

February 7, 1988

Docket Nos.: 50-373 and 50-374

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

DISTRIBUTION

Docket file	TMeek (8)
NRC & Local PDRs	WJones
PDIII-2 r/f	EButcher
GHolahan	DHagan
MVirgilio	ACRS (10)
LLuther	GPA/PA
DMuller	ARM/LFMB
EJordan	Plant file
BGrimes	HKnox
PShemanski	OGC-WF

SUBJECT: ISSUANCE OF AMENDMENT NOS. 63 AND 44 TO FACILITY OPERATING LICENSE NOS. NPF-11 AND NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC NOS. 64461 AND 64462)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. NPF-11 and Amendment No. 44 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letter dated January 19, 1987 supplemented February 24, 1987 and May 24, 1988.

The amendments revise the LaSalle County Station, Units 1 and 2 Technical Specifications to allow the continued operation of one unit for a period of 7 days, while the common plant Division 1 diesel-generator, "0" is out of service for the performance of certain Technical Specification surveillance requirements.

A copy of the related Safety Evaluation supporting Amendment No. 63 to Facility Operating License No. NPF-11 and Amendment No. 44 to Facility Operating License No. NPF-18 is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Paul C. Shemanski, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 63 to NPF-11
2. Amendment No. 44 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

8902100194 890207
PDR ADDCK 05000373
PIC

PDIII-P.S.	PDIII-2	OGC	PDIII
PShemanski:km	LLuther	1/30/89	DMuller
1/23/89	1/19/89		1/25/89

DF=01
1/11



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 7, 1989

Docket Nos.: 50-373 and 50-374

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: ISSUANCE OF AMENDMENT NOS. 63 AND 44 TO FACILITY OPERATING LICENSE NOS. NPF-11 AND NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC NOS. 64461 AND 64462)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. NPF-11 and Amendment No. 44 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letter dated January 19, 1987 supplemented February 24, 1987 and May 24, 1988.

The amendments revise the LaSalle County Station, Units 1 and 2 Technical Specifications to allow the continued operation of one unit for a period of 7 days, while the common plant Division 1 diesel-generator, "0" is out of service for the performance of certain Technical Specification surveillance requirements.

A copy of the related Safety Evaluation supporting Amendment No. 63 to Facility Operating License No. NPF-11 and Amendment No. 44 to Facility Operating License No. NPF-18 is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Paul C. Shemanski".

Paul C. Shemanski, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 63 to NPF-11
2. Amendment No. 44 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

Mr. Henry E. Bliss
Commonwealth Edison Company

LaSalle County Nuclear Power Station
Units 1 & 2

cc:

Philip P. Steptoe, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60603

John W. McCaffrey
Chief, Public Utilities Division
SOIC
100 West Randolph Street
Chicago, Illinois 60601

Assistant Attorney General
100 West Randolph Street
Suite 12
Chicago, Illinois 60601

Resident Inspector/LaSalle, NPS
U.S. Nuclear Regulatory Commission
Rural Route No. 1
P. O. Box 224
Marseilles, Illinois 61341

Chairman
LaSalle County Board of Supervisors
LaSalle County Courthouse
Ottawa, Illinois 61350

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Mr. Michael C. Parker, Chief
Division of Engineering
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated January 19, 1987 supplemented February 24, 1987 and May 24, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 63, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

8902100198 890207
PDR ADOCK 05000373
PDL

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Daniel R. Muller".

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 8-1a

3/4 8-4

INSERT

3/4 8-1a

3/4 8-4

ELECTRICAL POWER SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

*For the purpose of completing technical specification surveillance requirements 4.8.1.1.2d.1 and 4.8.1.1.2f.1, as part of a pre-planned preventive maintenance program, on the 0 diesel generator the requirements of action statement a are modified to:

1. Eliminate the requirement for performing technical specification surveillance requirements 4.8.1.1.1a and 4.8.1.1.2a.4, on each operable AC source, immediately and once per 8 hours thereafter, when the 0 diesel generator is declared inoperable.
2. Allow an additional 96 hours in excess of the 72 hours allowed in action statement a for the 0 diesel generator to be inoperable.

Provided that the following conditions are met:

- A. Unit 2 is in operational condition 4 or 5 or defueled prior to taking the 0 diesel generator out of service:
- B. Surveillance requirements 4.8.1.1.1a and 4.8.1.1.2a.4 are successfully completed, for the offsite power sources and the 1A and 2A diesel generators, within 48 hours prior to removal of the 0 diesel generator from service.
- C. No maintenance is performed on the offsite circuits or the 1A or 2A diesel generators, while the 0 diesel generator is inoperable.
- D. Technical specification requirement 4.8.1.1.1a is performed daily, while the 0 diesel generator is inoperable.
- E. The control circuit for the unit cross-tie circuit breakers between buses 142Y and 242Y are temporarily modified to allow the breakers to be closed with a diesel generator feeding the bus, while the 0 diesel generator is inoperable.

