

February 12, 1991

DISTRIBUTION

Docket Nos. 50-373  
and 50-374

Mr. Thomas J. Kovach  
Nuclear Licensing Manager  
Commonwealth Edison Company-Suite 300  
OPUS West III  
1400 OPUS Place  
Downers Grove, Illinois 60515

Docket Files NRC & Local PDRs  
PDIII-2 r/f BBoger, 14A2  
JZwolinski CMOore  
JHickman DHagan, MNBB 3206  
OGC-WF1 GHill(8), P1 37  
JCalvo, 11F24 WJones, MNBB 7112  
ACRS(10) GPA/PA  
OC/LFMB Plant File

Dear Mr. Kovach:

SUBJECT: AMENDMENT NO. 71 TO FACILITY OPERATING LICENSE NO. NPF-11  
AND AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. NPF-18 -  
LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC NOS. 74516 AND 74517)

By letter dated January 16, 1990, the NRC issued Amendment No. 71 to Facility Operating License No. NPF-11 and Amendment No. 55 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. Amendment No. 71 for LaSalle Unit 1 contained an error in the footnote on page 3/4 4-16. The footnote should have referred to Table 1.2 rather than 2.1. Amendment No. 55 for LaSalle Unit 2 contained errors on two revised Technical Specification pages. The same error noted for Unit 1 was made on page 3/4 4-17 for Unit 2 and four temperatures listed on page 3/4 4-17 and 3/4 4-18 for Unit 2 were unintentionally revised to the same values as for Unit 1. Corrected pages are enclosed.

Sincerely,

Original Signed By:

John B. Hickman, Project Manager  
Project Directorate III-2  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc w/enclosure:  
See next page

OFC	: LA: PDIII-2	: PM: PDIII-2	: D: PDIII-2	:	:
NAME	: CMOORE	: JHICKMAN: ta	: RBARRETT	:	:
DATE	: 2/11/91	: 2/11/91	: 2/12/91	:	:

OFFICIAL RECORD COPY  
Document Name: CORRECTION AMEND 74516/17

9102180179 910212  
PDR ADOCK 05000373  
P PDR

*QFOI  
1/1*

Mr. Thomas J. Kovach  
Commonwealth Edison Company

LaSalle County Station  
Unit Nos. 1 and 2

cc:

Phillip P. Steptoe, Esquire  
Sidley and Austin  
One First National Plaza  
Chicago, Illinois 60603

Robert Cushing  
Chief, Public Utilities Division  
Illinois Attorney General's Office  
100 West Randolph Street  
Chicago, Illinois 60601

Assistant Attorney General  
100 West Randolph Street  
Suite 12  
Chicago, Illinois 60601

Michael I. Miller, Esq.  
Sidley and Austin  
One First National Plaza  
Chicago, Illinois 60690

Resident Inspector/LaSalle, NPS  
U.S. Nuclear Regulatory Commission  
Rural Route No. 1  
P. O. Box 224  
Marseilles, Illinois 61341

Chairman  
LaSalle County Board of Supervisors  
LaSalle County Courthouse  
Ottawa, Illinois 61350

Attorney General  
500 South 2nd Street  
Springfield, Illinois 62701

Chairman  
Illinois Commerce Commission  
Leland Building  
527 East Capitol Avenue  
Springfield, Illinois 62706

Illinois Department of Nuclear Safety  
Office of Nuclear Facility Safety  
1035 Outer Park Drive  
Springfield, Illinois 62704

Regional Administrator, Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road, Bldg. #4  
Glen Ellyn, Illinois 60137

Robert Neumann  
Office of Public Counsel  
State of Illinois Center  
100 W. Randolph  
Suite 11-300  
Chicago, Illinois 60601

## REACTOR COOLANT SYSTEM

### 3/4.4.6 PRESSURE/TEMPERATURE LIMITS

## REACTOR COOLANT SYSTEM

### LIMITING CONDITION FOR OPERATION

---

---

3.4.6.1 The reactor coolant system temperature and pressure shall be limited in accordance with the limit lines shown on Figures 3.4.6.1-1 and 3.4.6.1-1a (1) curves A for hydrostatic or leak testing; (2) curves B for heatup by non-nuclear means, cooldown following a nuclear shutdown and low power PHYSICS TESTS; and (3) curves C for operations with a critical core other than low power PHYSICS TESTS, with:

- a. A maximum heatup of 100°F in any one hour period,
- b. A maximum cooldown of 100°F in any one hour period,
- c. A maximum temperature change of less than or equal to 20°F in any one hour period during inservice hydrostatic and leak testing operations above the heatup and cooldown limit curves, and
- d. The reactor vessel flange and head flange temperature greater than or equal to 80°F when reactor vessel head bolting studs are under tension.

