

50-373/374



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 19, 1997

Ms. Irene Johnson, Acting Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: CORRECTION TO AMENDMENTS - LASALLE COUNTY STATION, UNITS 1 AND 2
(TAC NOS. M94063, M94064, M97824 AND M97825)

Dear Ms. Johnson:

On March 11, 1996, the NRC issued Amendment Nos. 110 and 95 to the licenses for LaSalle, Units 1 and 2, that included a revision to page 6-20a of the Technical Specifications (TS). The page that was issued with this amendment for Unit 2 was in error as it did not include a change that had been made to this page on October 20, 1995. The change to Specification 6.3.b was correctly included in the Unit 1 TS, but was not included in the page issued for the Unit 2 TS. Specification 6.3.b states the requirements for review of reportable events. The Unit 2 TS refer to Specification 6.1.G.2.c(1), but should instead require review by the Onsite Review and Investigative Function. Enclosed is the corrected page 6-20a of the Unit 2 TS.

On July 16, 1997, the NRC issued Amendment Nos. 118 and 103 to the licenses for LaSalle, Units 1 and 2, TS. Typographical errors were included on the revised pages 3/4 4-7 and 3/4 6-18 for Unit 1, and 3/4 4-8, 3/4 5-3a, and 3/4 6-21 for Unit 2. In addition, index page XIV for both units should have been revised to indicate that a page change had occurred for Bases Section 3/4.5.3. The corrected pages are enclosed.

During a telephone call with the license on August 19, 1997, the licensee requested clarification of the Safety Evaluation (SE) issued with Amendment Nos. 118 and 103. Section 2.1 of the SE provides justification for relocation of the surveillance requirements for the control rod drive accumulator leak detectors and pressure detectors. The SE states that the automatic control rod drive charging water header low pressure scram initiates before any accumulator loses its full capability to insert the control rods. The licensee pointed out that this automatic scram is only active during modes 2 and 5 while the CRD scram accumulators are required to be operable in modes 1, 2, and 5. This clarification does not change the staff's conclusion because there are other factors that provide sufficient justification for removing this surveillance from the TS. Section 2.5 of this SE evaluates the removal of the TS requirements for the affected instruments against the criteria of 10 CFR 50.36. The discussion under Criterion 2 should state that if a process variable can not be determined to be within its TS limit, then the associated TS action requirements must be entered. The staff recognizes

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I. Johnson

- 2 -

that if the associated instrument is inoperable but the process variable can still be monitored by other means, then the action statement would not need to be entered.

Sincerely,

~~Original Signed By:~~

Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosures: As stated

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Project Directorate III-2
Division of Reactor Projects - III/IV
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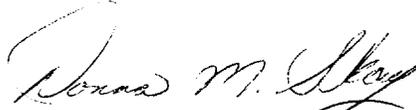
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Project Directorate III-2
Division of Reactor Projects - III/IV
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INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.5 EMERGENCY CORE COOLING SYSTEMS</u>	
3/4.5.1 and 3/4.5.2 ECCS-OPERATING and SHUTDOWN.....	B 3/4 5-1
3/4.5.3 SUPPRESSION CHAMBER.....	B 3/4 5-4
<u>3/4.6 CONTAINMENT SYSTEMS</u>	
3/4.6.1 PRIMARY CONTAINMENT	
Primary Containment Integrity.....	B 3/4 6-1
Primary Containment Air Locks.....	B 3/4 6-1
Drywell and Suppression Chamber Internal Pressure..	B 3/4 6-2
Drywell Average Air Temperature.....	B 3/4 6-2
Drywell and Suppression Chamber Purge System.....	B 3/4 6-2
3/4.6.2 DEPRESSURIZATION SYSTEMS.....	B 3/4 6-3
3/4.6.3 PRIMARY CONTAINMENT ISOLATION VALVES.....	B 3/4 6-4
3/4.6.4 VACUUM RELIEF.....	B 3/4 6-4
3/4.6.5 SECONDARY CONTAINMENT.....	B 3/4 6-5
3/4.6.6 PRIMARY CONTAINMENT ATMOSPHERE CONTROL.....	B 3/4 6-5

