

June 10, 1998

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M96266 AND M96267)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 127 to Facility Operating License No. NPF-11 and Amendment No. 112 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2, respectively. The amendments are in response to your application dated July 15, 1996, as supplemented on June 19, 1997, and February 2, 1998.

The amendments relocate certain requirements related to fire protection from the Technical Specifications (TS) to the Updated Final Safety Analysis Report. The TS sections to be relocated are: 3/4.3.7.9, Fire Detection Instrumentation; 3/4.7.5, Fire Suppression Systems; 3/4.7.6, Fire Rated Assemblies; and 6.1.C.4, Fire Brigade Staffing. The amendments also replace License Condition 2.C.(25) for Unit 1 and License Condition 2.C.(15) for Unit 2. These amendments are consistent with the guidance of NRC Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," and GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications."

The July 15, 1996, submittal also withdrew a proposed amendment application related to fire protection valves dated May 19, 1995. The Notice of Withdrawal was published in the Federal Register on January 30, 1997 (62 FR 4553).

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

- Enclosures: 1. Amendment No. 127 to NPF-11
2. Amendment No. 112 to NPF-18
3. Safety Evaluation

cc w/encl: see next page

DISTRIBUTION:

Docket File PUBLIC PDIII-2 r/f E. Adensam, (EGA1)
S. Richards C. Moore D. Skay L. Rossbach
L. Marsh, (LBM) G. Hill (4), T5C3 OGC, O15B18 ACRS, T2E26
T. Harris, (SE only) D. Hills, (DEH), RIII G. Grant, (GEG), RIII W. Beckner, O13H15

1/1
DF01

*See Previous Concurrence

**concurrent on 06/10/98 re: deletion of Attachment 1, sections E and F of License No. NPF-18.

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DATE	06/10/98		06/10/98		05/04/98		5/29/98--6/10/98**		06/10/98	

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June 10, 1998

Mr. Oliver D. Kingsley, President
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/s/
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Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

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*See Previous Concurrence

**concluded on 06/10/98 re: deletion of Attachment 1, sections E and F of License No. NPF-18.

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DATE	06/10/98		06/10/98		05/04/98	5/29/98-6/10/98**	06/10/98	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 10, 1998

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

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Sincerely,

A handwritten signature in cursive script that reads "Donna M. Skay".

Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

Enclosures: 1. Amendment No. 127 to NPF-11
 2. Amendment No. 112 to NPF-18
 3. Safety Evaluation

cc w/encl: see next page

O. Kingsley
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Units 1 and 2

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- 2 -

LaSalle County Station
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated July 15, 1996, as supplemented on June 19, 1997, and February 2, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraphs 2.C.(2) and 2.C.(25) of the Facility Operating License No. NPF-11 are hereby amended to read as follows:

License page 9 is provided, for convenience, for the composite license to reflect this change.

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PDR ADOCK 05000373
P PDR

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 127, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(25) Fire Protection Program

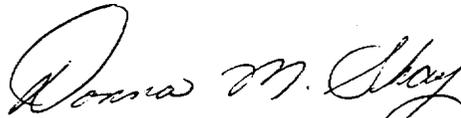
The licensee shall implement and maintain all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for LaSalle County Station, and as approved in NUREG-0519, "Safety Evaluation Report related to the operation of LaSalle County Station, Units 1 and 2," dated March 1981; Supplement 2 dated February 1982; Supplement 3 dated April 1982; Supplement 5 dated August 1983; Supplement 7 dated December 1983; Supplement 8 dated March 1984; and SERs for the following:

LaSalle Unit 1 License Amendment 1, dated June 18, 1982;
LaSalle Unit 1 License Amendment 18, dated August 8, 1984;
LaSalle Unit 1 License Amendment 23, dated May 22, 1985;
LaSalle Unit 1 License Amendment 44, dated June 20, 1986;
LaSalle Unit 1 License Amendment 127, dated June 10, 1998 ; and
NRC Evaluation of the Consequences of Postulated Failures of 1 Hour Fire Rated Darmatt KM-1 Fire Barrier under Seismic Loading at LaSalle County Station, dated March 29, 1996.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachments:

1. License page 9
2. Changes to the Technical Specifications

Date of Issuance: June 10, 1998

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INSTRUMENTATION

BASES

MONITORING INSTRUMENTATION (Continued)

3/4.3.7.5 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess important variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."

