

November 17, 1988

Docket Nos: 50-373  
and 50-374

Mr. Henry E. Bliss  
Nuclear Licensing Manager  
Commonwealth Edison Company  
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Chicago, Illinois 60690

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Dear Mr. Bliss:

SUBJECT: CORRECTIONS TO AMENDMENT NOS. 59 AND 60 TO FACILITY OPERATING LICENSE NO. NPF-11 LASALLE COUNTY STATION, UNIT 1 AND AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-18 - LASALLE COUNTY STATION, UNIT 2

On August 31, 1988 and September 7, 1988, the Nuclear Regulatory Commission issued Amendment Nos. 59/39 and 60, respectively. Amendment Nos. 59/39 revised the LaSalle County Station, Units 1 and 2 Technical Specifications to correct inconsistent requirements regarding the suppression pool high level alarm. Amendment No. 60 revised the LaSalle County Station, Unit 1 Technical Specifications by providing additional requirements for monitoring core performance and other actions to be taken by the reactor operator in the high power/low flow region of the power to flow map.

Amendment Nos. 59/39 and 60 contained several errors on the revised Technical Specifications. A copy of the corrected Technical Specification change pages to Amendment Nos. 59/39 and 60 is enclosed.

Sincerely,

*Paul Shemanski For*

Daniel R. Muller, Director  
Project Directorate III-2  
Division of Reactor Projects III,  
IV, V, and Special Projects

Enclosures:  
As stated

cc w/enclosure:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

November 7, 1988

Docket Nos: 50-373  
and 50-374

Mr. Henry E. Bliss  
Nuclear Licensing Manager  
Commonwealth Edison Company  
P.O. Box 767  
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: CORRECTIONS TO AMENDMENT NOS. 59 AND 60 TO FACILITY OPERATING  
LICENSE NO. NPF-11 LASALLE COUNTY STATION, UNIT 1 AND AMENDMENT  
NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-18 - LASALLE COUNTY  
STATION, UNIT 2

On August 31, 1988 and September 7, 1988, the Nuclear Regulatory Commission issued Amendment Nos. 59/39 and 60, respectively. Amendment Nos. 59/39 revised the LaSalle County Station, Units 1 and 2 Technical Specifications to correct inconsistent requirements regarding the suppression pool high level alarm. Amendment No. 60 revised the LaSalle County Station, Unit 1 Technical Specifications by providing additional requirements for monitoring core performance and other actions to be taken by the reactor operator in the high power/low flow region of the power to flow map.

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Sincerely,

*Paul S. Semarski FOR*

Daniel R. Muller, Director  
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Enclosures:  
As stated

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ENCLOSURE TO LICENSE AMENDMENT NO. 39

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 5-8

3/4 5-9

INSERT

3/4 5-8

3/4 5-9

EMERGENCY CORE COOLING SYSTEMS

3/4.5.3 SUPPRESSION CHAMBER#

LIMITING CONDITION FOR OPERATION

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3.5.3 The suppression chamber shall be OPERABLE:

- a. In OPERATIONAL CONDITION 1, 2, or 3 with a contained water volume of at least 128,800 ft<sup>3</sup>, equivalent to a level of -4 1/2 inches.\*\*
- b. In OPERATIONAL CONDITION 4 or 5\* with a contained water volume of at least 70,000 ft<sup>3</sup>, equivalent to a level of -12 feet 7 inches\*\*, except that the suppression chamber level may be less than the limit or may be drained in OPERATIONAL CONDITION 4 or 5\* provided that:
  1. No operations are performed that have a potential for draining the reactor vessel,
  2. The reactor mode switch is locked in the Shutdown or Refuel position,
  3. The condensate storage tank contains at least 135,000 available gallons of water, equivalent to a level of 14.5 feet, and
  4. The HPCS system is OPERABLE per Specification 3.5.2 with an OPERABLE flow path capable of taking suction from the condensate storage tank and transferring the water through the spray sparger to the reactor vessel.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4, and 5\*.

ACTION:

- a. In OPERATIONAL CONDITION 1, 2, or 3 with the suppression chamber water level less than the above limit, restore the water level to within the limit within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION 4 or 5\* with the suppression chamber water level less than the above limit or drained and the above required conditions not satisfied, suspend CORE ALTERATIONS and all operations that have a potential for draining the reactor vessel and lock the reactor mode switch in the Shutdown position. Establish SECONDARY CONTAINMENT INTEGRITY within 8 hours.

---

#See Specification 3.6.2.1 for pressure suppression requirements.

\*The suppression chamber is not required to be OPERABLE provided that the reactor vessel head is removed, the cavity is flooded or being flooded from the suppression pool, the spent fuel pool gates are removed when the cavity is flooded, and the water level is maintained within the limits of Specifications 3.9.8 and 3.9.9.

\*\*Level is referenced to a plant elevation of 699 feet 11 inches (see Figure B 3/4.6.2-1).

