October 9, 1997

Ms. Irene Johnson, Acting Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M99112 AND M99113)

Dear Ms. Johnson:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 121 to Facility Operating License No. NPF-11 and Amendment No.106 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2, respectively. The amendments are in response to your application dated July 1, 1997.

The amendments revise Technical Specification Table 3.3.7.1-1, "Radiation Monitoring Instrumentation," to require two channels to be operable per trip system as opposed to two per intake. This change reflects a modification to the design of the instrumentation logic to satisfy single failure requirements. The amendments also revise the associated action statement to clarify system logic wording.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:
Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

Enclosures: 1. Amendment No. 121 to NPF-11
2. Amendment No. 106 to NPF-18
3. Safety Evaluation

cc w/encl: see next page

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OFFICE	PM:PDIII-2	LA:PDIII-2	D:PDIII-2	OGC	BC:HICB
NAME	DSKAY	CMOORE	RCAPRA		JWERMIEL
DATE	09/12/97	09/12/97	09/9/97	09/16/97	09/16/97

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 9, 1997

Ms. Irene Johnson, Acting Manager
Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

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Dear Ms. Johnson:

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The amendments revise Technical Specification Table 3.3.7.1-1, "Radiation Monitoring Instrumentation," to require two channels to be operable per trip system as opposed to two per intake. This change reflects a modification to the design of the instrumentation logic to satisfy single failure requirements. The amendments also revise the associated action statement to clarify system logic wording.

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Sincerely,

Donna M. Skay, Project Manager
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Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

Enclosures: 1. Amendment No. 121 to NPF-11
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3. Safety Evaluation

cc w/encl: see next page

I. Johnson
Commonwealth Edison Company

cc:

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Chairman
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LaSalle County Courthouse
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated July 1, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 121 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 9, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 3-57	3/4 3-57
3/4 3-58	3/4 3-58
B 3/4 3-4a	B 3/4 3-4a

TABLE 3.3.7.1-1

RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE CONDITIONS	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
a. Main Control Room Atmospheric Control System Radiation Monitoring Subsystem	2 per trip system/train (intake)**	1,2,3,5 and *	3.5 mR/hr	0.1 to 10,000 mR/hr	70

NOTES

*When irradiated fuel is being handled in the secondary containment.
**A channel may be placed in an inoperable status for up to 6 hours for required surveillance testing without placing the Trip System in the tripped condition, provided at least one other operable channel in the same Trip System is monitoring that Trip Function.

TABLE 3.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION

ACTION

ACTION 70 -

a. With the number of OPERABLE channels per trip system one less than the minimum required, place the inoperable channel in the tripped condition within one hour.

b. With both channels in a trip system inoperable, declare the trip system inoperable. Restore the inoperable trip system to OPERABLE status within 7 days, or, within the next 6 hours, initiate and maintain operation of the control room emergency filtration system in the pressurization mode of operation.

c. Otherwise, initiate and maintain operation of the control room emergency filtration system in the pressurization mode of operation within 1 hour.

3/4.3.7.1 RADIATION MONITORING INSTRUMENTATION (Continued)

The Control Room and Auxiliary Electric Equipment Room (AEER) Emergency Filtration System (CREFS) consists of two trains. Each train has one outside air intake. The Main Control Room Atmospheric Control System (MCRACS) Radiation Monitoring System consists of two trains, one for each train of CREFS. Each MCRACS train contains four radiation monitors arranged in two trip systems. Each trip system contains two radiation monitors. Both radiation monitors in each trip system are required to be OPERABLE for that trip system to be OPERABLE.

Specified surveillance intervals and surveillance and maintenance outage times have been determined in accordance with GENE-770-06-1-A, "Bases for Changes to Surveillance Test Intervals and Allowed Out-Of-Service Times for Selected Instrumentation Technical Specifications," December 1992. When a channel is placed in an inoperable status solely for performance of required surveillances, entry into LCO and required ACTIONS may be delayed, provided the associated function maintains initiation capability.

3.4.3.7.2 DELETED

3/4.3.7.3 METEOROLOGICAL MONITORING INSTRUMENTATION

The OPERABILITY of the meteorological monitoring instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public. This instrumentation is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs," February 1972.