3/4.3.7.6 SOURCE RANGE MONITORS

The source range monitors provide the operator with information of the status of the neutron level in the core at very low power levels during startup and shutdown. At these power levels, reactivity additions should not be made without this flux level information available to the operator. When the intermediate range monitors are on scale adequate information is available without the SRMs and they can be retracted.

3/4.3.7.7 DELETED

3/4.3.7.8 DELETED

3/4.3.7.9 DELETED

PLANT SYSTEMS

BASES

3/4.7.4 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values. Sealed sources are classified into three groups according to their use, with surveillance requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism, i.e., sealed sources within radiation monitoring or boron measuring devices, are considered to be stored and need not be tested unless they are removed from the shielded mechanism.

3/4.7.5 DELETED

PLANT SYSTEMS

BASES

3/4.7.6 DELETED

3/4.7.7 AREA TEMPERATURE MONITORING

The area temperature limitations ensure that safety-related equipment will not be subjected to temperatures in excess of their environmental qualification temperatures. Exposure to excessive temperatures may degrade equipment and can cause loss of its OPERABILITY. The temperature limits include allowance for an instrument error of $\pm 7^{\circ}\text{F}$.

3/4.7.8 STRUCTURAL INTEGRITY OF CLASS 1 STRUCTURES

In order to assure that settlement does not exceed predicted and allowable settlement values, a program has been established to conduct a survey at the site. The allowable total differential settlement values are based on original settlement predictions. In establishing these tabulated values, an assumption is made that pipe and conduit connection have been designed to safely withstand the stresses which would develop due to total and differential settlement.

3/4 7.9 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the Reactor Coolant System and all other safety-related systems is maintained during and following a seismic or other event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

Snubbers are classified and grouped by design and manufacturer but not by size. For example, mechanical snubbers utilizing the same design features of the 2-kip, 10-kip, and 100-kip capacity manufactured by Company "A" are of the same type. The same design mechanical snubbers manufactured by Company "B" for the purpose of this Technical Specification would be of a different type, as would hydraulic snubbers from either manufacturer.

A list of individual snubbers with detailed information of snubbers location and size and of system affected shall be available at the plant in accordance with Section 50.71(c) of 10 CFR Part 50. The accessibility of each

1. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in OPERATIONAL CONDITION 1, 2 or 3, at least one licensed Senior Reactor Operator who has been designated by the Shift Supervisor to assume the control room direction responsibility shall be in the Control Room.
2. A radiation protection technician* shall be on site when fuel is in the reactor.
3. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
4. DELETED
5. The Independent Safety Engineering Group (ISEG) shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving unit safety. The ISEG shall be composed of at least three, dedicated, full-time engineers of multi-disciplines located on site and shall be augmented on a part-time basis by personnel from other parts of the Commonwealth Edison Company organization to provide expertise not represented in the group. The ISEG shall be responsible for maintaining surveillance of unit activities to provide independent verification# that these activities are performed correctly and that human errors are reduced as much as practical. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety to the Site Quality Verification Director and the Station Manager.
6. The Station Control Room Engineer (SCRE) may serve as the Shift Technical Advisor (STA) during abnormal operating and accident conditions. During these conditions, the SCRE or other on duty STA shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit.

* The radiation protection technician position may be less than the minimum requirement for a period of time not to exceed two hours in order to accommodate unexpected absence provided immediate action is taken to fill the required position.

