

July 18, 1990

Docket Nos. 50-373
and 50-374

Mr. Thomas J. Kovach
Nuclear Licensing Manager
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OC/LFMB	PDIII-2 Gray

Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE
NO. NPF-11 AND AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE
NO. NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2
(TAC NOS. 71411 AND 71412)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 74 to Facility Operating License No. NPF-11 and Amendment No. 58 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letter dated November 29, 1988, as supplemented March 15, 1990.

The amendments revise the LaSalle County Station, Units 1 and 2, Technical Specifications 4.8.2.3.2 regarding specific DC system load profiles for each battery and battery charges.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance of these amendments will be published in the Commission's next regular biweekly Federal Register Notice.

Sincerely,

Original Signed By:

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 74 to NPF-11
2. Amendment No. 58 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

DOCUMENT NAME: 71411 AND 71412 AMD

Office:	LA/III-2	PM/PDIII-2	PD/PDIII-2
Surname:	Luther	BPulsifer	RDudley
Date:	5/10/90	6/4/90	6/7/90

Handwritten initials and stamps:
 SELB
 FRosa
 6/7/90
 OGC-WF1
 6/14/90
 7/2/90
 CBach

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 18, 1990

Docket Nos. 50-373
and 50-374

Mr. Thomas J. Kovach
Nuclear Licensing Manager
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SUBJECT: ISSUANCE OF AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE
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A copy of the related Safety Evaluation is enclosed. Notice of Issuance of these amendments will be published in the Commission's next regular biweekly Federal Register Notice.

Sincerely,

A handwritten signature in dark ink, appearing to read "Robert M. Pulsifer".

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 74 to NPF-11
2. Amendment No. 58 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

Mr. Thomas J. Kovach
Commonwealth Edison Company

LaSalle County Nuclear Power Station
Unit Nos. 1 & 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated November 29, 1988, and supplemented on March 15, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jacob F. Wechselberger, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 18, 1990

ENCLOSURE TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 8-14
3/4 8-15
3/4 8-16
3/4 8-17
B 3/4 8-3

INSERT

3/4 8-14
3/4 8-15
3/4 8-16
3/4 8-17
B 3/4 8-3

ELECTRICAL POWER SYSTEMS

D.C. DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.2.3 The following D.C. distribution system electrical divisions shall be OPERABLE and energized:

- a. Division 1, consisting of;
 - 1. 125-volt battery 1A.
 - 2. 125-volt full capacity charger.
 - 3. 125-volt distribution panel 111Y.

- b. Division 2, consisting of;
 - 1. 125-volt battery 1B.
 - 2. 125-volt full capacity charger.
 - 3. 125-volt distribution panel 112Y.

- c. Division 3, consisting of;
 - 1. 125-volt battery 1C.
 - 2. 125-volt full capacity charger.
 - 3. 125-volt distribution panel 113.

- d. Unit 2 Division 2, consisting of;
 - 1. 125-volt battery 2B.
 - 2. 125-volt full capacity charger.
 - 3. 125-volt distribution panel 212Y.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With either Division 1 or Division 2 inoperable or not energized, restore the inoperable division to OPERABLE and energized status within 2 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

- b. With Division 3 inoperable or not energized, declare the HPCS system inoperable and take the ACTION required by Specification 3.5.1.

- c. With Unit 2 Division 2 inoperable or not energized, restore the inoperable division to OPERABLE and energized status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS

4.8.2.3.1 Each of the above required D.C. distribution system electrical divisions shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment, indicated power availability from the charger and battery, and voltage on the panel with an overall voltage of greater than or equal to 125 volts.

4.8.2.3.2 Each 125-volt battery and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
 1. The parameters in Table 4.8.2.3.2-1 meet the Category A limits, and
 2. Total battery terminal voltage is greater than or equal to 128 volts on float charge.
- b. At least once per 92 days and within 7 days after a battery discharge with battery voltage below 110 volts, or battery overcharge with battery terminal voltage above 150 volts, by verifying that:
 1. The parameters in Table 4.8.2.3.2-1 meet the Category B limits,
 2. There is no visible corrosion at either terminals or connectors, or the connection resistance of these items is less than 150×10^{-6} ohm, and
 3. The average electrolyte temperature of at least 10 connected cells is above 60°F.
- c. At least once per 18 months by verifying that:
 1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration,
 2. The cell-to-cell and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material,
 3. The resistance of each cell and terminal connection is less than or equal to 150×10^{-6} ohm, and
 4. The battery charger will supply a load equal to the manufacturer's rating for at least 8 hours.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 18 months, during shutdown, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual or simulated emergency loads for the 240 minute design cycle when the battery is subjected to a battery service test.
- e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturers rating when subjected to a performance discharge test. Once per 60 month interval, this performance discharge test may be performed in lieu of the battery service test.
- f. Annual performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.

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ELECTRICAL POWER SYSTEMS

BASES

A.C. SOURCES AND ONSITE POWER DISTRIBUTION SYSTEMS (Continued)

specific gravity, ensures that the decrease in rating will be less than the safety margin provided in sizing; (3) the allowable value for an individual cell's specific gravity ensures that an individual cell's specific gravity will not be more than 0.040 below the manufacturer's full charge specific gravity and that the overall capability of the battery will be maintained within an acceptable limit; and (4) the allowable value for an individual cell's float voltage, greater than 2.07 volts, ensures the battery's capability to perform its design function.

The battery load profile and battery charger specifications will be maintained and are located in Chapter 8, "Electrical Power", section of the Updated Final Safety Analysis Report, UFSAR.

