

October 25, 1991

Docket Nos. 50-373
and 50-374

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 80273 AND 80274)

DISTRIBUTION:

✓ Docket Files, P1-37	BBoger
PDIII-2 r/f	JZwolinski
RBarrett	BSiegel
Moore	OGC, 15B18
DHagan, MNBB 3206	EJordan, MNBB 3701
GHill(8), P1-37	WJones
CGrimes, 11E22	ACRS(10), P-315
GPA/PA, 2G5	OC/LFMB
PDIII-2 Gray	W. Kuo, 11F23
NRC & Local PDRs	C. Cheng, 7D4
RElliott	

The Commission has issued the enclosed Amendment No. 80 to Facility Operating License No. NPF-11 and Amendment No. 64 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2, respectively. The amendments are in response to your application dated April 1, 1991, as supplemented by your letters dated September 13 and October 17, 1991.

The amendments will incorporate changes into the Technical Specifications to meet the requirements of Generic Letter 88-01 (GL 88-01) regarding Intergranular Stress Corrosion Cracking.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

Byron L. Siegel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 80 to NPF-11
2. Amendment No. 64 to NPF-18
3. Safety Evaluation

cc w/enclosures:
See next page

NRC FILE CENTER COPY

* Please see previous concurrence

OFC	:LA:PDIII-2	:PE:PDIII-2	:PM:PDIII-2	:EMCB	:D:PDIII-2	:OGC
NAME	:CMOORE	:RELIOTT*	:BSIEGEL:JAP*	:CCHENG*	:RBARRETT	:EHOLLER*
DATE	:10/25/91	:10/04/91	:10/04/91	:10/04/91	:10/25/91	:10/11/91

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Document Name: AMENDMENT 80273/74

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Mr. Thomas J. Kovach
Commonwealth Edison Company

LaSalle County Station
Units Nos. 1 and 2

cc:
Phillip P. Steptoe, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60603

Robert Cushing
Chief, Public Utilities Division
Illinois Attorney General's Office
100 West Randolph Street
Chicago, Illinois 60601

Assistant Attorney General
100 West Randolph Street
Suite 12
Chicago, Illinois 60601

Michael I. Miller, Esq.
Sidley and Austin
One First National Plaza
Chicago, Illinois 60690

Resident Inspector/LaSalle, NPS
U.S. Nuclear Regulatory Commission
Rural Route No. 1
P. O. Box 224
Marseilles, Illinois 61341

Chairman
LaSalle County Board of Supervisors
LaSalle County Courthouse
Ottawa, Illinois 61350

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Illinois Department of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137

Robert Neumann
Office of Public Counsel
State of Illinois Center
100 W. Randolph
Suite 11-300
Chicago, Illinois 60601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee) dated April 1, 1991, as supplemented on September 13 and October 17, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

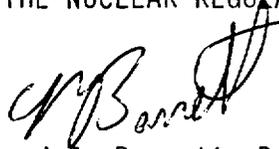
9111130260 911025
PDR ADOCK 05000373
P PDR

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 80 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective immediately to be implemented within 30 days after date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 25, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 0-3

3/3 4-7

INSERT

3/4 0-3

3/4 4-7

3/4 4-8

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- f. The inservice inspection program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion included in Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff.

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

- 3.4.3.2 Reactor coolant system leakage shall be limited to:
- No PRESSURE BOUNDARY LEAKAGE.
 - 5 gpm UNIDENTIFIED LEAKAGE.
 - 25 gpm total leakage averaged over any 24 hour period.
 - 1 gpm leakage at a reactor coolant system pressure at 1000 ± 50 psig from any reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1.
 - 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24 hour period.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

- With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- With any reactor coolant system leakage greater than the limits in b and/or c, above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- With any reactor coolant system pressure isolation valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed valves, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- With one or more high/low pressure interface valve leakage pressure monitors inoperable, restore the inoperable monitor(s) to OPERABLE status within 7 days or verify the pressure to be less than the alarm setpoint at least once per 12 hours by local indication; restore the inoperable monitor(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 12 hours.
- With any reactor coolant system leakage greater than the limit in e, above, identify the source of leakage within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.2.1 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- Monitoring the primary containment atmospheric particulate and gaseous radioactivity at least once per 12 hours,

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. Monitoring the primary containment sump flow rate on average once per 8 hours not to exceed 12 hours,* and
- c. Monitoring the primary containment air coolers condensate flow rate at least once per 12 hours.