## EMERGENCY CORE COOLING SYSTEMS

### LIMITING CONDITION FOR OPERATION (Continued)

#### ACTION: (Continued)

- c. With one suppression chamber water level instrumentation channel inoperable, restore the inoperable channel to OPERABLE status within 7 days or verify the suppression chamber water level to be greater than or equal to -4 1/2 inches\*\* or -12 feet 7 inches\*\*, as applicable, at least once per 12 hours by local indication.
- d. With both suppression chamber water level instrumentation channels inoperable, restore at least one inoperable channel to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours and verify the suppression chamber water level to be greater than or equal to -4 1/2 inches\*\* or -12 feet 7 inches\*\*, as applicable, at least once per 12 hours by local indication.

#### SURVEILLANCE REQUIREMENTS

4.5.3.1 The suppression chamber shall be determined OPERABLE by verifying:

- a. The water level to be greater than or equal to, as applicable:
  1. -4 1/2 inches\*\* at least once per 24 hours.
  2. -12 feet 7 inches\*\* at least once per 12 hours.
- b. Two suppression chamber water level instrumentation channels OPERABLE by performance of a:
  1. CHANNEL CHECK at least once per 24 hours,
  2. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
  3. CHANNEL CALIBRATION at least once per 18 months,with the low water level alarm setpoint at greater than or equal to -3 inches.\*\*

4.5.3.2 With the suppression chamber level less than the above limit or drained in OPERATIONAL CONDITION 4 or 5\*, at least once per 12 hours:

- a. Verify the required conditions of Specification 3.5.3.b. to be satisfied, or
- b. Verify footnote conditions\* to be satisfied.

---

\*The suppression chamber is not required to be OPERABLE provided that the reactor vessel head is removed, the cavity is flooded or being flooded from the suppression pool, the spent fuel pool gates are removed when the cavity is flooded, and the water level is maintained within the limits of Specifications 3.9.8 and 3.9.9.

\*\*Level is referenced to a plant elevation of 699 feet 11 inches (See Figure B 3/4.6.2-1).

ENCLOSURE TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

XIX

B 3/4 5-2

INSERT

XIX

B 3/4 5-2

## INDEX

### LIST OF FIGURES

<u>FIGURE</u>		<u>PAGE</u>
3.1.5-1	SODIUM PENTABORATE SOLUTION TEMPERATURE/ CONCENTRATION REQUIREMENTS .....	3/4 1-21
3.1.5-2	SODIUM PENTABORATE ( $\text{Na}_2\text{B}_{10}\text{O}_{16} \cdot 10 \text{H}_2\text{O}$ ) VOLUME/CONCENTRATION REQUIREMENTS .....	3/4 1-22
3.2.1-1	MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS AVERAGE PLANAR EXPOSURE, INITIAL CORE FUEL TYPES 8CRB176, 8CRB219, and 8CRB071 .....	3/4 2-2
3.2.1-2	MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS AVERAGE PLANAR EXPOSURE, FUEL TYPE BP8CRB299L .....	3/4 2-2a
3.2.1-3	MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS AVERAGE PLANAR EXPOSURE, FUEL TYPES BC301A AND BC320B .....	3/4 2-2b
3.2.3-1a	MINIMUM CRITICAL POWER RATIO (MCPR) VERSUS $t$ AT RATED FLOW .....	3/4 2-5
3.2.3-1b	MINIMUM CRITICAL POWER RATIO (MCPR) VERSUS $t$ AT RATED FLOW FOR END OF CYCLE RECIRCULATION PUMP TRIP AND MAIN TURBINE BYPASS SYSTEMS INOPERABLE ...	3/4 2-5a
3.2.3-2	$K_f$ FACTOR .....	3/4 2-6
3.4.1.1-1	CORE THERMAL POWER (% OF RATED) VERSUS TOTAL CORE FLOW (% OF RATED) .....	3/4 4-1b
3.4.6.1-1	MINIMUM REACTOR VESSEL METAL TEMPERATURE VS. REACTOR VESSEL PRESSURE .....	3/4 4-18
4.7-1	SAMPLE PLAN 2) FOR SNUBBER FUNCTIONAL TEST .....	3/4 7-32
B 3/4 3-1	REACTOR VESSEL WATER LEVEL .....	B 3/4 3-7
B 3/4.4.6-1	CALCULATED FAST NEUTRON FLUENCE ( $E > 1\text{MeV}$ ) at $1/4 T$ AS A FUNCTION OF SERVICE LIFE .....	B 3/4 4-7
B 3/4.6.2-1	SUPPRESSION POOL LEVEL SETPOINTS .....	B 3/4 6-3a
5.1.1-1	EXCLUSION AREA AND SITE BOUNDARY FOR GASEOUS AND LIQUID EFFLUENTS .....	5-2
5.1.2-1	LOW POPULATION ZONE .....	5-3



## EMERGENCY CORE COOLING SYSTEMS

### BASES

#### ECCS-OPERATING and SHUTDOWN (Continued)

the suppression pool into the reactor, but no credit is taken in the hazards analyses for the condensate storage tank water.

With the HPCS system inoperable, adequate core cooling is assured by the OPERABILITY of the redundant and diversified automatic depressurization system and both the LPCS and LPCI systems. In addition, the reactor core isolation cooling (RCIC) system, a system for which no credit is taken in the hazards analysis, will automatically provide makeup at reactor operating pressures on a reactor low water level condition. The HPCS out-of-service period of 14 days is based on the demonstrated OPERABILITY of redundant and diversified low pressure core cooling systems.