The provisions of technical specification 3.0.4 are not applicable.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 18 months during shutdown by:
1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.
 2. Verifying the diesel generator capability to reject a load of greater than or equal to 1190 kW for diesel generator 0, greater than or equal to 638 kW for diesel generators 1A and 2A, and greater than or equal to 2381 kW for diesel generator 1B while maintaining engine speed less than or equal to 75% of the difference between nominal speed and the overspeed trip setpoint or 15% above nominal, whichever is less.
 3. Verifying the diesel generator capability to reject a load of 2600 kW without tripping. The generator voltage shall not exceed 5000 volts during and following the load rejection.
 4. Simulating a loss of offsite power by itself, and:
 - a) For Divisions 1 and 2 and for Unit 2 Division 2:
 - 1) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
 - 2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 13 seconds, energizes the auto-connected loads and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady state voltage and frequency of the emergency busses shall be maintained at 4160 ± 150 volts and 60 ± 1.2 Hz during this test.
 - b) For Division 3:
 - 1) Verifying de-energization of the emergency bus.
 - 2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency bus with its loads within 13 seconds and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady state voltage and frequency of the emergency bus shall be maintained at 4160 ± 150 volts and 60 ± 1.2 Hz during this test.
 5. Verifying that on an ECCS actuation test signal, without loss of offsite power, diesel generators 0, 1A and 1B start on the auto-start signal and operate on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be $4160 + 416, -150$ volts and $60 + 3.0, -1.2$ Hz within 13 seconds after the auto-start signal; the steady state generator voltage and frequency shall be maintained within these limits during this test.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44
License No. NPF-18

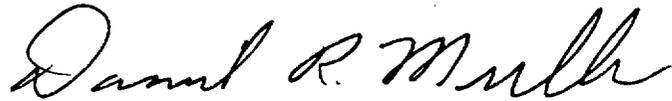
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated January 19, 1987 supplemented February 24, 1987 and May 24, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 44, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Daniel R. Muller".

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 8-1a

3/4 8-4

INSERT

3/4 8-1a

3/4 8-4

ELECTRICAL POWER SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

*For the purposes of completing technical specification surveillance requirements 4.8.1.1.2d.1 and 4.8.1.1.2f.1, as part of a pre-planned preventative maintenance program, on the 0 diesel generator the requirements of action statements a are modified to:

1. Eliminate the requirement for performing technical specification surveillance requirements 4.8.1.1.1a and 4.8.1.1.2a.4, on each operable AC source, immediately and once per 8 hours thereafter, when the 0 diesel generator is declared inoperable.
2. Allow an additional 96 hours in excess of the 72 hours allowed in action statement a for the 0 diesel generator to be inoperable.

Provided that the following conditions are met:

- A. Unit 1 is in operational condition 4 or 5 or defueled prior to taking the 0 diesel generator out of service.
- B. Surveillance requirements 4.8.1.1.1a and 4.8.1.1.2a.4 are successfully completed, for the offsite power sources and the 1A or 2A diesel generators, within 48 hours prior to removal of the 0 diesel generator from service.
- C. No maintenance is performed on the offsite circuits or the 1A or 2A diesel generators, while the 0 diesel generator is inoperable.
- D. Technical specification requirements 4.8.1.1.1a is performed daily, while the 0 diesel generator is inoperable.
- E. The control circuit for the unit cross-tie circuit breakers between buses 142Y and 242Y are temporarily modified to allow the breakers to be closed with a diesel generator feeding the bus, while the 0 diesel generator is inoperable.

The provisions of technical specification 3.0.4 are not applicable.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 18 months during shutdown by:
1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.
 2. Verifying the diesel generator capability to reject a load of greater than or equal to 1190 kW for diesel generator 0, greater than or equal to 638 kW for diesel generators 1A and 2A, and greater than or equal to 2381 kW for diesel generator 2B while maintaining engine speed less than or equal to 75% of the difference between nominal speed and the overspeed trip setpoint or 15% above nominal, whichever is less.
 3. Verifying the diesel generator capability to reject a load of 2600 kW without tripping. The generator voltage shall not exceed 5000 volts during and following the load rejection.
 4. Simulating a loss-of-offsite power by itself, and:
 - a) For Divisions 1 and 2 and for Unit 1 Division 2:
 - 1) Verifying deenergization of the emergency busses and load shedding from the emergency busses.
 - 2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 13 seconds, energizes the auto-connected loads and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady-state voltage and frequency of the emergency busses shall be maintained at 4160 ± 150 volts and 60 ± 1.2 Hz during this test.
 - b) For Division 3:
 - 1) Verifying deenergization of the emergency bus.
 - 2) Verifying the diesel generator starts on the auto-start signal, energizes the emergency bus with its loads within 13 seconds and operates for greater than or equal to 5 minutes while its generator is so loaded. After energization, the steady-state voltage and frequency of the emergency bus shall be maintained at 4160 ± 150 volts and 60 ± 1.2 Hz during this test.
 5. Verifying that on an ECCS actuation test signal, without loss-of-offsite power, diesel generators 0, 2A, and 2B start on the auto-start signal and operate on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be $4160 + 416$, -150 volts and $60 + 3.0$, -1.2 Hz within 13 seconds after the auto-start signal; the steady-state generator voltage and frequency shall be maintained within these limits during this test.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. NPF-18
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letters dated January 19, 1987, February 24, 1987 and May 24, 1988, Commonwealth Edison Company (CECo) requested a change to the LaSalle County Station Units 1 and 2 Technical Specifications so that preventive maintenance (required by Technical Specification 4.8.1.1.2.d.1 and 4.8.1.1.2.f.1) can be implemented on diesel generator 0 without the need for shutting down both units. Because diesel generator 0 is designed to supply both units' safety related equipment, the Technical Specifications require (for diesel generator 0 inoperability) shutdown of both units after 3 days and testing of the other diesel generators every 8 hours. CECo has requested that the required shutdown after 3 days be changed to shutdown after 7 days and that testing of other diesel generators every 8 hours be eliminated when inoperability of diesel generator 0 is due to required preventive maintenance. In addition, for implementation of this proposed Technical Specification, CECo committed to the following conditions.