Not responsible for sign-off feature.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated July 15, 1996, as supplemented on June 19, 1997, and February 2, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes the Technical Specifications as indicated in the enclosure to this license amendment and paragraphs 2.C.(2) and 2.C.(15) of the Facility Operating License No. NPF-18¹ are hereby amended to read as follows:

¹License pages 6 and 7 are provided, for convenience, for the composite license to reflect this change.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 112, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(15) Fire Protection Program

The licensee shall implement and maintain all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for LaSalle County Station, and as approved in NUREG-0519, "Safety Evaluation Report related to the operation of LaSalle County Station, Units 1 and 2," dated March 1981; Supplement 2 dated February 1982; Supplement 3 dated April 1982; Supplement 5 dated August 1983; Supplement 7 dated December 1983; Supplement 8 dated March 1984; and SERs for the following:

LaSalle Unit 2 License Amendment 11, dated May 22, 1985;
LaSalle Unit 2 License Amendment 14, dated October 2, 1985;
LaSalle Unit 2 License Amendment 112, dated June 10, 1998; and
NRC Evaluation of the Consequences of Postulated Failures of 1 Hour Fire Rated Darmatt KM-1 Fire Barrier under Seismic Loading at LaSalle County Station, dated March 29, 1996.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. Sections E and F (pages 2 and 3) of Attachment 1 to this license are deleted.
4. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachments:

1. License pages 6 and 7
2. Changes to the Technical Specifications

Date of Issuance: June 10, 1998

(1) battery current (ammeter-charge/discharge), (2) battery charger output voltage (voltmeter), (3) battery charger output current (ammeter), (4) battery high discharge rate alarm, and (5) battery charger trouble alarm. In the interim, the licensee shall implement approved procedures to monitor battery current, battery charger output voltage, and battery charger output current at the local panels at least once per eight-hour shift.

(14) Control of Heavy Loads (Section 9.1, SSER #1, SER #5)

Prior to startup after the first refueling, the licensee shall have made commitments acceptable to the NRC regarding the guidelines of Section 5.1.2 through 5.1.6 of NUREG-0612.

(15) Fire Protection Program

The licensee shall implement and maintain all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for LaSalle County Station, and as approved in NUREG-0519, "Safety Evaluation Report related to the operation of LaSalle County Station, Units 1 and 2," dated March 1981; Supplement 2 dated February 1982; Supplement 3 dated April 1982; Supplement 5 dated August 1983; Supplement 7 dated December 1983; Supplement 8 dated March 1984; and SERs for the following:

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The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(16) Industrial Security (Section 13.6, SER, SSER #3, SSER #5)

CECo shall fully implement and maintain in effect all provisions of the Commission approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled "LaSalle County Nuclear Station Security Plan," with revisions submitted through June 1, 1988, "LaSalle County Nuclear Power Station Security Personnel Training and Qualification Plan," with revisions submitted through June 13, 1986, and "LaSalle County Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through February 16, 1984. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

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INSTRUMENTATION

BASES

MONITORING INSTRUMENTATION (Continued)

3/4.3.7.5 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess important variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."

3/4.3.7.6 SOURCE RANGE MONITORS

The source range monitors provide the operator with information of the status of the neutron level in the core at very low power levels during startup and shutdown. At these power levels, reactivity additions should not be made without this flux level information available to the operator. When the intermediate range monitors are on scale adequate information is available without the SRMs and they can be retracted.

3/4.3.7.7 DELETED

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3/4.3.7.9 DELETED

3/4.3.7.10 DELETED

PLANT SYSTEMS

BASES

3/4.7.4 SEALED SOURCE CONTAMINATION

The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. This limitation will ensure that leakage from byproduct, source, and special nuclear material sources will not exceed allowable intake values. Sealed sources are classified into three groups according to their use, with surveillance requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism, i.e., sealed sources within radiation monitoring or boron measuring devices, are considered to be stored and need not be tested unless they are removed from the shielded mechanism.

3/4.7.5 DELETED

PLANT SYSTEMS

BASES

3/4.7.6 DELETED

3/4.7.7 AREA TEMPERATURE MONITORING

The area temperature limitations ensure that safety-related equipment will not be subjected to temperatures in excess of their environmental qualification temperatures. Exposure to excessive temperatures may degrade equipment and can cause loss of its OPERABILITY. The temperature limits include allowance for an instrument error of $\pm 7^{\circ}\text{F}$.

