

DOCKET FILE



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

April 27, 1989

Docket Nos: 50-373
and 50-374

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Kovach:

Subject: ISSUANCE OF AMENDMENT NOS. 66 AND 47 TO FACILITY OPERATING LICENSE
NOS. NPF-11 AND NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2
(TAC NOS. 71413 AND 71414)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 66 to Facility Operating License No. NPF-11 and Amendment No. 47 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letter dated February 17, 1989.

The amendments revise the LaSalle County Station, Units 1 and 2 Technical Specifications by changes to the Administrative Controls Section 6.0 to include removal of the Organizational figures, a position change from Radiation Chemistry Technician to Radiation Protection Technician, several position title changes and a clarification to the distribution requirements for Onsite Reviews.

A copy of the related Safety Evaluation supporting Amendment No. 66 to Facility Operating License No. NPF-11 and Amendment No. 47 to Facility Operating License No. NPF-18 is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Paul C. Shemanski

Paul C. Shemanski, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

- 1. Amendment No. 66 to NPF-11
- 2. Amendment No. 47 to NPF-18
- 3. Safety Evaluation

cc w/enclosure:
See next page

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April 27, 1989

Docket Nos: 50-373
and 50-374

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
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Division of Reactor Projects - III,
IV, V and Special Projects

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2. Amendment No. 47 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

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Mr. Thomas J. Kovach
Commonwealth Edison Company

LaSalle County Nuclear Power Station
Units 1 & 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated February 17, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.66, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: April 27, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

INSERT

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6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, INVESTIGATION, AND AUDIT

- A. Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.
1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Quality Assurance Manual or the Management Plan for Nuclear Operations, Section 3 Organizational Authority, Activities; Section 6 Interdepartmental Relationships.
 2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
 3. The Senior Vice President-Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
 4. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- B. The Shift Supervisor shall be responsible for directing and commanding the overall operation of the facility on his shift. The primary management responsibility of the Shift Supervisor shall be for safe operation of the nuclear facility on his shift under all conditions. A management directive signed by Vice President BWR Operations emphasizing this primary management responsibility and that clearly establishes the command duties of the Shift Supervisor shall be reissued to all station personnel on an annual basis.
- C. The shift manning for the station shall be as shown in Figure 6.1-3. The individual filling the position of Technical Staff Supervisor shall meet the minimum acceptable level for "Technical Manager" as described in Section 4.2.4 of ANSI N18.1-1971. The individuals filling the positions of Production Superintendent and Technical Superintendent shall meet the minimum acceptable level for "Plant Manager" as described in Section 4.2.1 of ANSI N18.1-1971.

ADMINISTRATIVE CONTROLS

1. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in OPERATIONAL CONDITION 1, 2 or 3, at least one licensed Senior Reactor Operator who has been designated by the Shift Supervisor to assume the control room direction responsibility shall be in the Control Room.
2. A radiation protection technician* shall be on site when fuel is in the reactor.
3. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
4. A site Fire Brigade of at least 5 members shall be maintained onsite at all times*. The Fire Brigade shall not include the Shift Supervisor, the Station Control Room Engineer and the 2 other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
5. The Onsite Nuclear Safety Group (ONSG) shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving unit safety. The ONSG shall be composed of at least three, dedicated, full-time engineers of multi-disciplines located on site and shall be augmented on a part-time basis by personnel from other parts of the Commonwealth Edison Company organization to provide expertise not represented in the group. The ONSG shall be responsible for maintaining surveillance of unit activities to provide independent verification# that these activities are performed correctly and that human errors are reduced as much as practical. The ONSG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety to the Safety Assessment Manager and the Station Manager.
6. The Station Control Room Engineer (SCRE) serves as the lead control room SRO during normal operations and as the Shift Technical Advisor (STA) during abnormal operating and accident conditions. In the event of abnormal operating or accident conditions, the SCRE will relinquish his job as control room SRO to the Shift Foreman and will assume the role of STA, when he shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to the safe operation of the unit.

*The radiation protection technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed two hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

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ADMINISTRATIVE CONTROLS

7. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g, senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. However, in the event that unforeseen problems require substantial amounts of overtime to be used, the following guidelines shall be followed:

- a. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- b. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period, all excluding shift turnover time.
- c. A break of at least eight hours should be allowed between work periods, including shift turnover time.
- d. The use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Production or Technical Superintendents for their individual departments, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Production or Technical Superintendents to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

8. The Assistant Superintendent Operating shall hold a Senior Reactor Operator License.
- D. Qualifications of the station management and operating staff shall meet minimum acceptable levels as described in ANSI N18.1, "Selection and Training of Nuclear Power Plant Personnel," dated March 8, 1971. The Health Physics Supervisor shall meet the requirements of radiation protection manager of Regulatory Guide 1.8, September 1975. The ANSI N18.1-1971 qualification requirements for Radiation Protection Technician may also be met by either of the following alternatives:
 1. Individuals who have completed the Radiation Protection Technician training program and have accrued 1 year of working experience in the specialty, or
 2. Individuals who have completed the Radiation Protection Technician training program, but have not yet accrued 1 year of working experience in the specialty, who are supervised by on-shift health physics supervision who meet the requirements of ANSI N18.1-1971 Section 4.3.2, "Supervisor Not Requiring AEC Licenses," or Section 4.4.4, "Radiation Protection."
 - E. Retraining and replacement training of Station personnel shall be in accordance with ANSI N18.1, "Selection and Training of Nuclear Power Plant Personnel", dated March 8, 1971 and Appendix "A" of 10 CFR Part 55, and shall include familiarization with relevant industry operational experience identified by the ONSG.

ADMINISTRATIVE CONTROLS

- F. Retraining shall be conducted at intervals not exceeding 2 years.
- G. The Review and Investigative Function and the Audit Function of activities affecting quality during facility operations shall be constituted and have the responsibilities and authorities outlined below:
 - 1. The Superintendent of the Offsite Review and Investigative Function shall be appointed by the Manager of Quality Assurance Nuclear Safety (QA/NS). The Corporate Audit Function shall be the responsibility of the Manager of QA/NS Assurance and shall be independent of operations.

The Manager of QA/NS reports directly to the Chief Executive officer and has the responsibility to set Corporate policy for both the areas of Quality Assurance and Nuclear Safety. Policy is promulgated through a central policy committee directed by the manager of QA/NS. The Manager of QA/NS has the responsibility for the performance of periodic audits of each nuclear station and corporate departments to determine that QA/NS policy is being carried out.

- a. Offsite Review and Investigative Function

The Superintendent of the Offsite Review and Investigative Function shall: (1) provide directions for the review and investigative function and appoint a senior participant to provide appropriate direction, (2) select each participant for this function, (3) select a complement of more than one participant who collectively possess background and qualifications in the subject matter under review to provide comprehensive interdisciplinary review coverage under this function, (4) independently review and approve the findings and recommendations developed by personnel performing the review and investigative function, (5) approve and report in a timely manner all findings of non-compliance with NRC requirements to the Station Manager, Vice President BWR Operations, Manager of QA/NS, Assistant Vice President (AVP) Quality Programs and Assessment, and the Senior Vice President - Nuclear Operations. During periods when the Superintendent of Offsite Review and Investigative Function is unavailable, he shall designate this responsibility to an established alternate, who satisfies the formal training and experience for the Superintendent of the Offsite Review and Investigative Function. The responsibilities of the personnel performing this function are stated below. The Offsite Review and Investigative Function shall review:

- 1) The safety evaluations for (1) changes to procedures, equipment, or systems as described in the safety analysis report and (2) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question. Proposed changes to the Quality Assurance Program description shall be reviewed and approved by the Manager of QA/NS.
- 2) Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.

ADMINISTRATIVE CONTROLS

- 3) Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
- 4) Proposed changes in Technical Specifications or NRC operating licenses.
- 5) Noncompliance with NRC requirements, or of internal procedures, or instructions having nuclear safety significance.
- 6) Significant operating abnormalities or deviation from normal and expected performance of plant equipment that affect nuclear safety as referred to it by the Onsite Review and Investigative Function.
- 7) All REPORTABLE EVENTS.
- 8) All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems, or components.
- 9) Review and report findings and recommendations regarding all changes to the Generating Stations Emergency Plan prior to implementation of such change.
- 10) Review and report findings and recommendations regarding all items referred by the Technical Staff Supervisor, Station Manager, Vice President BWR Operations, and AVP Quality Programs and Assessment.

b. Station Audit Function

The Station Audit Function shall be the responsibility of the AVP Quality Programs and Assessment independent of the BWR Operation. Such responsibility is delegated to the Nuclear Quality Programs Manager.

The Nuclear Quality Programs Manager, or designated Corporate Staff or Supervisor approved by the AVP Quality Programs and Assessment, shall approve the audit agenda and checklists, the findings and the report of each audit. Audits shall be performed in accordance with the Company Quality Assurance Program and Procedures. Audits shall be performed to assure that safety-related functions are covered within a period of 2 years or less as designated below.

