant fire stop for the penetrations identified as exceeding established and tested cable density criteria.
  8. The licensee shall have in place approved surveillance procedures for fire barrier penetration seals, including detailed acceptance criteria.
  9. The hose nozzles on the refueling floor will be replaced with nozzles appropriately rated.
  10. The licensee shall be in conformance to the fire watch qualification requirements of NFPA 51B.
  11. The licensee shall provide procedures for maintaining the "0" and "2A" diesel generator day tanks at the nominal full position.

F. The following items must be completed prior to startup following the first refueling outage:

1. The licensee shall complete all modifications to the fire detection system identified as being required.
2. The licensee shall review the performance of the fire protection system jockey booster pumps and implement any changes required by this review.

ATTACHMENT 2

The licensee shall complete the following Supplement No. 1 to NUREG-0737 requirements on the schedule noted below:

<u>Title</u>	<u>Requirement</u>	<u>Schedule</u>
1. Safety Parameter Display System (SPDS)	1(a) Submit Safety Analysis	
	(i) Criteria for parameter selection including V&V description	12/31/83
	(ii) HFR of data display and functions	07/01/86
	(iii) Verify parameter selection	07/01/86
	1(b) SPDS Operational - Defined as design, hardware and software installation, functional testing, and initial operator training complete	Prior to completion of startup test program
2. Detailed Control Room Design Review (DCRDR)	2(b) Submit a summary report <sup>1,2/</sup> to the NRC, including a proposed schedule for implementation	11/01/85
3. Regulatory Guide 1.97 Application to Emergency Response Facilities	3(b) Submit a final report including a schedule for installation	08/01/86
4. Upgrade Emergency Operating Procedures (EOPs) to BWROG Rev. 3	4(a) Submit a Procedures Generation Package to the NRC	09/30/84
	4(b) Implement the Upgraded EOPs	09/30/85
5. Emergency Response Facilities	5(a) Technical Support Center (TSC) fully functional	<u>3/</u>
	5(c) Emergency Operations Facility (EOF) fully functional	<u>3/</u>

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<sup>1/</sup> There are human engineering deficiencies associated with operational differences between Unit 1 and Unit 2 (the main turbine control valves opening sequence and position, and high pressure core spray initiation reset logic) and the licensee has not committed to change these. If such commitment is not made, a detailed analysis of such error shall be made during the DCRDR.

- 2/ The licensee shall perform a DCRDR analysis to determine the potential for operator error associated with common system alarms found on only one Unit's annunciator panels.
- 3/ Modifications and construction of the structures have been completed. The TSC and EOF are not considered fully functional until the changes resulting from the R. G. 1.97 and human factors reviews are implemented, and all testing and training are completed.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-374COMMONWEALTH EDISON COMPANYNOTICE OF ISSUANCE OF FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission), has issued Facility Operating License No. NPF-18, to Commonwealth Edison Company (the licensee) which authorizes operation of the La Salle County Station, Unit 2 (the facility), by Commonwealth Edison Company at reactor core power levels not in excess of 3323 megawatts thermal (100 percent power) in accordance with the provisions of the license, the Technical Specifications and the Environmental Protection Plan. However, the license contains a condition currently limiting operation to 5 percent of full power (166 megawatts thermal). Authorization to operate beyond 5 percent of full power will require specific Commission approval.

La Salle County Station, Unit 2, is a boiling water nuclear reactor located at the licensee's site in Brookfield Township, La Salle County, Illinois approximately 65 miles southwest of Chicago, Illinois. The license is effective as of the date of issuance.

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license. Prior public notice of the overall action involving the proposed issuance of an operating license was published in the Federal Register on June 9, 1977 (42 F.R. 29576-29577).

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The Commission has determined that the issuance of this license will not result in any environmental impacts other than those evaluated in the Final Environmental Statement and its Addendum since the activity authorized by the license is encompassed by the overall action evaluated in the Final Environmental Statement and its Addendum.