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE:

- a. Pursuant to Specification 4.0.5, except that in lieu of any leakage testing required by Specification 4.0.5, each valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:
 - 1. At least once per 18 months, and
 - 2. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

- b. By demonstrating OPERABILITY of the high/low pressure interface valve leakage pressure monitors by performance of a:
 - 1. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
 - 2. CHANNEL CALIBRATION at least once per 18 months,

With the alarm setpoint for the:

- 1. HPCS system \leq 100 psig.
- 2. LPCS system \leq 500 psig.
- 3. LPCI/shut-down cooling system \leq 400 psig.
- 4. RHR shutdown cooling \leq 190 psig.
- 5. RCIC \leq 90 psig.

*Technical Specification 4.0.2 does not apply.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee) dated April 1, 1991, as supplemented on September 13 and October 17, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 64 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective immediately to be implemented within 30 days after date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 25, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 64

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change. Pages identified by an asterisk are provided for convenience.

<u>REMOVE</u>	<u>INSERT</u>
3/4 0-3	3/4 0-3
*3/4 4-7	*3/4 4-7
3/4 4-8	3/4 4-8
3/4 4-9	3/4 4-9
*3/4 4-10	*3/4 4-10

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- f. The inservice inspection program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion included in Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff.

REACTOR COOLANT SYSTEM

3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant system leakage detection systems shall be OPERABLE:

- a. The primary containment atmosphere particulate radioactivity monitoring system,
- b. The primary containment sump flow monitoring system, and
- c. Either the primary containment air coolers condensate flow rate monitoring system or the primary containment atmosphere gaseous radioactivity monitoring system.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required gaseous and/or particulate radioactive monitoring system is inoperable; otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The reactor coolant system detection systems shall be demonstrated OPERABLE by:

- a. Primary containment atmosphere particulate and gaseous monitoring systems-performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.
- b. Primary containment sump flow monitoring system-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION TEST at least once per 18 months.
- c. Primary containment air coolers condensate flow rate monitoring system-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.3.2 Reactor coolant system leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE.
- b. 5 gpm UNIDENTIFIED LEAKAGE.
- c. 25 gpm total leakage averaged over any 24 hour period.
- d. 1 gpm leakage at a reactor coolant system pressure at 1000 ± 50 psig from any reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1.
- e. 2 gpm increase in UNIDENTIFIED LEAKAGE within any 24 hour period.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With any reactor coolant system leakage greater than the limits in b and/or c, above, reduce the leakage rate to within the limits within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. With any reactor coolant system pressure isolation valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed valves, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- d. With one or more high/low pressure interface valve leakage pressure monitors inoperable, restore the inoperable monitor(s) to OPERABLE status within 7 days or verify the pressure to be less than the alarm setpoint at least once per 12 hours by local indication; restore the inoperable monitor(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 12 hours.
- e. With any reactor coolant system leakage greater than the limit in e, above, identify the source of leakage within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.2.1 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:

- a. Monitoring the primary containment atmospheric particulate and gaseous radioactivity at least once per 12 hours,

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. Monitoring the primary containment sump flow rate on average once per 8 hours not to exceed 12 hours,* and
- c. Monitoring the primary containment air coolers condensate flow rate at least once per 12 hours.

4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE:

- a. Pursuant to Specification 4.0.5, except that in lieu of any leakage testing required by Specification 4.0.5, each valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:
 - 1. At least once per 18 months, and
 - 2. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

- b. By demonstrating OPERABILITY of the high/low pressure interface valve leakage pressure monitors by performance of a:
 - 1. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
 - 2. CHANNEL CALIBRATION at least once per 18 months,

With the alarm setpoint for the:

- 1. HPCS system \leq 100 psig.
- 2. LPCS system \leq 500 psig.
- 3. LPCI/shutdown cooling system \leq 400 psig.
- 4. RHR shutdown cooling \leq 190 psig.
- 5. RCIC \leq 90 psig.