- 1) Audit of the conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- 2) Audit of the adherence to procedures, training and qualification of the station staff at least once per year.
- 3) Audit of the results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or methods of operation that affect nuclear safety at least once per 6 months.
- 4) Audit of the performance of activities required by the Quality Assurance Program to meet the Criteria of Appendix "B" 10 CFR 50.

ADMINISTRATIVE CONTROLS

Audit Function (Continued)

- 5) Audit of the Facility Emergency Plan and implementing procedures at least once per 12 months.
- 6) Audit of the Facility Security Plan and implementing procedures.
- 7) Audit onsite and offsite reviews.
- 8) Audit the Facility Fire Protection Program and implementing procedures.
- 9) The radiological environmental monitoring program and the results thereof at least once per 12 months.
- 10) The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- 11) The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes.
- 12) The performance of activities required by the Company Quality Assurance Program and Procedures to meet the criteria of Regulatory Guide 4.15, December 1977, at least once per 12 months.

Report all findings of noncompliance with NRC requirements and recommendations and results of each audit to the Station Manager, Manager of QA/NS, Vice President BWR Operations, AVP Quality Programs and Assessment, the Senior Vice President-Nuclear Operations, and the Chief Operations Officer.

c. Authority

The Manager of QA/NS reports to the Chief Executive Officer. The manager of QA/NS has the authority to order unit shutdown or request any other action which he deems necessary to avoid unsafe plant conditions.

The AVP Quality Programs and Assessment reports to the Senior Vice President-Nuclear Operations. The AVP Quality Programs and Assessment has the authority to recommend unit shutdown or request any other action which he deems necessary to avoid unsafe plant conditions. All such disagreements shall be reported immediately to the Manager of QA/NS and the Chief Operation Officer.

d. Records

- 1) Reviews, audits, and recommendations shall be documented and distributed as covered in 6.1.G.1.a and 6.1.G.1.b
- 2) Copies of documentation, reports, and correspondence shall be kept on file at the station.

e. Procedures

Written administrative procedures shall be prepared and maintained for the offsite reviews and investigative functions described in Specification 6.1.G.1.a. and for the audit functions described in Specification 6.1.G.1.b. Those procedures shall cover the following:

ADMINISTRATIVE CONTROLS

Offsite Review and Investigative Function (Continued)

- 1) Content and method of submission of presentations to the Superintendent of the Offsite Review and Investigative Function.
- 2) Use of committees and consultants.
- 3) Review and approval.
- 4) Detailed listing of items to be reviewed.
- 5) Method of (1) appointing personnel, (2) performing reviews, investigations, (3) reporting findings and recommendations of reviews and investigations, (4) approving reports, and (5) distributing reports.
- 6) Determining satisfactory completion of action required based on approved findings and recommendations reported by personnel performing the review and investigative function.

f. Personnel

- 1) The persons, including consultants, performing the review and investigative function, in addition to the Superintendent of the Offsite Review and Investigative Function shall have expertise in one or more of the following disciplines as appropriate for the subject or subjects being reviewed and investigated:
 - a) nuclear power plant technology,
 - b) reactor operations,
 - c) utility operations,
 - d) power plant design,
 - e) reactor engineering,
 - f) radiological safety,
 - g) reactor safety analysis,
 - h) instrumentation and control,
 - i) metallurgy, and
 - j) any other appropriate disciplines required by unique characteristics of the facility.
- 2) Individuals performing the Review and Investigative Function shall possess a minimum formal training and experience as listed below for each discipline.
 - a) Nuclear Power Plant Technology
Engineering graduate or equivalent with 5 years experience in the nuclear power field design and/or operation.
 - b) Reactor Operations
Engineering graduate or equivalent with 5 years experience in nuclear power plant operations.
 - c) Utility Operations
Engineering graduate or equivalent with at least 5 years of experience in utility operation and/or engineering.

ADMINISTRATIVE CONTROLS

Offsite Review and Investigative Function (Continued)

- d) Power Plant Design
Engineering graduate or equivalent with at least 5 years of experience in power plant design and/or operation.
 - e) Reactor Engineering
Engineering graduate or equivalent. In addition, at least 5 years of experience in nuclear plant engineering, operation, and/or graduate work in nuclear engineering or equivalent in reactor physics is required.
 - f) Radiological Safety
Engineering graduate or equivalent with at least 5 years of experience in radiation control and safety.
 - g) Reactor Safety Analysis
Engineering graduate or equivalent with at least 5 years of experience in nuclear engineering.
 - h) Instrumentation and Control
Engineering graduate or equivalent with at least 5 years of experience in instrumentation and control design and/or operation.
 - i) Metallurgy
Engineering graduate or equivalent with at least 5 years of experience in the metallurgical field.
- 3) The Superintendent of the Offsite Review and Investigative Function shall have experience and training which satisfy ANSI N18.1-1971 requirements for plant managers.
2. The Onsite Review and Investigative Function shall be supervised by the Station Manager.
- a. Onsite Review and Investigative Function
The Station Manager shall: (1) provide direction for the Review and Investigative Function and appoint the Technical Staff Supervisor, or other comparably qualified individual as a senior participant to provide appropriate directions; (2) approve participants for this function; (3) assure that a component of more than one participant who collectively possess background and qualifications in the subject matter under review are selected to provide comprehensive interdisciplinary review coverage under this function; (4) independently review and approve the findings and recommendations developed by personnel performing the Review and Investigative Function; (5) report all findings of noncompliance with NRC requirements, and provide recommendations; and (6) submit to the Offsite Review and Investigative Function for concurrence in a timely manner, those items described in Specification 6.1.G.1.a which have been approved by the Onsite Review and Investigative Function.

ADMINISTRATIVE CONTROLS

Onsite Review and Investigative Function (Continued)

The responsibilities of the personnel performing this function are stated below:

- 1) Review of (1) procedures required by Specification 6.2 and changes thereto, (2) all programs required by Specification 6.2 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- 2) Review of all proposed test and experiments that affect nuclear safety.
- 3) Review of all proposed changes to the Technical Specifications.
- 4) Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- 5) Investigation of all noncompliance with NRC requirements and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence;
- 6) Review of facility operations to detect potential safety hazards.
- 7) Performance of special reviews and investigations and reports thereon as requested by the Superintendent of the Offsite Review and Investigative Function.
- 8) Review of the Station Security Plan and shall submit recommended changes to the Director of Corporate Security and the AVP Quality Programs and Assessment in lieu of distribution in accordance with Specification 6.1.G.2c(1).
- 9) Review of the Emergency Plan and station implementing procedures and identification of recommended changes;
- 10) Review of reportable events and actions taken to prevent recurrence.
- 11) Review of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence;
- 12) Review of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and radwaste treatment systems.

b. Authority

The Technical Staff Supervisor is responsible to the Station Manager and shall make recommendations in a timely manner in all areas of review, investigation, and quality control phases of plant maintenance, operation, and administrative procedures relating to facility operations and shall have the authority to request the action necessary to ensure compliance with rules, regulations, and procedures when in his opinion such action is

ADMINISTRATIVE CONTROLS

Onsite Review and Investigative Function (Continued)

necessary. The Station Manager shall follow such recommendations or select a course of action that is more conservative regarding safe operation of the facility. All such disagreements shall be reported immediately to the Vice President BWR Operations and the Superintendent of the Offsite Review and Investigative Function.

c. Records

- 1) Reports, reviews, investigations, and recommendations prepared and performed for Specification 6.1.G.2a shall be documented with copies to the Vice President BWR Operations the Superintendent of the Offsite Review and Investigative Function, the Station Manager, and the AVP Quality Programs and Assessment.
- 2) Copies of all records and documentation shall be kept on file at the station.

d. Procedures

Written administrative procedures shall be prepared and maintained for conduct of the Onsite Review and Investigative Function. These procedures shall include the following:

- 1) Content and method of submission and presentation to the Station Manager, Vice President BWR Operations and the Superintendent of the Offsite Review and Investigative Function.
- 2) Use of committees.
- 3) Review and approval.
- 4) Detailed listing of items to be reviewed.
- 5) Procedures for administration of the quality control activities.
- 6) Assignment of responsibilities.

e. Personnel

- 1) The personnel performing the Onsite Review and Investigative Function, in addition to the Station Manager, shall consist of persons having expertise in:
 - a) nuclear power plant technology,
 - b) reactor operations,
 - c) reactor engineering,
 - d) radiological safety and chemist,
 - e) Instrumentation and control, and
 - f) mechanical and electric systems.
- 2) Personnel performing the Onsite Review and Investigative Function shall meet minimum acceptable levels as described in ANSI N18.1-1971, Sections 4.2 and 4.4.