APPLICABILITY: At all times.\*

#### ACTION:

With any of the above limits exceeded, restore the temperature and/or pressure to within the limits within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the reactor coolant system; determine that the reactor coolant system remains acceptable for continued operations or be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

### SURVEILLANCE REQUIREMENTS

---

---

4.4.6.1.1 During system heatup, cooldown and inservice leak and hydrostatic testing operations, the reactor coolant system temperature and pressure shall be determined to be within the above required heatup and cooldown limits and to the right of the limit lines of Figures 3.4.6.1-1 and 3.4.6.1-1a curves A or B, as applicable, at least once per 30 minutes.

\*During shutdown conditions for hydrostatic or leak testing or heatup by nonnuclear means the average coolant temperature limit of Table 1.2 for Cold Shutdown and Hot Shutdown may be increased to 212°F.

## REACTOR COOLANT SYSTEM

### 3/4.4.6 PRESSURE/TEMPERATURE LIMITS

## REACTOR COOLANT SYSTEM

### LIMITING CONDITION FOR OPERATION

---

3.4.6.1 The reactor coolant system temperature and pressure shall be limited in accordance with the limit lines shown on Figures 3.4.6.1-1 and 3.4.6.1-1a; (1) curves A for hydrostatic or leak testing; (2) curves B for heatup by non-nuclear means, cooldown following a nuclear shutdown and low power PHYSICS TESTS; and (3) curves C for operations with a critical core other than low power PHYSICS TESTS, with:

- a. A maximum heatup of 100°F in any one hour period,
- b. A maximum cooldown of 100°F in any one hour period,
- c. A maximum temperature change of less than or equal to 20°F in any one hour period during inservice hydrostatic and leak testing operations above the heatup and cooldown limit curves, and
- d. The reactor vessel flange and head flange temperature greater than or equal to 86°F when reactor vessel head bolting studs are under tension.

APPLICABILITY: At all times.\*

#### ACTION:

With any of the above limits exceeded, restore the temperature and/or pressure to within the limits within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the reactor coolant system; determine that the reactor coolant system remains acceptable for continued operations or be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the following 24 hours.

### SURVEILLANCE REQUIREMENTS

---

4.4.6.1.1 During system heatup, cooldown and inservice leak and hydrostatic testing operations, the reactor coolant system temperature and pressure shall be determined to be within the above required heatup and cooldown limits and to the right of the limit lines of Figures 3.4.6.1-1 and 3.4.6.1-1a curves A or B, as applicable, at least once per 30 minutes.

---

\*During shutdown conditions for hydrostatic or leak testing or heatup by nonnuclear means, the average coolant temperature limit of Table 1.2 for Cold Shutdown and Hot Shutdown may be increased to 212°F.

## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS (Continued)

---

4.4.6.1.2 The reactor coolant system temperature and pressure shall be determined to be to the right of the criticality limit line of Figures 3.4.6.1-1 and 3.4.6.1-1a curves C within 15 minutes prior to the withdrawal of control rods to bring the reactor to criticality.

4.4.6.1.3 The reactor vessel material specimens shall be removed and examined to determine reactor pressure vessel fluence as a function of time and THERMAL POWER as required by 10 CFR Part 50, Appendix H in accordance with the schedule in Table 4.4.6.1.3-1. The results of these fluence determinations shall be used to update the curves of Figures 3.4.6.1-1 and 3.4.6.1-1a.

4.4.6.1.4 The reactor vessel flange and head flange temperature shall be verified to be greater than or equal to 86°F:

- a. In OPERATIONAL CONDITION 4 when the reactor coolant temperature is:
  1.  $\leq 106^{\circ}\text{F}$ , at least once per 12 hours.
  2.  $\leq 91^{\circ}\text{F}$ , at least once per 30 minutes.
- b. Within 30 minutes prior to and at least once per 30 minutes during tensioning of the reactor vessel head bolting studs.