For further details with respect to this action, see (1) Facility Operating License No. NPF-18, complete with Technical Specifications and Environmental Protection Plan; (2) the report of the Advisory Committee on Reactor Safeguards dated April 16, 1981; (3) the Commission's Safety Evaluation Report dated March 1981, Supplement No. 1 dated June 1981, Supplement No. 2 dated February 1982, Supplement No. 3 dated April 1982, Supplement No. 4 dated July 1982, Supplement No. 5 dated August 1983, Supplement No. 6 dated November 1983, and Supplement No. 7 dated December 1983; (4) the Final Safety Analysis Report and amendments thereto; (5) the Environmental Report and supplements thereto; (6) the Final Environmental Statement dated November 1978 and the Addendum to the Final Environmental Statement dated May 1981; and (7) NRC Flood Plain Review of La Salle Nuclear Plant Site dated January 29, 1981

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, DC 20555, and the Public Library of Illinois Valley Community College, Rural Route No. 1, Oglesby, Illinois 61348. A copy of Facility Operating License No. NPF-18 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing. Copies of the Safety Evaluation Report and its Supplements 1, 2, 3, 4, 5, 6 and 7 (NUREG-0519) may be purchased at current rates from the National Technical Information Service, Department of

Commerce, 5285 Port Royal Road, Springfield, Virginia 22161, and through the NRC GPU sales program by writing to the U. S. Nuclear Regulatory Commission, Attention: Sales Manager, Washington, DC 20555. GPU deposit account holders can call (301) 492-9530.

Dated at Bethesda, Maryland, this 16<sup>th</sup> day of December 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

*Original signed by:*

A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing

OFFICE	DL:LB#2/LA	DL:LB#2/BC					
SURNAME	EH:son:kw...	ASchwencer					
DATE	12/12/83	12/12/83					



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket Nos. 50-373  
50-374

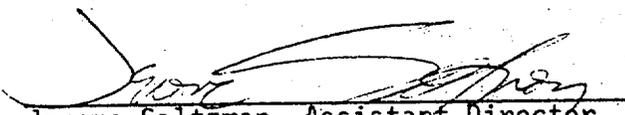
Amendment to Indemnity Agreement No. B-84  
Amendment No. 4

Effective Dec. 16, 1983 Indemnity Agreement No. B-84 between Commonwealth Edison Company and the Nuclear Regulatory Commission, dated September 25, 1978, as amended, is hereby further amended as follows:

Item 3 of the Attachment to the indemnity agreement is deleted in its entirety and the following substituted therefor:

- | Item 3   | - | License number or numbers   |
|----------|---|---|
| SNM-1802 |   | (From 12:01 a.m., September 25, 1978 to 12 midnight, April 16, 1982 inclusive)    |
| SNM-1833 |   | (From 12:01 a.m., September 25, 1978 to 12 midnight, December 15, 1983 inclusive) |
| NPF-11   |   | (From 12:01 a.m., April 17, 1982)   |
| NPF-18   |   | (From 12:01 a.m., Dec. 16, 1983 )   |

FOR THE NUCLEAR REGULATORY COMMISSION

  
Jerome Saltzman, Assistant Director  
State and Licensee Relations  
Office of State Programs

Accepted \_\_\_\_\_, 1983

By \_\_\_\_\_  
COMMONWEALTH EDISON COMPANY

*Handwritten:* ~~55PE~~ 8401060534



UNITED STATES  
 NUCLEAR REGULATORY COMMISSION  
 WASHINGTON, D.C. 20555  
 December 16, 1983

DISTRIBUTION:  
 Docket 50-374  
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Docket No. 50-374

Docketing and Service Section  
 Office of the Secretary of the Commission

SUBJECT: **NOTICE OF ISSUANCE OF FACILITY OPERATING LICENSE FOR LA SALLE COUNTY STATION, UNITS NO. 2 -- NPF-18**

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies ( 6 ) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).  
 NPF-18 for La Salle County Station, Unit No. 2
- Other: \_\_\_\_\_

Enclosure:  
 As Stated

Office of Nuclear Reactor Regulation

OFFICE →	DL:LB#2				
SURNAME →	EGHy1ton				
DATE →	12/16/83				