ADMINISTRATIVE CONTROLS

Onsite Review and Investigative Function (Continued)

H. Fire Protection Program

An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

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Figure 6.1-3

MINIMUM SHIFT CREW COMPOSITION

WITH UNIT 2 IN CONDITION 1, 2, OR 3		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITION 1, 2 and 3	CONDITION 4 and 5
SE	1 ^a	1 ^a
SF	1 ^a	None
RO	2 ^b	1
AO	2 ^b	1
SCRE	1 ^a	None

or, whenever a SCRE (SRO/STA) is not included in the shift crew composition, the minimum shift crew composition shall be as follows:

WITH UNIT 2 IN CONDITION 1, 2, OR 3		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITION 1, 2 and 3	CONDITION 4 and 5
SE	1 ^a	1 ^a
SF	1 ^a	None
RO	2 ^b	1
AO	2 ^b	1
STA	1 ^a	None

WITH UNIT 2 IN CONDITION 4 or 5 OR DEFUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITION 1, 2 and 3	CONDITION 4 and 5
SE	1 ^a	1 ^a
SF	1	None
RO	2	1
AO	2	2 ^b
STA	1 ^a	None

Figure 6.1-3 (Continued)

MINIMUM SHIFT CREW COMPOSITION

NOTES

- a/ Individual may fill the same position on Unit 2.
- b/ One of the two required individuals may fill the same position on Unit 2.
- SE - Shift Supervisor (Shift Engineer) with a Senior Reactor Operators License on Unit 1.
- SF - Shift Foreman with a Senior Reactor Operators License on Unit 1.
- RO - Individual with a Reactor Operators License on Unit 1.
- AO - Auxiliary Operator.
- SCRE - Station Control Room Engineer with a Senior Reactor Operators License.

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Figure 6.1-3 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Figure 6.1-3. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

While the unit is in OPERATIONAL CONDITION 1, 2, or 3, an individual with a valid SRO license shall be designated to assume the Control Room direction function. While the unit is in OPERATIONAL CONDITION 4 or 5, an individual with a valid SRO or RO license shall be designated to assume the Control Room direction function.

ADMINISTRATIVE CONTROLS

6.1.1 HIGH RADIATION AREAS

6.1.1.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR 20, in lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr* but less than 1000 mrem/hr* shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures, or personnel continuously escorted by such individuals, may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas in which the intensity of radiation is greater than 100 mrem/hr* but less than 1000 mrem/hr*, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. A health physics qualified individual, i.e., qualified in radiation protection procedures, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Health Physicist in the Radiation Work Permit (RWP).

6.1.1.2 In addition to the requirements of 6.1.1.1, above, for areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem*, the computer shall be programmed to permit entry through locked doors for any individual requiring access to any such High-High Radiation Areas for the time that access is required.

6.1.1.3 Keys to manually open computer controlled High Radiation Area doors and High-High Radiation Area doors shall be maintained under the Administration control of the Shift Supervisor on duty and/or the Health Physicist.

6.1.1.4 High-High Radiation areas, as defined in 6.1.1.2 above, not equipped with the computerized card readers shall be maintained in accordance with 10 CFR 20.203 c.2 (iii), locked except during periods when access to the area is required with positive control over each individual entry, or 10 CFR 20.203.c.4. In the case of a High Radiation Area established for a period of 30 days or less, direct surveillance to prevent unauthorized entry may be substituted. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem* that are located within

*Measurement made at 18" from source of radioactivity.

ADMINISTRATIVE CONTROLS

HIGH RADIATION AREAS (Continued)

large areas, such as the containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, direct or remote, such as use of closed circuit TV cameras, continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

6.2 PLANT OPERATING PROCEDURES AND PROGRAMS

- A. Detailed written procedures including applicable checkoff lists covering items listed below shall be prepared, approved, and adhered to:
1. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
 2. Refueling operations.
 3. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components including responses to alarms, suspected primary system leaks, and abnormal reactivity changes.
 4. Emergency conditions involving potential or actual release of radioactivity - "Generating Stations Emergency Plan" and station emergency and abnormal procedures.
 5. Instrumentation operation which could have an effect on the safety of the facility.
 6. Preventive and corrective maintenance operations which could have an effect on the safety of the facility.
 7. Surveillance and testing requirements.
 8. Tests and experiments.
 9. Procedure to ensure safe shutdown of the plant.
 10. Station Security Plan and implementation procedures.
 11. Fire Protection Program implementation.
 12. Process Control Program implementation.
 13. OFFSITE DOSE CALCULATION MANUAL implementation.
 14. Company Quality Assurance Program and Procedures for effluent and environmental monitoring, using the guidance in Regulatory Guide 4.15, December 1977.

ADMINISTRATIVE CONTROLS

PLANT OPERATING PROCEDURES AND PROGRAMS (Continued)

- B. Radiation control procedures shall be maintained, made available to all station personnel, and adhered to. These procedures shall show permissible radiation exposure and shall be consistent with the requirements of 10 CFR 20. This radiation protection program shall be organized to meet the requirements of 10 CFR 20.
- C. 1. Procedures for items identified in Specification 6.2.A and any changes to such procedures shall be reviewed and approved by the Operating Engineer and the Technical Staff Supervisor in the areas of operation, fuel handling, or instrument maintenance, and by the Maintenance Assistant Superintendent and Technical Staff Supervisor in the areas of plant maintenance and plant inspection. Procedures for items identified in Specification 6.2.B and any changes to such procedures shall be reviewed and approved by the Technical Staff Supervisor and the Health Physics Supervisor. At least one person approving each of the above procedures shall hold a valid senior operator's license. In addition, these procedures and changes thereto, must have authorization by the Station Manager before being implemented.
2. Work and instruction type procedures which implement approved maintenance or modification procedures shall be approved and authorized by the Maintenance Assistant Superintendent where the written authority has been provided by the Station Manager. The "Maintenance/Modification Procedure" utilized for safety-related work shall be so approved only if procedures referenced in the "Maintenance/Modification Procedure" have been approved as required by 6.2.A. Procedures which do not fall within the requirements of 6.2.A or 6.2.B may be approved by the Department Heads.
- D. Temporary changes to procedures 6.2.A and 6.2.B above may be made provided:
1. The intent of the original procedure is not altered.
 2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 3. The change is documented, reviewed by the Onsite Review and Investigative Function and approved by the Station Manager within 14 days of implementation.
- E. Drills of the emergency procedures described in Specification 6.2.A.4 shall be conducted at frequencies as specified in the Generating Stations Emergency Plan (GSEP). These drills will be planned so that during the course of the year, communication links are tested and outside agencies are contacted.

ADMINISTRATIVE CONTROLS

PLANT OPERATING PROCEDURES AND PROGRAMS (Continued)

F. The following programs shall be established, implemented, and maintained:

1. Primary Coolant Sources Outside Primary Containment

A program to reduce leakage from those portions of systems outside primary containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include LPCS, HPCS, RHR/LPCI, RCIC, hydrogen recombiner, process sampling, containment monitoring, and standby gas treatment systems. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements, and
- b. Integrated leak test requirements for each system at refueling cycle intervals or less.

2. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- a. Training of personnel,
- b. Procedures for monitoring, and
- c. Provisions for maintenance of sampling and analysis equipment.

3. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- a. Training of personnel,
- b. Procedures for sampling and analysis,
- c. Provisions for maintenance of sampling and analysis equipment.

6.3 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE EVENT IN PLANT OPERATION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a Licensee Event Report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed pursuant to Specification 6.1.G.2.c(1).

ADMINISTRATIVE CONTROLS

6.4 ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED

If a safety limit is exceeded, the reactor shall be shut down immediately pursuant to Specification 2.1.1, 2.1.2 and 2.1.3, and critical reactor operation shall not be resumed until authorized by the NRC. The conditions of shutdown shall be promptly reported to the Vice President BWR Operations or his designated alternate. The incident shall be reviewed pursuant to Specifications 6.1.G.1.a and 6.1.G.2.a and a separate License Event Report for each occurrence shall be prepared in accordance with Section 50.73 to 10 CFR Part 50. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President BWR Operations and the Offsite Review and Investigative Function shall be notified within 24 hours.

6.5 PLANT OPERATING RECORDS

- A. Records and/or logs relative to the following items shall be kept in a manner convenient for review and shall be retained for at least 5 years:
1. Records of normal plant operation, including power levels and periods of operation at each power level;
 2. Records of principal maintenance and activities, including inspection and repair, regarding principal items of equipment pertaining to nuclear safety;
 3. Records and reports of reportable events;
 4. Records and periodic checks, inspection and/or calibrations performed to verify that the surveillance requirements (see Section 4 of these specifications) are being met. All equipment failing to meet surveillance requirements and the corrective action taken shall be recorded;
 5. Records of changes to operating procedures;
 6. Shift engineers' logs; and
 7. Byproduct material inventory records and source leak test results.

