

March 1, 1988

Docket Nos. 50-373
and 50-374

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Mr. L. D. Butterfield, Jr.
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Butterfield:

SUBJECT: IMPLEMENTATION OF 10 CFR 50.62, ATWS RULE, LASALLE COUNTY STATION
UNITS 1 AND 2, TAC NOS. 59107 AND 59108

By letters dated April 6, 1987 and October 23, 1987, Commonwealth Edison Company submitted information concerning the requirements of 10 CFR 50.62 for LaSalle County Station Units 1 and 2. We find the proposed design of the LaSalle Standby Liquid Control System consistent with the requirements of 10 CFR 50.62 for that system and acceptable to us. However, the described values of concentration and relief valve setpoint should be included in a Technical Specification revision applicable to the SLCS. The single pump minimum flow requirement of 41.2 gpm (4.1.5.c.2) should be retained even though the ATWS rule requirement is for simultaneous two pump operation at a total flow rate of 82.4 gpm. Enclosure 1 is our Safety Evaluation

If you should have any questions, please contact Paul Shemanski, Project Manager, at (301) 492-3017.

Sincerely,

151

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc: See next page

See previous concurrence
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Mr. L. D. Butterfield, Jr.
Commonwealth Edison Company

LaSalle County Nuclear Power Station
Units 1 & 2

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Nuclear Facility Safety
Illinois Department of Nuclear Safety
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Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
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Glen Ellyn, Illinois 60137

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Commonwealth Edison Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ENCLOSURE 1

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATING TO STANDBY LIQUID CONTROL SYSTEM
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter dated April 6, 1987 from C. M. Allen, Commonwealth Edison Company (CECo), to the U. S. Nuclear Regulatory Commission (NRC), CECo submitted proposed system modifications to the LaSalle County Station (LSCS) Units 1 and 2 required to meet the Anticipated Transients Without Scram (ATWS) rule, 10 CFR 50.62. In addition, by letter dated October 23, 1987, from C. M. Allen (CECo), to the NRC, CECo responded to a request for additional information regarding ATWS system modifications which addresses the equivalency requirement in the Standby Liquid Control System (SLCS).

2.0 EVALUATION

The present LSCS Standby Liquid Control System (SLCS) is a two-pump system for each unit. Each pump is designed to inject a minimum of 41.2 gpm. The present sodium pentaborate concentration is 13.4 weight percent. The proposed mode of operation to meet ATWS Rule requirements is for simultaneous two pump operation at a total flow rate of 82.4 gpm. This flow rate in conjunction with a minimum sodium pentaborate concentration of 13.6 weight percent is proposed to satisfy the "equivalent control" requirements of paragraph (c)(4) of 10 CFR 50.62.

The system description provided by the licensee has been reviewed by the staff against the requirements of the ATWS Rule (10 CFR 50.62), and Generic Letter 85-03, "Clarification of Equivalent Control Capacity for Standby Liquid

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Control Systems," dated January 28, 1985. The proposed mode of operation to achieve a flow rate of 82.4 gpm with a sodium pentaborate minimum concentration of 13.6 weight percent is equivalent to 86 gpm at 13 weight percent and therefore is in compliance with 10 CFR 50.62(c)(4). Accordingly, this proposed mode of operation is acceptable.

The licensee indicated that both SLCS pumps will be test-operated simultaneously to verify that a minimum allowable double pump flow rate of 82.4 gpm can be achieved. This test will be performed once as part of the post-modification testing program after which the original LSCS Technical Specification requirements for periodic testing will be in effect. The licensee has also stated that the modifications will include pulsation dampers in the discharge piping to attenuate variations in the positive displacement pumps' pressure and an increased relief valve setpoint to 1500 psi to avoid inadvertent lifting of the valve during system testing and operation. We find the proposed modifications acceptable. The described values of concentration, flow rate, and relief valve setpoint should be included in a Technical Specification revision applicable to the SLCS.

3.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that the Standby Liquid Control System for LaSalle County Station Units 1 and 2 meets the requirements of 10 CFR 50.62(c)(4) and is acceptable.



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NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ENCLOSURE 1

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATING TO STANDBY LIQUID CONTROL SYSTEM
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION UNITS 1 AND 2
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 1, 1988

MEMORANDUM FOR: Sholly Coordinator

FROM: Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE
OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES
(TAC NOS. 66100 and 66101)

Commonwealth Edison Company, Docket Nos. 50-373 and 50-374, LaSalle County
Station, Units 1 and 2, LaSalle County, Illinois

Date of application for amendments: August 24, 1987

Brief description of amendments: These amendments involve administrative changes which clarify the conditions necessary to perform the Technical Specifications jet pump operability surveillance and correct typographical errors.

Date of issuance: March 4, 1988

Effective date: Fifteen days following date of issuance.

Amendment Nos.: 55 and 36

Facility Operating License Nos. NPF-11 and NPF-18: Amendments revise the Technical Specifications.

Date of initial notice in FEDERAL REGISTER: December 30, 1987 (52 FR 49220)

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 4, 1988.

No significant hazards consideration comments received: No.

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CF

Local Public Document Room location: Public Library of Illinois Valley
Community College, Rural Route No. 1., Oglesby, Illinois 61348.

Original Signed by/

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
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