3/4.8.3 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

Primary containment electrical penetrations and penetration conductors are protected by either de-energizing circuits not required during reactor operation or demonstrating the OPERABILITY of primary and backup overcurrent protection circuit breakers by periodic surveillance.

The surveillance requirements applicable to lower voltage circuit breakers and fuses provides assurance of breaker and fuse reliability by testing at least one representative sample of each manufacturers brand of circuit breaker and/or fuse. Each manufacturer's molded case and metal case circuit breakers and/or fuses are grouped into representative samples which are then tested on a rotating basis to ensure that all breakers and/or fuses are tested. If a wide variety exists within any manufacturer's brand of molded case circuit breakers and/or fuses, it is necessary to divide that manufacturer's breakers and/or fuses into groups and treat each group as a separate type of breaker or fuses for surveillance purposes.

The bypassing of the motor operated valves thermal overload protection continuously or during accident conditions by integral bypass devices ensures that the thermal overload protection will not prevent safety related valves from performing their function. The Surveillance Requirements for demonstrating the bypassing of the thermal overload protection continuously and during accident conditions are in accordance with Regulatory Guide 1.106 "Thermal Overload Protection for Electric Motors on Motor Operated Valves", Revision 1, March 1977.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. NPF-18

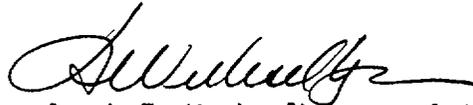
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated November 29, 1988, and supplemented on March 15, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 58, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jacob F. Wechselberger, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 18, 1990

ENCLOSURE TO LICENSE AMENDMENT NO. 58

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 8-15
3/4 8-16
3/4 8-17
B 3/4 8-3

INSERT

3/4 8-15
3/4 8-16
3/4 8-17
B 3/4 8-3

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS

4.8.2.3.1 Each of the above required D.C. distribution system electrical divisions shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment, indicated power availability from the charger and battery, and voltage on the panel with an overall voltage of greater than or equal to 125 volts.

4.8.2.3.2 Each 125-volt battery and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
 1. The parameters in Table 4.8.2.3.2-1 meet the Category A limits, and
 2. Total battery terminal voltage is greater than or equal to 128 volts on float charge.
- b. At least once per 92 days and within 7 days after a battery discharge with battery voltage below 110 volts, or battery overcharge with battery terminal voltage above 150 volts, by verifying that:
 1. The parameters in Table 4.8.2.3.2-1 meet the Category B limits,
 2. There is no visible corrosion at either terminals or connectors, or the connection resistance of these items is less than 150×10^{-6} ohm, and
 3. The average electrolyte temperature of at least 10 connected cells is above 60°F.
- c. At least once per 18 months by verifying that:
 1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration,
 2. The cell-to-cell and terminal connections are clean, tight, free of corrosion, and coated with anticorrosion material,
 3. The resistance of each cell and terminal connection is less than or equal to 150×10^{-6} ohm, and
 4. The battery charger will supply a load equal to the manufacturer's rating for at least 8 hours.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 18 months, during shutdown, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual or simulated emergency loads for the 240 minute design cycle when the battery is subjected to a battery service test.
- e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturers rating when subjected to a performance discharge test. Once per 60-month interval, this performance discharge test may be performed in lieu of the battery service test.
- f. Annual performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.

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ELECTRICAL POWER SYSTEMS

BASES

A.C. SOURCES AND ONSITE POWER DISTRIBUTION SYSTEMS (Continued)

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The battery load profile and battery charger specifications will be maintained and are located in Chapter 8, "Electrical Power", section of the Updated Final Safety Analysis Report, UFSAR.

3/4.8.3 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

Primary containment electrical penetrations and penetration conductors are protected by either de-energizing circuits not required during reactor operation or demonstrating the OPERABILITY of primary and backup overcurrent protection circuit breakers by periodic surveillance.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. NPF-18
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter dated November 29, 1988 and supplemented March 15, 1990 (W. E. Morgan, CECO, to USNRC), Commonwealth Edison Company (the licensee), proposed a Technical Specification change to delete specific load profiles for each battery and battery charger for LaSalle County Station, Units 1 and 2. Listings of DC bus load profiles will be maintained in the Updated Final Safety Analysis Report (UFSAR).

2.0 EVALUATION

Specific load profiles for the batteries and battery chargers in the Technical Specifications need to be revised by a supplement each time modifications are added to the DC loads. This change eliminates the need to revise the Technical Specifications every time a DC system load profile change is made by removing the detailed listing of the load profile from the Technical Specifications. The DC bus load profiles are presently listed in Tables 8.3-12 through 8.3-14 of the UFSAR. The battery charger specifications are listed in Section 8.3.2.1.1, Unit Class 1E D-C Power System, of the UFSAR. The licensee annually updates the UFSAR in accordance with 10 CFR 50.71(e). After discussions with the licensee, a statement was added to the Bases to indicate that the battery load profile and battery charger specifications will be maintained and are located in the UFSAR. The licensee is obligated under 10 CFR 50.59 to have Commission approval if a change in the facility as described in the safety analysis report involves an unreviewed safety question.

This change would make the Technical Specifications similar to the Byron Station Technical Specifications.

The revised pages also reflect format changes to make the Unit 2 Technical Specification pages similar to the Unit 1 pages.

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3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (53 FR 53090) on December 30, 1988. No public comments were received, and the State of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Pulsifer, NRR/PDIII-2

Dated: July 18, 1990