ADMINISTRATIVE CONTROLS

PLANT OPERATING RECORDS (Continued)

- B. Records and/or logs relative to the following items shall be recorded in a manner convenient for review and shall be retained for the life of the plant:
1. Substitution or replacement of principal items of equipment pertaining to nuclear safety;
 2. Changes made to the plant as it is described in the SAR;
 3. Records of new and spent fuel inventory and assembly histories;
 4. Updated, corrected, and as-built drawings of the plant;
 5. Records of plant radiation and contamination surveys;
 6. Records of offsite environmental monitoring surveys;
 7. Records of radiation exposure for all plant personnel, including all contractors and visitors to the plant, in accordance with 10 CFR Part 20;
 8. Records of radioactivity in liquid and gaseous wastes released to the environment;
 9. Records of transient or operational cycling for those components that have been designed to operate safely for a limited number of transient or operational cycles (identified in Table 5.7.1-1);
 10. Records of individual staff members indicating qualifications, experience, training, and retraining;
 11. Inservice inspections of the reactor coolant system;
 12. Minutes of meetings and results of reviews and audits performed by the offsite and onsite review and audit functions;
 13. Records of reactor tests and experiments;
 14. Records of Quality Assurance activities required by the QA Manual, except for those items specified in Section 6.5.A;
 15. Records of reviews performed for changes made to procedures on equipment or reviews of tests and experiments pursuant to 10 CFR 50.59; and
 16. Records of the service lives of all hydraulic and mechanical snubbers required by specification 3.7.9 including the date at which the service life commences and associated installation and maintenance records.
 17. Records of analyses required by the radiological environmental monitoring program.

ADMINISTRATIVE CONTROLS

6.6 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

A. Routine Reports

1. Startup Report

A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the startup report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every 3 months until all three events have been completed.

2. Annual Report

A tabulation shall be submitted on an annual basis prior to March 1 of each year of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions (Note: this tabulation supplements the requirements of Section 20.407 of 10 CFR 20), e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignments to various duty functions may be estimated based on pocket dosimeter, TLD, or film badge measurements. Small exposures totaling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

ADMINISTRATIVE CONTROLS

Annual Report (continued)

The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.4.5 shall be included in the Annual Report along with the following information: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Clean-up system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

3. Annual Environmental Radiological Operating Report^{3/}

- a. Routine environmental radiological operating reports covering the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year. The initial report shall be submitted prior to May 1 of the year following initial criticality.
- b. The annual environmental radiological operating reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, as appropriate, operational controls, as appropriate, and previous environmental surveillance reports and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of land use censuses required by Specification 3.12.2.

The annual environmental radiological operating reports shall include summarized and tabulated results in the format of Regulatory Guide 4.8, December 1975, of all radiological environmental samples taken during the report period. In the event that some results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

The reports shall also include the following: a summary description of the environmental radiological monitoring program; a map of all sampling locations keyed to a table giving distances and directions from one reactor; and the results of licensee participation in the Interlaboratory Comparison Program, required by Specification 3.12.3.

The report shall include an annual summary of hourly meteorological data collected over the previous year. This annual summary may be either in the form of an hour-by-hour listing of wind speed, wind direction, and atmospheric stability, and precipitation (if measured) on magnetic tape, or in the form of joint frequency distributions of wind speed, wind direction, and atmospheric stability. This same report shall include an assessment of the radiation doses due to the radioactive liquid and gaseous effluents released from the unit or station during the previous calendar year. The assessment of radiation doses shall be performed in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

The report shall also include an assessment of radiation doses to the likely most exposed member of the public from reactor releases and other nearby uranium fuel cycle sources (including

Annual Environmental Radiological Operating Report (Continued)

doses from primary effluent pathways and direct radiation) for the previous 12 consecutive months to show conformance with 40 CFR 190, Environmental Radiation Protection Standards for Nuclear Power Operation. The assessment of radiation doses shall be performed in accordance with the ODCM.

4. Semiannual Radioactive Effluent Release Report^{3/}
 - a. Routine radioactive effluent release reports covering the operation of the unit during the previous 6 months of operation shall be submitted within 60 days after January 1 and July 1 of each year. The period of the first report shall begin with the date of initial criticality.
 - b. The radioactive effluent release reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21, "Measuring, Evaluating and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, June 1974, with data summarized on a quarterly basis following the format of Appendix B thereof.

^{3/}A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.

ADMINISTRATIVE CONTROLS

Semiannual Radioactive Effluent Release Report (Continued)

The radioactive effluent release report shall include the following information for each type of solid waste shipped offsite during the report period:

- a. Container volume,
- b. Total curie quantity (specify whether determined by measurement or estimate),
- c. Principal radionuclides (specify whether determined by measurement or estimate),
- d. Type of waste (e.g., spent resin, compacted dry waste, evaporator bottoms),
- e. Type of container (e.g., LSA, Type A, Type B, Large Quantity), and
- f. Solidification agent (e.g., cement, urea formaldehyde).

The radioactive effluent release reports shall include unplanned releases from the site to unrestricted areas of radioactive materials in gaseous and liquid effluents on a quarterly basis.

The radioactive effluent release reports shall include any changes to the PROCESS CONTROL PROGRAM (PCP) made during the reporting period.

5. Monthly Operating Report

Routine reports of operating statistics and shutdown experience, including documentation of all challenges to safety/relief valves, shall be submitted on a monthly basis to the Director, Office of Nuclear Reactor Regulation, Mail Station P1-137, US Nuclear Regulatory Commission, Washington, DC 20555, with a copy of the appropriate Regional Office, to arrive no later than the 15th of each month following the calendar month covered by the report.

Any changes to the OFFSITE DOSE CALCULATION MANUAL shall be submitted with the Monthly Operating Report within 90 days in which the change(s) was made effective. In addition, a report of any major changes to the radioactive waste treatment systems shall be submitted with the Monthly Operating Report for the period in which the evaluation was reviewed and accepted by Onsite Review and Investigative Function.

B. Deleted

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ADMINISTRATIVE CONTROLS

C. Unique Reporting Requirements

1. Special Reports shall be submitted to the Director of the Office of Inspection and Enforcement (Region III) within the time period specified for each report.

6.7 PROCESS CONTROL PROGRAM (PCP)*

6.7.1 The PCP shall be approved by the Commission prior to implementation.

6.7.2 Licensee initiated changes to the PCP:

- a. Shall be submitted to the Commission in the semi annual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 1. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 2. A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
 3. Documentation of the fact that the change has been reviewed and found acceptable by the Onsite Review and Investigative Function.
- b. Shall become effective upon review and acceptance by the Onsite Review and Investigative Function.

6.8 OFFSITE DOSE CALCULATION MANUAL (ODCM)*

6.8.1 The ODCM shall be approved by the Commission prior to implementation.

6.8.2 Licensee initiated changes to the ODCM:

- a. Shall be submitted to the Commission within 90 days of the date the change(s) was made effective. This submittal shall contain:
 1. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluations justifying the change(s);
 2. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and

(PCP)* Common to LaSalle Unit 1 and LaSalle Unit 2

*(ODCM) Common to LaSalle Unit 1 and LaSalle Unit 2

ADMINISTRATIVE CONTROLS

Licensee initiated changes to the ODCM: (Continued)

3. Documentation of the fact that the change has been reviewed and found acceptable by the Onsite Review and Investigative Function.
- b. Shall become effective upon review and acceptance by the Onsite Review and Investigative Function.

6.9 MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS

6.9.1 Licensee initiated major changes to the radioactive waste systems (liquid, gaseous and solid):

- a. Shall be reported to the Commission in the Monthly Operating Report for the period in which the evaluation was reviewed by the Onsite Review and Investigative Function. The discussion of each change shall contain:
 1. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR 50.59;
 2. Sufficient detailed information to totally support the reason for the change without benefit or additional or supplemental information;
 3. A detailed description of the equipment, components and processes involved and the interfaces with other plant systems;
 4. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments thereto;
 5. An evaluation of the change which shows the expected maximum exposures to individual in the unrestricted area and to the general population that differ from those previously estimated in the license application and amendments thereto;
 6. A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period to when the changes are to be made;
 7. An estimate of the exposure to plant operating personnel as a result of the change; and
 8. Documentation of the fact that the change was reviewed and found acceptable by the Onsite Review and Investigative Function.
- b. Shall become effective upon review and acceptance by the Onsite Review and Investigative Function.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated February 17, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 47, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: April 27, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 47

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

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6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, INVESTIGATION, AND AUDIT

- A. Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.
1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Quality Assurance Manual or the Management Plan for Nuclear Operations, Section 3 Organizational Authority, Activities; Section 6 Interdepartmental Relationships.
 2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
 3. The Senior Vice President-Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
 4. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- B. The Shift Supervisor shall be responsible for directing and commanding the overall operation of the facility on his shift. The primary management responsibility of the Shift Supervisor shall be for safe operation of the nuclear facility on his shift under all conditions. A management directive signed by Vice President BWR Operations emphasizing this primary management responsibility and that clearly establishes the command duties of the Shift Supervisor shall be reissued to all station personnel on an annual basis.
- C. The shift manning for the station shall be as shown in Figure 6.1-3. The individual filling the position of Technical Staff Supervisor shall meet the minimum acceptable level for "Technical Manager" as described in Section 4.2.4 of ANSI N18.1-1971. The individuals filling the positions of Production Superintendent and Technical Superintendent shall meet the minimum acceptable level for "Plant Manager" as described in Section 4.2.1 of ANSI N18.1-1971.