3/4.3.7.4 REMOTE SHUTDOWN MONITORING INSTRUMENTATION

The OPERABILITY of the remote shutdown monitoring instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT SHUTDOWN of the unit from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated July 1, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 106 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Donna M. Skay, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 9, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE	INSERT
3/4 3-58	3/4 3-58
B 3/4 3-4a	B 3/4 3-4a

TABLE 3.3.7.1-1

RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE CONDITIONS	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
a. Main Control Room Atmospheric Control System Radiation Monitoring Subsystem	2 per trip system/train (intake)**	1,2,3,5 and *	3.5 mR/hr	0.1 to 10,000 mR/hr	70

TABLE NOTATIONS

*When irradiated fuel is being handled in the secondary containment.
**A channel may be placed in an inoperable status for up to 6 hours for required surveillance testing without placing the Trip System in the tripped condition, provided at least one other operable channel in the same Trip System is monitoring that Trip Function.

ACTION STATEMENT

ACTION 70 -

a. With the number of OPERABLE channels per trip system one less than the minimum required, place the inoperable channel in the tripped condition within one hour.

b. With both channels in a trip system inoperable, declare the trip system inoperable. Restore the inoperable trip system to OPERABLE status within 7 days, or, within the next 6 hours, initiate and maintain operation of the control room emergency filtration system in the pressurization mode of operation.

c. Otherwise, initiate and maintain operation of the control room emergency filtration system in the pressurization mode of operation within 1 hour.

3/4.3.7.1 RADIATION MONITORING INSTRUMENTATION (Continued)

The Control Room and Auxiliary Electric Equipment Room (AEER) Emergency Filtration System (CREFS) consists of two trains. Each train has one outside air intake. The Main Control Room Atmospheric Control System (MCRACS) Radiation Monitoring System consists of two trains, one for each train of CREFS. Each MCRACS train contains four radiation monitors arranged in two trip systems. Each trip system contains two radiation monitors. Both radiation monitors in each trip system are required to be OPERABLE for that trip system to be OPERABLE.

Specified surveillance intervals and surveillance and maintenance outage times have been determined in accordance with GENE-770-06-1-A, "Bases for Changes to Surveillance Test Intervals and Allowed Out-Of-Service Times for Selected Instrumentation Technical Specifications," December 1992. When a channel is placed in an inoperable status solely for performance of required surveillances, entry into LCO and required ACTIONS may be delayed, provided the associated function maintains initiation capability.

3.4.3.7.2 DELETED

3/4.3.7.3 METEOROLOGICAL MONITORING INSTRUMENTATION

The OPERABILITY of the meteorological monitoring instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public. This instrumentation is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs," February 1972.

3/4.3.7.4 REMOTE SHUTDOWN MONITORING INSTRUMENTATION

The OPERABILITY of the remote shutdown monitoring instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT SHUTDOWN of the unit from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. NPF-11 AND

AMENDMENT NO. 106 TO FACILITY OPERATING LICENSE NO. NPF-18

COMMONWEALTH EDISON COMPANY

LASALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter dated July 1, 1997, Commonwealth Edison Company (ComEd, the licensee) requested changes to the Technical Specifications (TS) for the LaSalle County Station, Units 1 and 2. The proposed changes reflect modifications to the Control Room Ventilation Air Intake Radiation Monitoring System logic that are being made to restore separation and redundancy of the radiation monitoring trip systems.

The Main Control Room Atmospheric Control System (MCRACS) is designed to limit the exposure of control room personnel to less than the limits specified in General Design Criteria 19 of 10 CFR Part 50, Appendix A. The system consists of two 100 percent heating, ventilation, and air conditioning (HVAC) trains, each supplied through its own ventilation intake. The MCRACS radiation monitoring instrumentation continuously monitors outside air intake and, upon detection of high radiation, isolates the control room from the normal outside air by closing dampers and initiates the Emergency Makeup Mode of the Control Room Emergency Filtration System.

The MCRACS radiation monitoring instrumentation consists of four radiation monitors in each intake. In the current configuration, if any two monitors in an intake sense radiation above the setpoint of 3.5 mR/hr, the system will trip to initiate the protective features. Therefore, the current design is a two-out-of-four logic.