1. One unit must be in cold shutdown, the refueling mode, or defueled before diesel generator 0 is declared inoperable for required preventive maintenance.
2. Within 24 hours prior to removal of diesel generator 0 from service, the diesel generators dedicated to the operating unit and diesel generator A of the unit in refueling must be started and loaded (Technical Specification 4.8.1.1.2.a.4 and 4.8.1.1.2.a.5).
3. No maintenance of the offsite power circuits or the A and B diesel generators of the operating unit or the A diesel generator of the unit in refueling may be performed while diesel generator 0 is out of service.
4. The control circuit for the unit cross-tie circuit breakers between buses 142Y and 242Y will be temporarily modified to allow the breakers to be closed with a diesel generator feeding one of the buses.
5. Within 24 hours prior to removal of diesel generator 0 from service and once per 24 hours thereafter, the alignment of the offsite power circuits for the operating unit must be verified (Technical Specification 4.8.1.1.1.a).

2.0 DISCUSSION AND EVALUATION

In order to allow continued operation of one unit while diesel generator 0 is out of service to satisfy maintenance requirements, CECO has requested an extension of the allowable out-of-service time from 3 to 7 days for diesel generator 0 once per 18 months. To demonstrate that no significant additional risk will be incurred by this increased out-of-service time, CECO provided the following results of a comparative Probabilistic Risk Assessment (PRA).

1. The Core Damage Frequency (CDF) for the 3-day allowable out-of-service time under current Technical Specifications was calculated to be $1.1E-08$. $1.1E-08$ is 0.02 percent of an annual CDF estimated for a similar plant.
2. The CDF for the 7-day allowable out-of-service time under the restrictions imposed for implementation of this Technical Specification (i.e., items 2, 3, and 5 listed above) was calculated to be $1.9E-08$. $1.9E-08$ is about 0.03 percent of an annual CDF estimated for a similar plant.
3. The extension of the allowable out-of-service time from 3 to 7 days results in an increased risk of approximately $8E-09$. When compared to an annual core damage frequency estimated for a similar plant, the risk increased approximately 0.01 percent. CECO considers this increase in risk to be insignificant.

Based on the results of comparative probabilistic risk assessment summarized above, the staff agrees that the proposed change does not significantly increase risk. The staff, therefore, concludes that the 7-day inoperability time for diesel generator 0 meets requirements of Section 4.11 of IEEE Standard 279-1971 and is acceptable.

In addition, CECO requested that testing of the diesel generators every 8 hours be eliminated when inoperability of diesel generator 0 is due to required preventive maintenance. In justification, CECO indicated that since diesel generator 0 is removed from service to perform preplanned preventive maintenance, there is no reason to expect that the other diesels may have experienced a common mode failure. Hence, the required surveillance of the other diesels has little effect on improving diesel generator availability. Also, analysis performed by Brookhaven National Laboratory for the Nuclear Regulatory Commission (NUREG/CR-4810 Evaluation of Diesel Unavailability and Risk Effective Surveillance Test Intervals) indicates that increased demands on the diesel generators tend to degrade their reliability. Based on justification and required testing that will be performed on the other diesels prior to removal of diesel generator 0 from service, the staff concludes that CECO's request for the elimination of testing every 8 hours reduces unnecessary diesel generator testing, improves diesel generator availability for accident mitigation, and is therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation and use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of an effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (52 FR 20797) on June 3, 1987, and consulted with the state of Illinois. No public comments were received, and the state of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCES

Letter from C. Allen, Commonwealth Edison to USNRC dated January 19, February 24, 1987, and May 24, 1988.

Principal Contributors: John Knox, NRR/SELB
Paul Shemanski, NRR/PDIII-2

Dated: February 7, 1989