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1. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in OPERATIONAL CONDITION 1, 2 or 3, at least one licensed Senior Reactor Operator who has been designated by the Shift Supervisor to assume the control room direction responsibility shall be in the Control Room.
2. A radiation protection technician* shall be on site when fuel is in the reactor.
3. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
4. A site Fire Brigade of at least 5 members shall be maintained onsite at all times*. The Fire Brigade shall not include the Shift Supervisor, the Station Control Room Engineer and the 2 other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
5. The Onsite Nuclear Safety Group (ONSG) shall function to examine unit operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving unit safety. The ONSG shall be composed of at least three, dedicated, full-time engineers of multi-disciplines located on site and shall be augmented on a part-time basis by personnel from other parts of the Commonwealth Edison Company organization to provide expertise not represented in the group. The ONSG shall be responsible for maintaining surveillance of unit activities to provide independent verification# that these activities are performed correctly and that human errors are reduced as much as practical. The ONSG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving unit safety to the Safety Assessment Manager and the Station Manager.
6. The Station Control Room Engineer (SCRE) serves as the lead control room SRO during normal operations and as the Shift Technical Advisor (STA) during abnormal operating and accident conditions. In the event of abnormal operating or accident conditions, the SCRE will relinquish his job as control room SRO to the Shift Foreman and will assume the role of STA, when he shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to the safe operation of the unit.

*The radiation protection technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed two hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

#Not responsible for sign-off feature.

ADMINISTRATION CONTROLS

7. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g, senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. However, in the event that unforeseen problems require substantial amounts of overtime to be used, the following guidelines shall be followed:

- a. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- b. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period, all excluding shift turnover time.
- c. A break of at least eight hours should be allowed between work periods, including shift turnover time.
- d. The use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Production or Technical Superintendents for their individual departments, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Production or Technical Superintendents to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

8. The Assistant Superintendent Operating shall hold a Senior Reactor Operator License.

- D. Qualifications of the station management and operating staff shall meet minimum acceptable levels as described in ANSI N18.1, "Selection and Training of Nuclear Power Plant Personnel," dated March 8, 1971. The Health Physics Supervisor shall meet the requirements of radiation protection manager of Regulatory Guide 1.8, September 1975. The ANSI N18.1-1971 qualification requirements for Radiation Protection Technician may also be met by either of the following alternatives:
1. Individuals who have completed the Radiation Protection Technician training program and have accrued 1 year of working experience in the specialty, or
 2. Individuals who have completed the Radiation Protection Technician training program, but have not yet accrued 1 year of working experience in the specialty, who are supervised by on-shift health physics supervision who meet the requirements of ANSI N18.1-1971 Section 4.3.2, "Supervisor Not Requiring AEC Licenses," or Section 4.4.4, "Radiation Protection."
- E. Retraining and replacement training of Station personnel shall be in accordance with ANSI N18.1, "Selection and Training of Nuclear Power Plant Personnel", dated March 8, 1971 and Appendix A of 10 CFR Part 55, and shall include familiarization with relevant industry operational experience identified by the ONSG.

ADMINISTRATION CONTROLS

Offsite Review and Investigative Function (Continued)

- F. Retraining shall be conducted at intervals not exceeding 2 years.
- G. The Review and Investigative Function and the Audit Function of activities affecting quality during facility operations shall be constituted and have the responsibilities and authorities outlined below:
1. The Superintendent of the Offsite Review and Investigative Function shall be appointed by the Manager of Quality Assurance/Nuclear Safety (QA/NS). The Corporate Audit Function shall be the responsibility of the Manager of QA/NS and shall be independent of operations.

The Manager of QA/NS reports directly to the Chief Executive Officer and has the responsibility to set Corporate Policy for both the areas of Quality Assurance and Nuclear Safety. Policy is promulgated through a central policy committee directed by the Manager of QA/NS. The Manager of QA/NS has the responsibility for the performance of periodic audits of each nuclear station and corporate departments to determine that QA/NS policy is being carried out.

a. Offsite Review and Investigative Function

The Superintendent of the Offsite Review and Investigative Function shall: (1) provide directions for the review and investigative function and appoint a senior participant to provide appropriate direction, (2) select each participant for this function, (3) select a complement of more than one participant who collectively possess background and qualifications in the subject matter under review to provide comprehensive interdisciplinary review coverage under this function, (4) independently review and approve the findings and recommendations developed by personnel performing the review and investigative function, (5) approve and report in a timely manner all findings of non-compliance with NRC requirements to the Station Manager, Vice President BWR Operations, Manager of QA/NS, Assistant Vice President (AVP) Quality Programs and Assessment, and the Senior Vice President - Nuclear Operations. During periods when the Superintendent of Offsite Review and Investigative Function is unavailable, he shall designate this responsibility to an established alternate, who satisfies the formal training and experience for the Superintendent of the Offsite Review and Investigative Function. The responsibilities of the personnel performing this function are stated below. The Offsite Review and Investigative Function shall review:

- 1) The safety evaluations for (1) changes to procedures, equipment, or systems as described in the safety analysis report and (2) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question. Proposed changes to the Quality Assurance Program description shall be reviewed and approved by the Manager of QA/NS.
- 2) Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59.

ADMINISTRATION CONTROLS

- 3) Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59.
- 4) Proposed changes in Technical Specifications or NRC operating licenses.
- 5) Noncompliance with NRC requirements, or of internal procedures, or instructions having nuclear safety significance.
- 6) Significant operating abnormalities or deviation from normal and expected performance of plant equipment that affect nuclear safety as referred to it by the Onsite Review and Investigative Function.
- 7) ALL REPORTABLE EVENTS.
- 8) All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems, or components.
- 9) Review and report findings and recommendations regarding all changes to the Generating Stations Emergency Plan prior to implementation of such change.
- 10) Review and report findings and recommendations regarding all items referred by the Technical Staff Supervisor, Station Manager, Vice President BWR Operations, and the AVP Quality Programs and Assessment.

b. Station Audit Function

The Station Audit Function shall be the responsibility of the AVP Quality Programs and Assessment independent of the BWR Operations. Such responsibility is delegated to the Nuclear Quality Programs Manager.

The Nuclear Quality Programs Manager, or designated Corporate Staff or Supervisor approved by the AVP Quality Programs and Assessment, shall approve the audit agenda and checklists, the findings and the report of each audit. Audits shall be performed in accordance with the Company Quality Assurance Program and Procedures. Audits shall be performed to assure that safety-related functions are covered within a period of 2 years or less as designated below.

- 1) Audit of the conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- 2) Audit of the adherence to procedures, training and qualification of the station staff at least once per year.
- 3) Audit of the results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or methods of operation that affect nuclear safety at least once per 6 months.

ADMINISTRATION CONTROLS

Audit Function (Continued)

- 4) Audit of the performance of activities required by the Quality Assurance Program to meet the Criteria of Appendix "B" 10 CFR 50.
- 5) Audit of the Facility Emergency Plan and implementing procedures at least once per 12 months.
- 6) Audit of the Facility Security Plan and implementing procedures.
- 7) Audit onsite and offsite reviews.
- 8) Audit the Facility Fire Protection Program and implementing procedures.
- 9) The radiological environmental monitoring program and the results thereof at least once per 12 months.
- 10) The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- 11) The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes.
- 12) The performance of activities required by the Company Quality Assurance Program and Procedures to meet the criteria of Regulatory Guide 4.15, December 1977, at least once per 12 months.

Report all findings of noncompliance with NRC requirements and recommendations and results of each audit to the Station Manager, Manager of QA/NS, Vice President BWR Operations, AVP Quality Programs and Assessment, the Senior Vice President-Nuclear Operations, and the Chief Operations Officer.

c. Authority

The Manager of QA/NS reports to the Chief Executive Officer. The Manager of QA/NS has the authority to order unit shutdown or request any other action which he deems necessary to avoid unsafe plant conditions.

The AVP Quality Programs and Assessment reports to the Senior Vice President-Nuclear Operations. The AVP Quality Programs and Assessment has the authority to recommend unit shutdown or request any other action which he deems necessary to avoid unsafe plant conditions. All such disagreements shall be reported immediately to the Manager of QA/NS and the Chief Operating Officer.

d. Records

- 1) Reviews, audits, and recommendations shall be documented and distributed as covered in 6.1.G.1.a and 6.1.G.1.b

ADMINISTRATION CONTROLS

Offsite Review and Investigative Function (Continued)

- 2) Copies of documentation, reports, and correspondence shall be kept on file at the station.

e. Procedures

Written administrative procedures shall be prepared and maintained for the offsite reviews and investigative functions described in Specification 6.1.G.1.a. and for the audit functions described in Specification 6.1.G.1.b. Those procedures shall cover the following:

- 1) Content and method of submission of presentations to the Superintendent of the Office Review and Investigative Function.
- 2) Use of committees and consultants.
- 3) Review and approval.
- 4) Detailed listing of items to be reviewed.
- 5) Method of (1) appointing personnel, (2) performing reviews, investigations, (3) reporting findings and recommendations of reviews and investigations, (4) approving reports, and (5) distributing reports.
- 6) Determining satisfactory completion of action required based on approved findings and recommendations reported by personnel performing the review and investigative function.

f. Personnel

- 1) The persons, including consultants, performing the review and investigative function, in addition to the Superintendent of the Offsite Review and Investigative Function shall have expertise in one or more of the following disciplines as appropriate for the subject or subjects being reviewed and investigated:
 - a) nuclear power plant technology,
 - b) reactor operations,
 - c) utility operations,
 - d) power plant design,
 - e) reactor engineering,
 - f) radiological safety,
 - g) reactor safety analysis,
 - h) instrumentation and control,
 - i) metallurgy, and
 - j) any other appropriate disciplines required by unique characteristics of the facility.
- 2) Individuals performing the Review and Investigative Function shall possess a minimum formal training and experience as listed below for each discipline.
 - a) Nuclear Power Plant Technology
Engineering graduate or equivalent with 5 years experience in the nuclear power field design and/or operation.