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE.
- b. 5 gpm UNIDENTIFIED LEAKAGE.
- c. 25 gpm total leakage averaged over any 24 hour period.
- d. 1 gpm leakage at a reactor coolant system pressure at 1000 ± 50 psig from any reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1.
- e. 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24 hour period.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With any reactor coolant system leakage greater than the limits in b and/or c, above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With any reactor coolant system pressure isolation valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed valves, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- d. With one or more high/low pressure interface valve leakage pressure monitors inoperable, restore the inoperable monitor(s) to OPERABLE status within 7 days or verify the pressure to be less than the alarm setpoint at least once per 12 hours by local indication; restore the inoperable monitor(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 12 hours.
- e. With any reactor coolant system leakage greater than the limit in e above, identify the source of leakage within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. Deleted.
- d. By conducting drywell-to-suppression chamber bypass leak tests at least once per 18 months at an initial differential pressure of 1.5 psi and verifying that the A/\sqrt{k} calculated from the measured leakage is within the specified limit.

If any 1.5 psi leak test results in a calculated A/\sqrt{k} >20% of the specified limit, then the test schedule for subsequent tests shall be reviewed by the Commission.

If two consecutive 1.5 psi leak tests result in a calculated A/\sqrt{k} greater than the specified limit, then:

1. A 1.5 psi leak test shall be performed at least once per 9 months until two consecutive 1.5 psi leak tests result in the calculated A/\sqrt{k} within the specified limits, and
2. A 5 psi leak test, performed with the second consecutive successful 1.5 psi leak test, results in a calculated A/\sqrt{k} within the specified limit, after which the above schedule of once per 18 months for only 1.5 psi leak tests may be resumed.

If any required 5 psi leak test results in a calculated A/\sqrt{k} greater than the specified limit, then the test schedule for subsequent tests shall be reviewed by the Commission.

If two consecutive 5 psi leak tests result in a calculated A/\sqrt{k} greater than the specified limit, then a 5 psi leak test shall be performed at least once per 9 months until two consecutive 5 psi leak tests result in a calculated A/\sqrt{k} within the specified limit, after which the above schedule of once per 18 months for only 1.5 psi leak tests may be resumed.

INDEX

BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.5 EMERGENCY CORE COOLING SYSTEMS</u>	
3/4.5.1 and 3/4.5.2 ECCS-OPERATING and SHUTDOWN.....	B 3/4 5-1
3/4.5.3 SUPPRESSION CHAMBER.....	B 3/4 5-4
<u>3/4.6 CONTAINMENT SYSTEMS</u>	
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Primary Containment Integrity.....	B 3/4 6-1
Primary Containment Air Locks.....	B 3/4 6-2
Drywell and Suppression Chamber Internal Pressure..	B 3/4 6-2a
Drywell Average Air Temperature.....	B 3/4 6-2a
Drywell and Suppression Chamber Purge System.....	B 3/4 6-2a
3/4.6.2 DEPRESSURIZATION SYSTEMS.....	B 3/4 6-3
3/4.6.3 PRIMARY CONTAINMENT ISOLATION VALVES.....	B 3/4 6-4
3/4.6.4 VACUUM RELIEF.....	B 3/4 6-4
3/4.6.5 SECONDARY CONTAINMENT.....	B 3/4 6-5
3/4.6.6 PRIMARY CONTAINMENT ATMOSPHERE CONTROL.....	B 3/4 6-5