3/4.7.8 STRUCTURAL INTEGRITY OF CLASS 1 STRUCTURES

In order to assure that settlement does not exceed predicted and allowable settlement values, a program has been established to conduct a survey at the site. The allowable total differential settlement values are based on original settlement predictions. In establishing these tabulated values, an assumption is made that pipe and conduit connections have been designed to safely withstand the stresses which would develop due to total and differential settlement.

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1. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in OPERATIONAL CONDITION 1, 2 or 3, at least one licensed Senior Reactor Operator who has been designated by the Shift Supervisor to assume the control room direction responsibility shall be in the Control Room.
2. A radiation protection technician* shall be on site when fuel is in the reactor.
3. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
4. DELETED
5. The Independent Safety Engineering Group (ISEG) shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving unit safety. The ISEG shall be composed of at least three, dedicated, full-time engineers of multi-disciplines located on site and shall be augmented on a part-time basis by personnel from other parts of the Commonwealth Edison Company organization to provide expertise not represented in the group. The ISEG shall be responsible for maintaining surveillance of unit activities to provide independent verification# that these activities are performed correctly and that human errors are reduced as much as practical. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety to the Site Quality Verification Director and the Station Manager.
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* The radiation protection technician position may be less than the minimum requirement for a period of time not to exceed two hours in order to accommodate unexpected absence provided immediate action is taken to fill the required position.

Not responsible for sign-off feature.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. NPF-18
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

Section 50.48, "Fire protection," of Part 50 of Title 10 of the Code of Federal Regulations (10 CFR Part 50) requires that each operating nuclear power plant have a fire protection plan that satisfies General Design Criterion 3 (GDC 3), "Fire protection," of Appendix A to 10 CFR Part 50. The fire protection plan must describe the overall fire protection program for the facility, outline the plans for fire protection, fire detection, and fire suppression capability, and limitations of fire damage. The program must also describe specific features necessary to implement the program, such as administrative controls and personnel requirements for fire prevention and manual fire suppression activities, automatic and manually operated fire detection and suppression systems, and the means to limit fire damage to structures, systems, or components important to safety so that the capability to safely shut down the plant is ensured.

By letter dated July 15, 1996, as supplemented on June 19, 1997, and February 2, 1998, the Commonwealth Edison Company (ComEd, the licensee) requested amendments to the Technical Specifications (TSs) appended to Facility Operating License Nos. NPF-11 and NPF-18 for the LaSalle County Station, Units 1 and 2. The proposed amendments relocate certain fire protection requirements from the TSs to the Fire Protection Program in accordance with the guidance provided in Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," and GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications." Specifically, the licensee proposed to relocate the fire protection requirements of TS Section 3/4.3.7.9 (Instrumentation - Fire Detection Instrumentation), TS Section 3/4.7.5 (Plant Systems - Fire Suppression Systems), TS Section 3/4.7.6 (Plant Systems - Fire Rated Assemblies), and TS Section 6.1.C.4 (Organization), from the TSs to the Fire Protection Program (consisting of the Updated Final Safety Analysis Report (UFSAR), station procedures, and station Administrative Technical Requirements). In addition, the licensee proposed to revise the operating licenses to include the NRC's standard fire protection license condition as specified in GL 86-10. The June 19, 1997, and February 2, 1998, supplements clarified the license conditions by providing specific approval dates for previous fire protection safety evaluations. This information was within the scope of the original application and did not change the staff's initial proposed no significant hazards consideration determination.

GL 86-10 and GL 88-12 referred to removing fire protection requirements from TSs. License amendments that relocate fire protection requirements to the Final Safety Analysis Report (FSAR) in accordance with GL 86-10 and GL 88-12 do not revise the requirements for fire protection operability, testing, or inspections. Such amendments simply replace the fire

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protection TS sections with the standard fire protection license condition. The license condition implements and maintains the NRC-approved fire protection program, including the fire protection requirements previously specified in the TSs, in accordance with 10 CFR 50.48. Therefore, such amendments, including the ones proposed by the licensee, are administrative in nature and have no effect on public health and safety.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (Act) requires applicants for nuclear power plant operating licenses to state TSs to be included as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

The Commission has provided guidance for the contents of TSs in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (Final Policy Statement), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies Section 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TSs to licensee-controlled documents, consistent with the standard enunciated in *Portland General Electric Co. (Trojan Nuclear Plant)*, ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety." The criteria set forth in the Final Policy Statement have been incorporated into 10 CFR 50.36 (60 FR 36953).