ADMINISTRATION CONTROLS

Offsite Review and Investigative Function (Continued)

- b) Reactor Operations
Engineering graduate or equivalent with 5 years experience in nuclear power plant operations.
 - c) Utility Operations
Engineering graduate or equivalent with at least 5 years of experience in utility operation and/or engineering.
 - d) Power Plant Design
Engineering graduate or equivalent with at least 5 years of experience in power plant design and/or operation.
 - e) Reactor Engineering
Engineering graduate or equivalent. In addition, at least 5 years of experience in nuclear plant engineering, operation, and/or graduate work in nuclear engineering or equivalent in reactor physics is required.
 - f) Radiological Safety
Engineering graduate or equivalent with at least 5 years of experience in radiation control and safety.
 - g) Reactor Safety Analysis
Engineering graduate or equivalent with at least 5 years of experience in nuclear engineering.
 - h) Instrumentation and Control
Engineering graduate or equivalent with at least 5 years of experience in instrumentation and control design and/or operation.
 - i) Metallurgy
Engineering graduate or equivalent with at least 5 years of experience in the metallurgical field.
- 3) The Superintendent of the Offsite Review and Investigative Function shall have experience and training which satisfy ANSI N18.1-1971 requirements for plant managers.
2. The Onsite Review and Investigative Function shall be supervised by the Station Manager.
- a. Onsite Review and Investigative Function
The Station Manager shall: (1) provide direction for the Review and Investigative Function and appoint the Technical Staff Supervisor, or other comparably qualified individual as a senior participant to provide appropriate directions; (2) approve participants for this function; (3) assure that a component of more than one participant who collectively possess background and qualifications in the subject matter under review are selected to provide

ADMINISTRATION CONTROLS

Onsite Review and Investigative Function (Continued)

comprehensive interdisciplinary review coverage under this function; (4) independently review and approve the findings and recommendations developed by personnel performing the Review and Investigative Function; (5) report all findings of noncompliance with NRC requirements, and provide recommendations; and (6) submit to the Offsite Review and Investigative Function for concurrence in a timely manner, those items described in Specification 6.1.G.1.a which have been approved by the Onsite Review and Investigative Function.

The responsibilities of the personnel performing this function are stated below:

- 1) Review of (1) procedures required by Specification 6.2 and changes thereto, (2) all programs required by Specification 6.2 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- 2) Review of all proposed test and experiments that affect nuclear safety.
- 3) Review of all proposed changes to the Technical Specifications.
- 4) Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- 5) Investigation of all noncompliance with NRC requirements and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence;
- 6) Review of facility operations to detect potential safety hazards.
- 7) Performance of special reviews and investigations and reports thereon as requested by the Superintendent of the Offsite Review and Investigative Function.
- 8) Review of the Station Security Plan and shall submit recommended changes to the Director of Corporate Security and the AVP Quality Programs and Assessment in lieu of distribution in accordance with Specification 6.1.G.2.c(1).
- 9) Review of the Emergency Plan and station implementing procedures and identification of recommended changes;
- 10) Review of reportable events and actions taken to prevent recurrence.
- 11) Review of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence;
- 12) Review of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and radwaste treatment systems.

ADMINISTRATION CONTROLS

Onsite Review and Investigative Function (Continued)

b. Authority

The Technical Staff Supervisor is responsible to the Station Manager and shall make recommendations in a timely manner in all areas of review, investigation, and quality control phases of plant maintenance, operation, and administrative procedures relating to facility operations and shall have the authority to request the action necessary to ensure compliance with rules, regulations, and procedures when in his opinion such action is necessary. The Station Manager shall follow such recommendations or select a course of action that is more conservative regarding safe operation of the facility. All such disagreements shall be reported immediately to the Vice President BWR Operations and the Superintendent of the Offsite Review and Investigative Function.

c. Records

- 1) Reports, reviews, investigations, and recommendations shall be documented with copies to the Vice President BWR Operations prepared and performed for Specification 6.1.G.2a, the Superintendent of the Offsite Review and Investigative Function, the Station Manager and the Manager of Quality Assurance.
- 2) Copies of all records and documentation shall be kept on file at the station.

d. Procedures

Written administrative procedures shall be prepared and maintained for conduct of the Onsite Review and Investigative Function. These procedures shall include the following:

- 1) Content and method of submission and presentation to the Station Manager, Vice President BWR Operations and the Superintendent of the Offsite Review and Investigative Function.
- 2) Use of committees.
- 3) Review and approval.
- 4) Detailed listing of items to be reviewed.
- 5) Procedures for administration of the quality control activities.
- 6) Assignment of responsibilities.

e. Personnel

- 1) The personnel performing the Onsite Review and Investigative Function, in addition to the Station Manager, shall consist of persons having expertise in:
 - a) nuclear power plant technology,
 - b) reactor operations,
 - c) reactor engineering,
 - d) radiological safety and chemist,
 - e) Instrumentation and control, and
 - f) mechanical and electric systems.

ADMINISTRATION CONTROLS

Onsite Review and Investigative Function (Continued)

- 2) Personnel performing the Onsite Review and Investigative Function shall meet minimum acceptable levels as described in ANSI N18.1-1971, Sections 4.2 and 4.4.

H. Fire Protection Program

An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

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Figure 6.1-3

MINIMUM SHIFT CREW COMPOSITION

WITH UNIT 1 IN CONDITION 1, 2, OR 3		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITIONS 1, 2 and 3	CONDITIONS 4 and 5
SE	1 ^a	1 ^a
SF	1 ^a	None
RO	2 ^b	1
AO	2 ^b	1
SCRE	1 ^a	None

or, whenever a SCRE (SRO/STA) is not included in the shift crew composition, the minimum shift crew composition shall be as follows:

WITH UNIT 1 IN CONDITION 1, 2, OR 3		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITIONS 1, 2 and 3	CONDITIONS 4 and 5
SE	1 ^a	1 ^a
SF	1 ^a	None
RO	2 ^b	1
AO	2 ^b	1
STA	1 ^a	None

WITH UNIT 1 IN CONDITION 4 OR 5 OR DEFUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	CONDITIONS 1, 2 and 3	CONDITIONS 4 and 5
SE	1 ^a	1 ^a
SF	1	None
RO	2	1
AO	2	2 ^b
STA	1	None

Figure 6.1-3 (Continued)

MINIMUM SHIFT CREW COMPOSITION

NOTES

a/ Individual may fill the same position on Unit 1.

b/ One of the two required individuals may fill the same position on Unit 1.

- SE - Shift Supervisor (Shift Engineer) with a Senior Reactor Operators License on Unit 2.
- SF - Shift Foreman with a Senior Reactor Operators License on Unit 2.
- RO - Individual with a Reactor Operators License on Unit 2.
- AO - Auxiliary Operator.
- SCRE - Station Control Room Engineer with a Senior Reactor Operators License.

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Figure 6.1-3 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Figure 6.1-3. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

While the unit is in OPERATIONAL CONDITION 1, 2, or 3, an individual with a valid SRO license shall be designated to assume the Control Room direction function. While the unit is in OPERATIONAL CONDITION 4 or 5, an individual with a valid SRO or RO license shall be designated to assume the Control Room direction function.

ADMINISTRATION CONTROLS

6.1.1 HIGH RADIATION AREAS

6.1.1.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR 20, in lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr* but less than 1000 mrem/hr* shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures, or personnel continuously escorted by such individuals, may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas in which the intensity of radiation is greater than 100 mrem/hr* but less than 1000 mrem/hr*, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. A health physics qualified individual, i.e., qualified in radiation protection procedures, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Health Physicist in the Radiation Work Permit (RWP).

6.1.1.2 In addition to the requirements of 6.1.1.1, above, for areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem*, the computer shall be programmed to permit entry through locked doors for any individual requiring access to any such High-High Radiation Areas for the time that access is required.

6.1.1.3 Keys to manually open computer controlled High Radiation Area doors and High-High Radiation Area doors shall be maintained under the Administration control of the Shift Supervisor on duty and/or the Health Physicist.

