

October 19, 1987

Docket No. 50-373

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Mr. L. D. Butterfield, Jr.
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Butterfield:

Subject: ISSUANCE OF AMENDMENT NO. 51 TO FACILITY OPERATING LICENSE
NO. NPF-11 - LA SALLE COUNTY STATION, UNIT 1
(TAC NO. 65659)

The U. S Nuclear Regulatory Commission has issued the enclosed Amendment No. 51 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated June 16, 1987.

The amendment revises the La Salle County Station, Unit 1 Technical Specifications to perform the required snubber functional testing in approximately 22 months \pm 25% instead of 18 months \pm 25% on a one-time-basis.

A copy of the related Safety Evaluation supporting Amendment No. 51 to Facility Operating License No. NPF-11 is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Paul Shemanski, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 51 to NPF-11
2. Safety Evaluation

cc w/enclosure:
See next page

P.S. For
PDIII-2:LA
LLuther:bj
10/15/87

P.S.
PDIII-2:PM
PShemanski
10/15/87

PDIII-2:PD
DMuller
/ /87

w/ noted change to SE
OGC-Bethesda
Woodhead
10/15/87

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PDR ADDCK 05000373
P PDR

Mr. L. D. Butterfield, Jr.
Commonwealth Edison Company

La Salle County Nuclear Power Station
Units 1 & 2

cc:
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Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated June 16, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

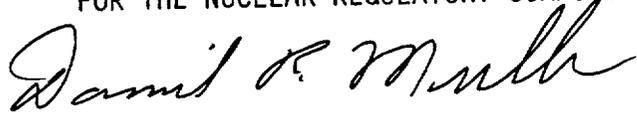
(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 51, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Daniel R. Muller".

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: October 19, 1987

ENCLOSURE TO LICENSE AMENDMENT NO. 51

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 7-28

INSERT

3/4 7-28

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

c. Visual Inspection Acceptance Criteria

Visual inspections shall verify that: there are no visible indications of damage or impaired OPERABILITY and (2) attachments to the foundation or supporting structure are secure, and (3) fasteners for attachment of the snubber to the component and to the snubber anchorage are secure. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, provided that: (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers irrespective of type on that system that may be generically susceptible; and (2) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specification 4.7.9f. All snubbers connected to an inoperable common hydraulic fluid reservoir shall be counted as inoperable snubbers. For those snubbers common to more than one system, the OPERABILITY of such snubbers shall be considered in assessing the surveillance schedule for each of the related systems.

d. Transient Event Inspection

An inspection shall be performed of all hydraulic and mechanical snubbers attached to sections of systems that have experienced unexpected, potentially damaging transients as determined from a review of operational data and a visual inspection of the systems within 6 months following such an event. In addition to satisfying the visual inspection acceptance criteria, freedom-of-motion of mechanical snubbers shall be verified using at least one of the following: (1) manually induced snubber movement; or (2) evaluation of in-place snubber piston setting; or (3) stroking the mechanical snubber through its full range of travel.

e. Functional Tests

During the first refueling shutdown and at least once per 18 months* thereafter during shutdown, a representative sample of snubbers shall be tested using one of the following sample plans. The sample plan shall be selected prior to the test period and cannot be changed during the test period. The NRC Regional Administrator shall be notified in writing of the sample plan selected prior to the test period or the sample plan used in the prior test period shall be implemented:

- 1) At least 10% of the total of each type of snubber shall be functionally tested either in-place or in a bench test. For each snubber of a type that does not meet the functional test acceptance criteria of Specification 4.7.9f., an additional 10% of that type of snubber shall be functionally tested until no more failures are found or until all snubbers of that type have been functionally tested; or
- 2) A representative sample of each type of snubber shall be functionally tested, in accordance with Figure 4.7-1. "C" is the

*The second functional test surveillance shall be conducted 18 months from the commencement of power operation for cycle 2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 51 TO FACILITY OPERATING LICENSE NO. NPF-11
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNIT 1
DOCKET NO. 50-373

1.0 INTRODUCTION

By letter dated June 16, 1987, the Commonwealth Edison Company requested to amend Facility Operating License NPF-11. The proposed revision will permit the LaSalle County Station Unit 1 to perform the required snubber functional tests in approximately 22 months \pm 25% instead of 18 months \pm 25% on a one time basis.

2.0 DISCUSSION

To verify the operability of safety related-snubbers, Technical Specifications (T.S.) of LaSalle 1 require the plant to perform functional testing on safety-related snubbers every 18 \pm 25% months according to any one of the acceptable sampling plans listed. However, due to the extended period of outage after Cycle 1, the next outage will occur well beyond the 18 month interval required for the functional tests. The original 18-month period requirement was arbitrarily determined to accommodate the most common fueling outage.

All safety related snubbers were tested during the surveillance performed at the LSCS-1 first refueling outage, with 104 snubbers failing to meet acceptance limits. Completion of the surveillance, along with the associated corrective actions for the test failures, returned the unit's snubber population to a baseline one hundred percent operable condition.

Certain factors which contributed to the LSCS-1 mechanical snubber failures discovered at the outage were absent, or greatly reduced while the unit was in cold shutdown. The prevailing causes of snubber failures were:

- low frequency steady state vibration of system piping,
- transient forces due to system startup/shutdown,
- overheating of snubber which causes lubricant breakdown, and
- corrosion of snubber internals.

Since a significant portion of the snubber surveillance interval (between the first and second surveillance) was spent with the unit in cold shutdown, which is a period of reduced failure potential for the snubber population, the proposed amendment to extend the surveillance interval should not have a significant impact on snubber operability or their ability to maintain the structural integrity of critical piping systems.

All safety-related mechanical snubbers in LSCS-1 will be visually inspected per surveillance requirement 4.7.9.b during the current second fuel cycle. The snubbers in the primary containment, which are normally inaccessible during reactor power operation, were inspected during a short unit outage in June 1987. The remaining snubbers are scheduled to be inspected in the fall of 1987. Visual inspection of the snubber population provides additional assurance that the snubbers are in good overall condition and operable.

The staff has concluded that the requested Technical Specifications revision of extending the time period between functional tests is acceptable on a one-time basis as requested.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation and use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (52 FR 34001) on September 9, 1987, and consulted with the state of Illinois. No public comments were received, and the state of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCE

Letter from Commonwealth Edison to NRC, "LaSalle County Station, Unit 1, Proposed Amendment to Technical Specification for Facility Operating License NPF-11", June 16, 1987.

Principal Contributor: Horace Shaw, NRR

Dated: October 19, 1987