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Commission undertook a number of actions to ensure that improvements were implemented in the fire protection programs for all power reactor facilities. Because of the extensive modification of fire protection programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, licensees were requested by GL 86-10 to incorporate their final NRC-approved fire protection programs in their FSARs. In this manner, the fire protection program, including the systems, certain administrative and technical controls, the organization, and other plant features associated with fire protection, would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the fire protection program as described in the FSAR, should be used to ensure uniform enforcement of the fire protection requirements. Finally, the Commission stated that, with the required actions, licensees may request an amendment to delete the fire protection TSs that would now be unnecessary. Subsequently, the NRC issued GL 88-12 to give guidance for the preparation of the license amendment request to implement GL 86-10.

3.0 PROPOSED CHANGES

The specific changes proposed by the licensee are as follows:

1. Replace License Condition 2.C.(25) for Unit 1 and License Condition 2.C.(15) for Unit 2 with the standard fire protection license condition as stated in GL 86-10. In addition, delete Sections E and F of Attachment 1 to Facility Operating License NPF-18 which specified items to be completed for the Fire Protection Program prior to exceeding 5 percent of rated full power and prior to startup from the first refueling outage.
2. Delete TS 3/4.3.7.9 (INSTRUMENTATION - FIRE DETECTION INSTRUMENTATION) and relocate the limiting conditions for operation (LCOs), surveillance requirements (SRs), and the associated bases to Administrative Technical Requirements (ATRs). The ATRs will be incorporated by reference into the LaSalle UFSAR.
3. Delete TS 3/4.7.5 (PLANT SYSTEMS - FIRE SUPPRESSION SYSTEMS) and relocate the LCOs, SRs, and the associated bases to an ATR which will be incorporated by reference into the LaSalle UFSAR.
4. Delete TS 3/4.7.6 (PLANT SYSTEMS - FIRE RATED ASSEMBLIES) and relocate the LCOs, SRs, and the associated bases to an ATR which will be incorporated by reference into the LaSalle UFSAR.
5. Delete TS 6.1.C.4, fire brigade staffing, and relocate the requirements to an ATR which will be incorporated by reference into the LaSalle UFSAR. Also, delete the words "and Fire Brigade composition" from the footnote (*) to TSs 6.1.C.2 and 6.1.C.4.

4.0 EVALUATION

The NRC staff reviewed the license amendment request against the guidance provided in GLs 86-10 and 88-12. GL 86-10 stated that the licensee should incorporate its NRC-approved fire protection program in its UFSAR and specified a standard fire protection license condition that the licensee may use to replace current license conditions once the fire protection program is incorporated into the UFSAR.

GL 88-12 addressed the elements a licensee should include in a license amendment request to remove fire protection requirements from the plant TSs. These elements are (1) the NRC-approved fire protection program must be incorporated into the UFSAR; (2) the LCOs and SRs associated with fire detection systems, fire suppression systems, fire barriers, and the administrative controls that address fire brigade staffing would be relocated from the TSs (the existing administrative controls associated with fire protection audits and specifications related to the capability for safe shutdown following a fire would be retained); (3) all operational conditions, remedial actions, and test requirements presently included in the TSs for these systems, as well as the fire brigade staffing requirements, shall be incorporated into the fire protection program; (4) the standard fire protection license condition specified in GL 86-10 must be included in the facility operating license; (5) the Administrative Controls section of the TSs shall provide that the Onsite Review Group shall be given responsibility for the review of the program and implementing procedures and for the submittal of recommended changes to the Offsite or Corporate Review Group; and (6) the Administrative Controls section of the TSs shall provide

that implementation of the program shall be added to the list of elements for which written procedures shall be established, implemented, and maintained.

The staff's analysis of the license amendment request is set forth below.