6.1.1.4 High-High Radiation areas, as defined in 6.1.1.2 above, not equipped with the computerized card readers shall be maintained in accordance with 10 CFR 20.203 c.2 (iii), locked except during periods when access to the area is required with positive control over each individual entry, or 10 CFR 20.203.c.4. In the case of a High Radiation Area established for a period of 30 days or less, direct surveillance to prevent unauthorized entry may be substituted. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. For

*Measurement made at 18" from source of radioactivity.
LA SALLE - UNIT 2

ADMINISTRATION CONTROLS

HIGH RADIATION AREAS (Continued)

individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem* that are located within large areas, such as the containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, direct or remote, such as use of closed circuit TV cameras, continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

6.2 PLANT OPERATING PROCEDURES AND PROGRAMS

- A. Detailed written procedures including applicable checkoff lists covering items listed below shall be prepared, approved, and adhered to:
1. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
 2. Refueling operations.
 3. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components including responses to alarms, suspected primary system leaks, and abnormal reactivity changes.
 4. Emergency conditions involving potential or actual release of radioactivity - "Generating Stations Emergency Plan" and station emergency and abnormal procedures.
 5. Instrumentation operation which could have an effect on the safety of the facility.
 6. Preventive and corrective maintenance operations which could have an effect on the safety of the facility.
 7. Surveillance and testing requirements.
 8. Tests and experiments.
 9. Procedure to ensure safe shutdown of the plant.
 10. Station Security Plan and implementation procedures.
 11. Fire Protection Program implementation.
 12. Process Control Program implementation.
 13. OFFSITE DOSE CALCULATION MANUAL implementation.
 14. Company Quality Assurance Program and Procedures for effluent and environmental monitoring, using the guidance in Regulatory Guide 4.15, December 1977.

*Measurement made at 18" from source of radioactivity.
LA SALLE - UNIT 2

ADMINISTRATION CONTROLS

PLANT OPERATING PROCEDURES AND PROGRAMS (Continued)

- B. Radiation control procedures shall be maintained, made available to all station personnel, and adhered to. These procedures shall show permissible radiation exposure and shall be consistent with the requirements of 10 CFR 20. This radiation protection program shall be organized to meet the requirements of 10 CFR 20.
- C. 1. Procedures for items identified in Specification 6.2.A and any changes to such procedures shall be reviewed and approved by the Operating Engineer and the Technical Staff Supervisor in the areas of operation, fuel handling, or instrument maintenance, and by the Maintenance Assistant Superintendent and Technical Staff Supervisor in the areas of plant maintenance and plant inspection. Procedures for items identified in Specification 6.2.B and any changes to such procedures shall be reviewed and approved by the Technical Staff Supervisor and the Health Physics Supervisor. At least one person approving each of the above procedures shall hold a valid senior operator's license. In addition, these procedures and changes thereto, must have authorization by the Station Manager before being implemented.
2. Work and instruction type procedures which implement approved maintenance or modification procedures shall be approved and authorized by the Maintenance Assistant Superintendent where the written authority has been provided by the Station Manager. The "Maintenance/Modification Procedure" utilized for safety-related work shall be so approved only if procedures referenced in the "Maintenance/Modification Procedure" have been approved as required by 6.2.A. Procedures which do not fall within the requirements of 6.2.A or 6.2.B may be approved by the Department Heads.
- D. Temporary changes to procedures 6.2.A and 6.2.B above may be made provided:
1. The intent of the original procedure is not altered.
 2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 3. The change is documented, reviewed by the Onsite Review and Investigative Function and approved by the Station Manager within 14 days of implementation.
- E. Drills of the emergency procedures described in Specification 6.2.A.4 shall be conducted at frequencies as specified in the Generating Stations Emergency Plan (GSEP). These drills will be planned so that during the course of the year, communication links are tested and outside agencies are contacted.

ADMINISTRATION CONTROLS

PLANT OPERATING PROCEDURES AND PROGRAMS (Continued)

F. The following programs shall be established, implemented, and maintained:

1. Primary Coolant Sources Outside Primary Containment

A program to reduce leakage from those portions of systems outside primary containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include LPCS, HPCS, RHR/LPCI, RCIC, hydrogen recombiner, process sampling, containment monitoring, and standby gas treatment systems. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements, and
- b. Integrated leak test requirements for each system at refueling cycle intervals or less.

2. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- a. Training of personnel,
- b. Procedures for monitoring, and
- c. Provisions for maintenance of sampling and analysis equipment.

3. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- a. Training of personnel,
- b. Procedures for sampling and analysis,
- c. Provisions for maintenance of sampling and analysis equipment.

6.3 ACTION TO BE TAKEN IN THE EVENT OF A REPORTABLE EVENT IN PLANT OPERATION

The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a Licensee Event Report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed pursuant to Specification 6.1.G.2.c(1).

ADMINISTRATION CONTROLS

6.4 ACTION TO BE TAKEN IN THE EVENT A SAFETY LIMIT IS EXCEEDED

If a safety limit is exceeded, the reactor shall be shut down immediately pursuant to Specification 2.1.1, 2.1.2 and 2.1.3, and critical reactor operation shall not be resumed until authorized by the NRC. The conditions of shutdown shall be promptly reported to the Vice President BWR Operations or his designated alternate. The incident shall be reviewed pursuant to Specifications 6.1.G.1.a and 6.1.G.2.a and a separate Licensee Event Report for each occurrence shall be prepared in accordance with Section 50.73 to 10 CFR Part 50. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President BWR Operations and the Off-site Review and Investigative Function shall be notified within 24 hours.

6.5 PLANT OPERATING RECORDS

- A. Records and/or logs relative to the following items shall be kept in a manner convenient for review and shall be retained for at least 5 years:
1. Records of normal plant operation, including power levels and periods of operation at each power level;
 2. Records of principal maintenance and activities, including inspection and repair, regarding principal items of equipment pertaining to nuclear safety;
 3. Records and reports of reportable events;
 4. Records and periodic checks, inspection and/or calibrations performed to verify that the surveillance requirements (see Section 4 of these specifications) are being met. All equipment failing to meet surveillance requirements and the corrective action taken shall be recorded;
 5. Records of changes to operating procedures;
 6. Shift engineers' logs; and
 7. Byproduct material inventory records and source leak test results.

ADMINISTRATION CONTROLS

PLANT OPERATING RECORDS (Continued)

- B. Records and/or logs relative to the following items shall be recorded in a manner convenient for review and shall be retained for the life of the plant:
1. Substitution or replacement of principal items of equipment pertaining to nuclear safety;
 2. Changes made to the plant as it is described in the SAR;
 3. Records of new and spent fuel inventory and assembly histories;
 4. Updated, corrected, and as-built drawings of the plant;
 5. Records of plant radiation and contamination surveys;
 6. Records of offsite environmental monitoring surveys;
 7. Records of radiation exposure for all plant personnel, including all contractors and visitors to the plant, in accordance with 10 CFR Part 20;
 8. Records of radioactivity in liquid and gaseous wastes released to the environment;
 9. Records of transient or operational cycling for those components that have been designed to operate safely for a limited number of transient or operational cycles (identified in Table 5.7.1-1);
 10. Records of individual staff members indicating qualifications, experience, training, and retraining;
 11. Inservice inspections of the reactor coolant system;
 12. Minutes of meetings and results of reviews and audits performed by the offsite and onsite review and audit functions;
 13. Records of reactor tests and experiments;
 14. Records of Quality Assurance activities required by the QA Manual, except for those items specified in Section 6.5.A;
 15. Records of reviews performed for changes made to procedures on equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
 16. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.9 including the date at which the service life commences and associated installation and maintenance records; and
 17. Records of analyses required by the radiological environmental monitoring program.

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6.6 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

A. Routine Reports

1. Startup Report

A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the startup report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every 3 months until all three events have been completed.

2. Annual Report

A tabulation shall be submitted on an annual basis prior to March 1 of each year of the number of station, utility, and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions (Note: this tabulation supplements the requirements of Section 20.407 of 10 CFR 20), e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignments to various duty functions may be estimated based on pocket dosimeter, TLD, or film badge measurements. Small exposures totaling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

ADMINISTRATION CONTROLS

Annual Report (continued)

The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.4.5 shall be included in the Annual Report along with the following information: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Clean-up system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

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3. Annual Environmental Radiological Operating Report^{3/}

- a. Routine environmental radiological operating reports covering the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year. The initial report shall be submitted prior to May 1 of the year following initial criticality.
- b. The annual environmental radiological operating reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, as appropriate, operational controls, as appropriate, and previous environmental surveillance reports and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of land use censuses required by Specification 3.12.2.

The annual environmental radiological operating reports shall include summarized and tabulated results in the format of Regulatory Guide 4.8, December 1975, of all radiological environmental samples taken during the report period. In the event that some results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

The reports shall also include the following: a summary description of the environmental radiological monitoring program; a map of all sampling locations keyed to a table giving distances and directions from one reactor; and the results of licensee participation in the Interlaboratory Comparison Program, required by Specification 3.12.3.

The report shall include an annual summary of hourly meteorological data collected over the previous year. This annual summary may be either in the form of an hour-by-hour listing of wind speed, wind direction, and atmospheric stability, and precipitation (if measured) on magnetic tape, or in the form of joint frequency distributions of wind speed, wind direction, and atmospheric stability. This same report shall include an assessment of the radiation doses due to the radioactive liquid and gaseous effluents released from the unit or station during the previous calendar year. The assessment of radiation doses shall be performed in accordance with the OFFSITE DOSE CALCULATION MANUAL (ODCM).