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE.
- b. 5 gpm UNIDENTIFIED LEAKAGE.
- c. 25 gpm total leakage averaged over any 24 hour period.
- d. 1 gpm leakage at a reactor coolant system pressure at 1000 ± 50 psig from any reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1.
- e. 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24 hour period.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With any reactor coolant system leakage greater than the limits in b and/or c, above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With any reactor coolant system pressure isolation valve leakage greater than the above limits, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed valves, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- d. With one or more high/low pressure interface valve leakage pressure monitors inoperable, restore the inoperable monitor(s) to OPERABLE status within 7 days or verify the pressure to be less than the alarm setpoint at least once per 12 hours by local indication; restore the inoperable monitor(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 12 hours.
- e. With any reactor coolant system leakage greater than the limit in e, above, identify the source of leakage within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

- i. In the event an ECCS system is actuated and injects water into the Reactor Coolant System, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.6.C within 90 days describing the circumstances of the actuation and the total accumulated actuation cycles to date. The current value of the usage factor for each affected safety injection nozzle shall be provided in this Special Report whenever its value exceeds 0.70.
- j. With one or more ECCS corner room watertight doors inoperable, restore all the inoperable ECCS corner room watertight doors to OPERABLE status within 14 days, otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- k. With ADS accumulator backup compressed gas system bottle pressure less than 500 psig, restore ADS accumulator backup compressed gas system bottle pressure to greater than 500 psig within 72 hours or declare the associated ADS valves inoperable, and follow Action e of this specification.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

c. Deleted.

d. By conducting drywell-to-suppression chamber bypass leak tests at least once per 18 months at an initial differential pressure of 1.5 psi and verifying that the A/\sqrt{k} calculated from the measured leakage is within the specified limit.

If any 1.5 psi leak test results in a calculated A/\sqrt{k} >20% of the specified limit, then the test schedule for subsequent tests shall be reviewed by the Commission.

If two consecutive 1.5 psi leak tests result in a calculated A/\sqrt{k} greater than the specified limit, then:

1. A 1.5 psi leak test shall be performed at least once per 9 months until two consecutive 1.5 psi leak tests result in the calculated A/\sqrt{k} within the specified limits, and
2. A 5 psi leak test, performed with the second consecutive successful 1.5 psi leak test, results in a calculated A/\sqrt{k} within the specified limit, after which the above schedule of once per 18 months for only 1.5 psi leak tests may be resumed.

If any required 5 psi leak test results in a calculated A/\sqrt{k} greater than the specified limit, then the test schedule for subsequent tests shall be reviewed by the Commission.

If two consecutive 5 psi leak tests result in a calculated A/\sqrt{k} greater than the specified limit, then a 5 psi leak test shall be performed at least once per 9 months until two consecutive 5 psi leak tests result in a calculated A/\sqrt{k} within the specified limit, after which the above schedule of once per 18 months for only 1.5 psi leak tests may be resumed.

ADMINISTRATIVE CONTROLS

PLANT OPERATING PROCEDURES AND PROGRAMS (Continued)

the Initial Structural Integrity Tests were not within 2 years of each other.

The Onsite Review and Investigative Function shall be responsible for reviewing and approving changes to the Inservice Inspection Program for Post Tensioning Tendons.

The provisions of 4.0.2 and 4.0.3 are applicable to the Tendon Surveillance Program inspection frequencies.

6.2.F.7 Primary Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the primary containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Testing Program," dated September 1995.

The peak calculated primary containment internal pressure for the design basis loss of coolant accident, P_a , is 39.6 psig.

The maximum allowable primary containment leakage rate, L_a , at P_a , is 0.635% of primary containment air weight per day.

Leakage rate acceptance criteria are:

- a. Primary containment overall leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the combined Type B and Type C tests, and $\leq 0.75 L_a$ for Type A tests.
- b. Air lock testing acceptance criteria are:
 - 1) Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 - 2) For each door, the seal leakage rate is ≤ 5 scf per hour when the gap between the door seals is pressurized to ≥ 10 psig.

The provisions of specification 4.0.2 do not apply to the test frequencies specified in the Primary Containment Leakage Rate Testing Program.

The provisions of specification 4.0.3 are applicable to the Primary Containment Leakage Rate Testing Program.

6.3 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE EVENT IN PLANT OPERATION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a Licensee Event Report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the Onsite Review and Investigative Function.