

February 7, 1989

Docket Nos: 50-373 and 50-374

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

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Dear Mr. Bliss:

Subject: ISSUANCE OF AMENDMENT NOS. 64 AND 45 TO FACILITY OPERATING LICENSES NPF-11 AND NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC NOS. 68858 AND 69075)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 64 to Facility Operating License No. NPF-11 and Amendment No. 45 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letters dated May 23 and August 9, 1988.

The amendments revise the LaSalle County Station, Units 1 and 2 and Technical Specifications by modifying the table of primary containment isolation valves to identify new excess flow check valves being installed during the second refuel outage and removing the License Condition which has been satisfied by this modification.

A copy of the related Safety Evaluation supporting Amendment No. 64 to Facility Operating License No. NPF-11 and Amendment No. 45 to Facility Operating License No. NPF-18 is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Paul C. Shemanski, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 64 to NPF-11
2. Amendment No. 45 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

PDIII-2 P.S.
PShemanski: km
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R. Bachmann
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 7, 1989

Docket Nos: 50-373 and 50-374

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

Subject: ISSUANCE OF AMENDMENT NOS. 64 AND 45 TO FACILITY OPERATING LICENSES
NPF-11 AND NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC NOS.
68858 AND 69075)

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A copy of the related Safety Evaluation supporting Amendment No. 64 to Facility Operating License No. NPF-11 and Amendment No. 45 to Facility Operating License No. NPF-18 is enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Paul C. Shemanski".

Paul C. Shemanski, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 64 to NPF-11
2. Amendment No. 45 to NPF-18
3. Safety Evaluation

cc w/enclosure:
See next page

Mr. Henry E. Bliss
Commonwealth Edison Company

LaSalle County Nuclear Power Station
Units 1 & 2

cc:

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LaSalle County Courthouse
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Chairman
Illinois Commerce Commission
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Springfield, Illinois 62706

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Illinois Department of Nuclear Safety
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U. S. Nuclear Regulatory Commission
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Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated May 23, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by removing License Condition 2.C.(30)(i) which is obsolete and by changes to the Technical Specifications as indicated in the enclosure to this license amendment. Paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.64, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective 45 days from date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, reading "Daniel R. Muller". The signature is written in a cursive style with a large initial 'D' and 'M'.

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1989

ENCLOSURE TO LICENSE AMENDMENT NO. 64

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 6-31

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INSERT

3/4 6-31

3/4 6-31a

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

LA SALLE - UNIT 1

3/4 6-31

Amendment No. 64

VALVE FUNCTION AND NUMBER

Excess Flow Check Valves^(g) (Continued)

- 46. 1E12-F359A, B
- 47. 1E12-F319
- 48. 1E12-F317
- 49. 1E12-F360A, B
- 50. 1E21-F304
- 51. 1E22-F304
- 52. 1E22-F341
- 53. 1E22-F342
- 54. 1B33-F319A, B
- 55. 1B33-F317A, B
- 56. 1B33-F313A, B, C, D
- 57. 1B33-F311A, B, C, D
- 58. 1B33-F315A, B, C, D
- 59. 1B33-F301A, B
- 60. 1B33-F307A, B, C, D
- 61. 1B33-F305A, B, C, D
- 62. 1CM004
- 63. 1CM002
- 64. 1CM012
- 65. 1CM010
- 66. 1VQ061
- 67. 1B21-F457
- 68. 1B21-F459

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

Excess Flow Check Valves^(g) (Continued)

69. 1B21-F461

70. 1CM102

71. 1B21-F570

72. 1B21-F571

ENCLOSURE TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 6-34

INSERT

3/4 6-34



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated August 9, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by removing License Condition 2.C.(18)(c) which is obsolete and by changes to the Technical Specifications as indicated in the enclosure to this license amendment. Paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 45, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective 45 days from date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Daniel R. Muller". The signature is written in a cursive style with a large initial 'D'.

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: February 7, 1989

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER

Excess Flow Check Valves^(g) (Continued)

- 48. 2E12-F317
- 49. 2E12-F360A, B
- 50. 2E21-F304
- 51. 2E22-F304
- 52. 2E22-F341
- 53. 2E22-F342
- 54. 2B33-F319A, B
- 55. 2B33-F317A, B
- 56. 2B33-F313A, B, C, D
- 57. 2B33-F311A, B, C, D
- 58. 2B33-F315A, B, C, D
- 59. 2B33-F301A, B
- 60. 2B33-F307A, B, C, D
- 61. 2B33-F305A, B, C, D
- 62. 2CM004
- 63. 2CM002
- 64. 2CM012
- 65. 2CM010
- 66. 2VQ061
- 67. 2B21-F457
- 68. 2B21-F459
- 69. 2B21-F461
- 70. 2CM102
- 71. 2B21-F570
- 72. 2B21-F571



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 64 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. NPF-18
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

The proposed amendment to Operating License No. NPF-11 and Operating License No. NPF-18 would revise the LaSalle Units 1 and 2 Technical Specifications by modifying the table of primary containment isolation valves to identify new excess flow check valves being installed during the second refuel outage. These valves are being added as part of a modification to improve the reactor water level instrumentation system as required by NUREG-0737, Item II.F.2 and Generic Letter 84-23. In addition, Amendment No. 64 to Operating License No. NPF-11 removes NPF-11 License Condition 2.C.(18)(c) and Amendment No. 45 to Operating License No. NPF-18 removes License Condition 2.C.(30)(i) both of which are now obsolete.

2.0 Evaluation

Generic Letter 84-23 indicated a large error could occur in indicated reactor vessel water level under certain accident conditions. This potential error was required to be eliminated by improvements to existing instrumentation by NUREG-0737, II.F.2. This requirement was made part of the Unit 1 license in License Condition 2.C.(30)(i) and part of the Unit 2 license in License Condition 2.C.(18)(c).

Commonwealth Edison (CECo) provided a response to Generic Letter 84-23 in a letter dated June 10, 1986 in which certain design changes were proposed to correct this problem. The NRC staff accepted the CECo proposed solution in a letter dated March 2, 1987.

Installation of the proposed modification has resulted in two additional instrument lines which pass through the primary containment in existing containment penetrations. General Design Criteria 55 requires the use of isolation devices for these lines. Excess flow check valves and manual isolation valves were installed outside the containment to perform this function. The excess flow check valves acting as automatic primary containment isolation valves must be added to Technical Specification Table 3.6.3-1.

The proposed Technical Specification amendment adds the excess flow check valves installed as automatic isolation devices to Table 3.6.3-1. These

valves are designated as 1B21-F570 and 1B21-F571 for Unit 1 and 2B21-F570 and 2B21-F571 for Unit 2. These changes to the Technical Specifications are administrative in nature resulting from the excess flow check valve modification which was approved by NRC in a letter to CECo dated March 2, 1987. The staff finds the proposed amendment to be acceptable.

3.0 Environmental Consideration

The amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 Conclusion

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the FEDERAL REGISTER (53 FR 32291) on August 24, 1988 and (53 FR 34601) on September 7, 1988 and consulted with the state of Illinois. No public comments were received, and the state of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCES

Letters from C. Allen, Commonwealth Edison to USNRC dated May 23 and August 9, 1988.

Principal Contributor: Paul Shemanski, NRR/PDIII-2

Dated: February 7, 1989