The licensee has determined that the current design does not ensure redundancy and separation because all four monitors in an intake are electrically connected. Therefore, a single fault in one radiation monitor could result in a loss of the automatic isolation function for that intake. The licensee has determined that the logic must be modified to meet single failure criteria. The revised design will consist of two independent trip systems per intake, each consisting of two monitor channels. The revised logic will require that both channels in a trip system indicate high radiation for the trip system to actuate. Actuation of either trip system will result in isolation of that intake. The revised logic will maintain the current two-out-of-four design

and will meet the single failure requirements as described in the Safety Evaluation Report.

2.0 EVALUATION

Current TS Table 3.3.7.1-1 requires two channels per intake to be operable at all times except when a unit is in cold shutdown or defueled. Due to the modification to the system design, and to clarify the system requirements, the licensee has proposed to modify the TS to require that two channels per trip system per train (intake) be operable.

The current trip logic configuration consists of four monitors divided into two trip systems. The licensee had implemented a Technical Specification Clarification in 1993 that defined a trip channel as consisting of two radiation monitors. Therefore, the current TS was interpreted to require all four monitors operable, but the TS wording is unclear. In the proposed modification, each radiation monitor is considered a channel. The proposed TS will require that two channels per trip system per train be operable and the Bases will clarify that each intake consists of two trip systems. The proposed TS requirement is consistent with the proposed design change which will require both channels in a trip system to actuate the protective actions, and is consistent with the intent of the current TS. The wording of the proposed TS will also clarify that both trains (intakes) are required to be operable to ensure operability of the system, consistent with the current TS. The proposed change is an enhancement of the current TS and is acceptable.

The current TS contains a footnote (**) which allows a channel to be placed in an inoperable status for up to 6 hours for the performance of required surveillance testing without placing the Trip System in the tripped condition, provided at least one other operable channel in the same Trip System is monitoring that Trip Function. This footnote allows a channel to be inoperable for up to 6 hours without placing the control room emergency filtration system in the pressurization mode. This footnote is based on surveillance intervals specified in General Electric Topical report GENE-770-06-1, "Bases for Changes to Surveillance Test Intervals and Allowed Outage Times for Selected Instrumentation Technical Specifications" and was approved by Amendment Nos. 104 and 90 to the LaSalle TS. This footnote uses the generic term "Trip System" as defined in GENE-770-06-1 to refer to the train of instrumentation logic which will actuate the protective function. Therefore, in the current and proposed TS, the Trip System referred to in this footnote is a train of MCRACS radiation monitoring instrumentation and the Trip Function is initiation of the automatic actions to close the intake dampers. The footnote requires that the train maintain automatic initiation capability (i.e., a channel may not be removed from service for 6 hours for surveillance testing if the redundant trip system is inoperable). The Bases for this TS clarify this requirement. This footnote is not revised in the proposed TS and is applicable to the modified logic configuration and proposed TS.

The licensee also proposed to revise action 70 in Table 3.3.7.1-1 for clarification. The current action requires that with one of the required monitors inoperable, the inoperable channel must be placed in the downscale tripped condition within 1 hour and must be restored to operable status within 7 days, or, within the next 6 hours, the control room emergency filtration system must be operated in the pressurization mode. The wording in the current TS is unclear as to which are the required monitors and what constitutes a channel. In addition, placing a channel in the downscale tripped condition does not result in a trip of that channel and a downscaled channel is not available to initiate the protective actions. Therefore, a Technical Specification Clarification was prepared in 1993 to specify actions that would ensure automatic trip capability was maintained.

The licensee proposed a revised action 70(a) which would require that with the number of operable channels one less than the minimum required, the inoperable channel must be placed in the tripped condition within one hour. Revised action 70(b) would require that with both channels in a trip system inoperable, the trip system must be declared inoperable and must be restored within 7 days, or, within the next 6 hours, the control room emergency filtration system must be operated in the pressurization mode. The proposed action maintains the intent of the current TS since, under the current logic configuration, a trip system is considered to be a channel. The proposed action ensures that automatic trip capability is maintained as long as one trip system in the train is operable. If automatic trip capability can not be maintained (i.e., if both trip systems in a train become inoperable) the licensee is required to place the control room emergency filtration system in the pressurization mode of operation within one hour in accordance with action 70(c). This is consistent with the current TS action which requires that with both of the required monitors inoperable, the control room emergency filtration system must be placed in the pressurization mode within one hour. The proposed TS maintains the current requirements and allowed outage times and provides clarification. The proposed actions ensure that automatic trip capability is maintained and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been

no public comment on such finding (62 FR 45455). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Skay

Date: October 9, 1997