The surveillance requirements provide adequate assurance that the HPCS system will be OPERABLE when required. Although all active components are testable and full flow can be demonstrated by recirculation through a test loop during reactor operation, a complete functional test with reactor vessel injection requires reactor shutdown. The pump discharge piping is maintained full to prevent water hammer damage and to provide cooling at the earliest moment.

Upon failure of the HPCS system to function properly, if required, the automatic depressurization system (ADS) automatically causes selected safety-relief valves to open, depressurizing the reactor so that flow from the low pressure core cooling systems can enter the core in time to limit fuel cladding temperature to less than 2200°F. ADS is conservatively required to be OPERABLE whenever reactor vessel pressure exceeds 122 psig even though low pressure core cooling systems provide adequate core cooling up to 350 psig.

ADS automatically controls seven selected safety-relief valves. Six valves are required to be OPERABLE since the LOCA analysis assumes 6 ADS valves in addition to a single failure. It is therefore appropriate to permit one of the required valves to be out-of-service for up to 14 days without materially reducing system reliability.

#### 3/4.5.3 SUPPRESSION CHAMBER

The suppression chamber is also required to be OPERABLE as part of the ECCS to ensure that a sufficient supply of water is available to the HPCS, LPCS and LPCI systems in the event of a LOCA. This limit on suppression chamber minimum water volume ensures that sufficient water is available to permit recirculation cooling flow to the core (See Figure B 3/4.6.2-1). The OPERABILITY of the suppression chamber in OPERATIONAL CONDITIONS 1, 2 or 3 is required by Specification 3.6.2.1.

Repair work might require making the suppression chamber inoperable. This specification will permit those repairs to be made and at the same time give assurance that the irradiated fuel has an adequate cooling water supply when the suppression chamber must be made inoperable, including draining, in OPERATIONAL CONDITION 4 or 5.

In OPERATIONAL CONDITION 4 and 5 the suppression chamber minimum required water volume is reduced because the reactor coolant is maintained at or below 200°F. Since pressure suppression is not required below 212°F, the minimum water volume is based on NPSH, recirculation volume, vortex prevention plus a 2'-4" safety margin for conservatism.

ENCLOSURE TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

REMOVE

XIX

INSERT

XIX

INDEX

LIST OF FIGURES

<u>FIGURE</u>		<u>PAGE</u>
3.1.5-1	SODIUM PENTABORATE SOLUTION TEMPERATURE/ CONCENTRATION REQUIREMENTS .....	3/4 1-21
3.1.5-2	SODIUM PENTABORATE ( $\text{Na}_2\text{B}_{10}\text{O}_{16} \cdot 10 \text{H}_2\text{O}$ ) VOLUME/CONCENTRATION REQUIREMENTS .....	3/4 1-22
3.2.1-1	MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS AVERAGE PLANAR EXPOSURE, INITIAL CORE FUEL TYPES 8CRB176, 8CRB219, and 8CRB071 .....	3/4 2-2
3.2.1-2	MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS AVERAGE PLANAR EXPOSURE, FUEL TYPE BP8CRB299L .....	3/4 2-2a
3.2.1-3	MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS AVERAGE PLANAR EXPOSURE, FUEL TYPES BC301A AND BC320B .....	3/4 2-2b
3.2.3-1a	MINIMUM CRITICAL POWER RATIO (MCPR) VERSUS $t$ AT RATED FLOW .....	3/4 2-5
3.2.3-1b	MINIMUM CRITICAL POWER RATIO (MCPR) VERSUS $t$ AT RATED FLOW FOR END OF CYCLE RECIRCULATION PUMP TRIP AND MAIN TURBINE BYPASS SYSTEMS INOPERABLE ...	3/4 2-5a
3.2.3-2	$K_f$ FACTOR .....	3/4 2-6
3.4.1.5-1	CORE THERMAL POWER (% OF RATED) VERSUS TOTAL CORE FLOW (% OF RATED) .....	3/4 4-4c
3.4.6.1-1	MINIMUM REACTOR VESSEL METAL TEMPERATURE VS. REACTOR VESSEL PRESSURE .....	3/4 4-18
4.7-1	SAMPLE PLAN 2) FOR SNUBBER FUNCTIONAL TEST .....	3/4 7-32
B 3/4 3-1	REACTOR VESSEL WATER LEVEL .....	B 3/4 3-7
B 3/4.4.6-1	CALCULATED FAST NEUTRON FLUENCE ( $E > 1\text{MeV}$ ) at $1/4 T$ AS A FUNCTION OF SERVICE LIFE .....	B 3/4 4-7
B 3/4.6.2-1	SUPPRESSION POOL LEVEL SETPOINTS .....	B 3/4 6-3a
5.1.1-1	EXCLUSION AREA AND SITE BOUNDARY FOR GASEOUS AND LIQUID EFFLUENTS .....	5-2
5.1.2-1	LOW POPULATION ZONE .....	5-3