The report shall also include an assessment of radiation doses to the likely most exposed member of the public from reactor releases and other nearby uranium fuel cycle sources (including

ADMINISTRATION CONTROLS

Annual Environmental Radiological Operating Report (Continued)

doses from primary effluent pathways and direct radiation) for the previous 12 consecutive months to show conformance with 40 CFR 190, Environmental Radiation Protection Standards for Nuclear Power Operation. The assessment of radiation doses shall be performed in accordance with the ODCM.

4. Semiannual Radioactive Effluent Release Report^{3/}
 - a. Routine radioactive effluent release reports covering the operation of the unit during the previous 6 months of operation shall be submitted within 60 days after January 1 and July 1 of each year. The period of the first report shall begin with the date of initial criticality.
 - b. The radioactive effluent release reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21, "Measuring, Evaluating and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water- Cooled Nuclear Power Plants," Revision 1, June 1974, with data summarized on a quarterly basis following the format of Appendix B thereof.

^{3/} A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.

ADMINISTRATION CONTROLS

Semiannual Radioactive Effluent Release Report (Continued)

The radioactive effluent release report shall include the following information for each type of solid waste shipped offsite during the report period:

- a. Container volume,
- b. Total curie quantity (specify whether determined by measurement or estimate),
- c. Principal radionuclides (specify whether determined by measurement or estimate),
- d. Type of waste (e.g., spent resin, compacted dry waste, evaporator bottoms),
- e. Type of container (e.g., LSA, Type A, Type B, Large Quantity), and
- f. Solidification agent (e.g., cement, urea formaldehyde).

The radioactive effluent release reports shall include unplanned releases from the site to unrestricted areas of radioactive materials in gaseous and liquid effluents on a quarterly basis.

The radioactive effluent release reports shall include any changes to the PROCESS CONTROL PROGRAM (PCP) made during the reporting period.

5. Monthly Operating Report

Routine reports of operating statistics and shutdown experience, including documentation of all challenges to safety/relief valves, shall be submitted on a monthly basis to the Director, Office of Nuclear Reactor Regulation, Mail Station P1-137, US Nuclear Regulatory Commission, Washington, DC 20555, with a copy of the appropriate Regional Office, to arrive no later than the 15th of each month following the calendar month covered by the report.

Any changes to the OFFSITE DOSE CALCULATION MANUAL shall be submitted with the Monthly Operating Report within 90 days in which the change(s) was made effective. In addition, a report of any major changes to the radioactive waste treatment systems shall be submitted with the Monthly Operating Report for the period in which the evaluation was reviewed and accepted by Onsite Review and Investigative Function.

B. Deleted.

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ADMINISTRATION CONTROLS

Semiannual Radioactive Effluent Release Report (Continued)

C. Unique Reporting Requirements

1. Special Reports shall be submitted to the Director of the Office of Inspection and Enforcement (Region III) within the time period specified for each report.

6.7 PROCESS CONTROL PROGRAM (PCP)*

6.7.1 The PCP shall be approved by the Commission prior to implementation.

6.7.2 Licensee-initiated changes to the PCP:

- a. Shall be submitted to the Commission in the semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 1. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 2. A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
 3. Documentation of the fact that the change has been reviewed and found acceptable by the Onsite Review and Investigative Function.
- b. Shall become effective upon review and acceptance by the Onsite Review and Investigative Function.

*The Process Control Program (PCP) is common to La Salle Unit 1 and La Salle Unit 2.

ADMINISTRATION CONTROLS

6.8 OFFSITE DOSE CALCULATION MANUAL (ODCM)*

6.8.1 The ODCM shall be approved by the Commission prior to implementation.

6.8.2 Licensee-initiated changes to the ODCM:

- a. Shall be submitted to the Commission within 90 days of the date the change(s) was made effective. This submittal shall contain:
 1. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluations justifying the change(s);
 2. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
 3. Documentation of the fact that the change has been reviewed and found acceptable by the Onsite Review and Investigative Function.
- b. Shall become effective upon review and acceptance by the Onsite Review and Investigative Function.

6.9 MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS

6.9.1 Licensee initiated major changes to the radioactive waste treatment systems (liquid, gaseous, and solid):

- a. Shall be reported to the Commission in the Monthly Operating Report for the period in which the evaluation was reviewed by the Onsite Review and Investigative Function. The discussion of each change shall contain:
 1. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR 50.59;
 2. Sufficient detailed information to totally support the reason for the change without benefit or additional or supplemental information;
 3. A detailed description of the equipment, components and processes involved and the interfaces with other plant systems;

*The OFFSITE DOSE CALCULATION MANUAL (ODCM) is common to La Salle Unit 1 and La Salle Unit 2.

ADMINISTRATION CONTROLS

MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS (Continued)

4. An evaluation of the change which shows the predicted releases of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments thereto;
 5. An evaluation of the change which shows the expected maximum exposures to individual in the unrestricted area and to the general population that differ from those previously estimated in the license application and amendments thereto;
 6. A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period to when the changes are to be made;
 7. An estimate of the exposure to plant operating personnel as a result of the change; and
 8. Documentation of the fact that the change was reviewed and found acceptable by the Onsite Review and Investigative Function.
- b. Shall become effective upon review and acceptance by the Onsite Review and Investigative Function.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 66 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. NPF-18
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter dated February 17, 1989, Commonwealth Edison Company proposed changes to Technical Specifications (TS) Sections 6.1 through 6.4 and Section 6.6. The proposed changes would remove Figure 6.1-1, Offsite Organization, and Figure 6.1-2, Station Organization, and replace them with a narrative description of the offsite and onsite organizations functional requirements and unit staff qualifications in TS Section 6.1. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter 88-06, dated March 22, 1988. In addition, the proposed changes would change several position titles, responsibilities, and lines of authority to reflect a major reorganization, clarify the distribution requirements for Onsite Reviews and correct several typographical and editorial errors.

2.0 BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Specifications 6.2.1 and 6.2.2 of the administrative control requirements have referenced offsite and unit (onsite) organization charts that are provided as figures to these sections. On a plant specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring of either the offsite or unit organizations, following the issuance of an operating license, has required licensees to submit a license amendment for NRC approval to reflect the desired changes in these organizations. As a consequence, organizational changes have necessitated the need to request an amendment of the operating license.

Removal of organization charts is a line item improvement that was proposed on a lead-plant basis for the Shearon Harris plant and was endorsed by the Westinghouse Owners Group. This change was reviewed as part of the NRC's program for improvements in TS. The objectives of that program were established by the Commission's Interim Policy Statement on Technical Specification Improvements. The staff concluded that removal of organization charts from TS will provide greater flexibility for licensees to implement changes in both the onsite and offsite organizational structure, consistent with Commission policy.

In addition to CECo's intent to delete organization charts, the licensee proposed to revise significant aspects of TS Section 6. This revision would reflect changes in the CECo organization and update station and corporate position titles, description, responsibilities, and lines of authority. The purpose of the organization change was to improve communication channels and eliminate redundant levels of management.

3.0 EVALUATION

The licensee's proposed changes to its TS are in accordance with the guidance provided by Generic Letter 88-06 and addressed the items listed below.

- (1) Specification 6.1 was revised to delete the references to Figures 6.1-1 and 6.1-2 that were removed from the TS.
- (2) Functional requirements of the offsite and onsite organizations were defined and added to Specification 6.1 and they are consistent with the guidance provided in Generic Letter 88-06. The specification notes that implementation of these requirements is documented in the Quality Assurance Manual or the Management Plan for Nuclear Operations, Section 3 Organizational Authority, Activities; Section 6 Interdepartmental Relationships.
- (3) The senior reactor operator license qualified position of Assistant Superintendent Operating was added to Specification 6.1. Therefore, this requirement that was identified on the organization chart for the unit staff will be retained. Other senior reactor operator and reactor operator license qualified positions of the unit staff were already identified in areas of the TS other than Figure 6.1.
- (4) Consistent with requirements to document the offsite and onsite organization relationships in the form of organization charts, the licensee has confirmed that this documentation currently exists in the Quality Assurance Manual of the Management Plan for Nuclear Operations.

On the basis of its review of the above items, the staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in Generic Letter 88-06. In addition to deleting organization charts, changes in on-site and off-site company position titles, responsibilities, description, and lines of authority identified throughout TS Section 6 were made to reflect a major CECo reorganization. Station and corporate positions affecting nuclear operations and management are being reorganized to enhance plant safety by improving channels of communication and authority. The NRC staff concludes that the proposed TS amendments retain all portions of organizational characteristics which are important to safety, and that the re-organizational changes should not adversely impact safe operation and management of the stations. The independence between operations and quality assurance is still maintained. Consequently, these amendments are acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, or administrative procedures or requirements. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

On the basis of the considerations discussed above, the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Paul C. Shemanski, NRR/DRSP

Dated: April 27, 1989