1. LaSalle's Fire Protection System is presented in UFSAR Section 9.5.1 and the Fire Hazards Analysis, including the Safe Shutdown Analysis, is incorporated in Appendix H of the UFSAR. Therefore, the licensee has satisfied Element 1 of GL 88-12.
2. In its submittal dated July 15, 1996, the licensee stated that it will incorporate the current TS LCOs and SRs for the fire detection systems, fire suppression systems, fire barriers, and the administrative controls that address fire brigade staffing into a LaSalle ATR document. In its letter dated February 2, 1998, the licensee stated that the above mentioned TS requirements will be included in the Fire Protection Program by reference to the ATR. Therefore, the licensee has satisfied Elements 2 and 3 of GL 88-12.
3. The licensee proposed to incorporate the standard fire protection license condition specified in GL 86-10 for LaSalle. The licensee has, therefore, satisfied Element 4 of GL 88-12.
4. With respect to Element 5 of GL 88-12, the licensee's existing or proposed TSs do not include a requirement in the Administrative Controls section that the Onsite Review Group have responsibility for review of the fire protection program and implementing procedures, and for the submittal of recommended changes to the Offsite or Corporate Review Group. All TSs requirements related to responsibilities of the Onsite Review Group, including its responsibility for review of the fire protection program, were relocated from the TSs to the ComEd Quality Assurance Topical Report, CE-1-A, by a license amendment dated October 20, 1995. The licensee implements its Quality Assurance (QA) program in accordance with the requirements of 10 CFR Part 50, Appendix B. Changes to the QA program that constitute a reduction in commitment require NRC review and approval pursuant to 10 CFR 50.54(a). The staff has determined that in light of the regulatory controls that exist under 10 CFR 50.54 to adequately control future modifications to the QA program, the underlying objective of Element 5 of GL 88-12 has been met.
5. The TSs currently contain a requirement (TS 6.2.A.g) that written procedures be established, implemented, and maintained for implementation of the fire protection program. Therefore, the licensee has satisfied Element 6 of GL 88-12.

In summary, the licensee has proposed to incorporate the existing TS fire protection requirements as stated above into the fire protection program which is, by reference, incorporated into the UFSAR. This conforms to staff guidance in GL 86-10, "Implementation of Fire Protection Requirements," and GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications," for removing unnecessary fire protection TSs in four major areas: fire detection systems, fire suppression systems, fire barriers, and fire brigade staffing requirements. The proposed TSs are consistent with NUREG-1434, "Standard Technical Specifications, General Electric Plants." In addition, incorporating these requirements into the UFSAR is consistent with 10 CFR 50.36, as amended, because these TSs do not impact reactor operations, do not identify a parameter which is an initial condition assumption for a design-basis accident or transient, do not identify a significant abnormal degradation of the reactor coolant pressure boundary, and do not provide any mitigation of a design-basis event.

The fire protection plan required by 10 CFR 50.48, as implemented and maintained by the fire protection license condition, provides reasonable assurance that fires will not give rise to an immediate threat to public health and safety. Although there are aspects of the fire detection and mitigation functions that have been determined to be risk significant, such that Criterion 4 of 10 CFR 50.36 would otherwise seem to apply, the minimum requirements for those functions were established in General Design Criterion 3 and 10 CFR 50.48, and further controls are not necessary since the licensee must comply with these minimum requirements regardless of whether they are restated in the TSs or not.

The licensee's fire protection program is required by 10 CFR 50.48, and any changes to that program are governed by 10 CFR 50.48 and license conditions 2.C.(25) (Unit 1) and 2.C.(15) (Unit 2). Therefore, the requirements relocated to the UFSAR may be controlled in accordance with 10 CFR 50.59.

These relocated requirements relating to fire protection features are not required to be in the TSs under 10 CFR 50.36 or other regulations, or by Section 182a of the Atomic Energy Act, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to public health and safety. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.48 and 10 CFR 50.59 to address future changes to these requirements. Accordingly, the staff has concluded that these requirements may be relocated from the TSs to the licensee's UFSAR, and that the proposed amendments are acceptable.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change administrative requirements, or change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change SRs. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 50340). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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