*Technical Specification 4.0.2 does not apply.

TABLE 3.4.3.2-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

<u>SYSTEM</u>	<u>VALVE NUMBER</u>	<u>FUNCTION</u>
a. LPCS	E21-F006 E21-F005	LPCS Injection LPCS Injection
b. HPCS	E22-F005 E22-F004	HPCS Injection HPCS Injection
c. RHR	E12-F041A E12-F041B E12-F041C E12-F042A E12-F042B E12-F042C E12-F050A E12-F050B E12-F053A E12-F053B E12-F009 E12-F008	LPCI Injection LPCI Injection LPCI Injection LPCI Injection LPCI Injection LPCI Injection Shutdown Cooling Return Shutdown Cooling Return Shutdown Cooling Return Shutdown Cooling Return Shutdown Cooling Suction Shutdown Cooling Suction
d. RCIC	E51-F066 E51-F065	RCIC Head Spray RCIC Head Spray



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 64 TO FACILITY OPERATING LICENSE NO. NPF-18

COMMONWEALTH EDISON COMPANY

LASALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

Commonwealth Edison Company (CECo, the licensee) submitted its response to the NRC Generic Letter (GL) 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping" for LaSalle County Station, Units 1 and 2, by letter dated July 29, 1988. Additional information was provided in response to an NRC Request for Additional Information (RAI) by letter dated June 30, 1989. The staff, with assistance from its contractor Viking Systems International (VSI), reviewed the licensee's responses and issued its Safety Evaluation (SE) with an attached Technical Evaluation Report (TER) on August 22, 1990. The SE found the licensee's response acceptable with five (5) exceptions. CECo was requested to:

- (1) Include a statement in the surveillance or administrative controls section of the Technical Specifications that includes the following:
"The inservice inspection program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion included in Generic Letter 88-01 or in accordance with alternate measures approved by the NRC staff."
- (2) Include an additional Limiting Condition for Operation (LCO) in the Technical Specifications that specifies reactor coolant system leakage shall be limited to a 2 gpm increase in unidentified leakage within any 24-hour period.
- (3) Include a surveillance requirement in the Technical Specifications that primary containment sump flow rate will be monitored at least once per 8 hours. It should be noted that the position in GL 88-01 on leak rate monitoring was modified to permit leakage measurements based on sump flow instruments to be taken every 8 hours instead of every 4 hours.
- (4) For LaSalle County Station Unit 1, include an LCO in the Technical Specifications on operability of sump monitoring instruments. Since your plant has IGSCC weld category E weldments, Generic Letter 88-01 provides an allowed outage time of 24 hours for repairing the drywell floor drain sump monitoring system, or an orderly shutdown should be

initiated. As an alternative, the staff recommends that when the drywell sump monitoring system is inoperable, the operator should use a demonstrated manual method for determining leak rate, such as measuring the time to manually pump the sump at a fixed interval (4 hours). The staff considers manual measurement a viable sump monitoring method without hardship to the operator; therefore, this method could be added to the appropriate LCO section. With the manual method operable, the outage time for the drywell sump monitoring system could be extended to 30 days. However, if the sump pump and drywell sump monitoring systems are inoperable concurrently, then either system has to be repaired within 24 hours or an orderly shutdown should be initiated.

- (5) Address the Reactor Water Cleanup system (RWCU) piping outboard of the isolation valves in the ISI program. If the piping is within the scope of GL 88-01, the licensee will need to modify the program to include the identity of the welds as well as plans for mitigation and inspections. A minimum of 10% of the RWCU system piping outboard of the isolation valves should be inspected at each refueling outage. If cracks are found, the licensee should discuss sample expansion and mitigation methods with the NRC staff.

CECo submitted a response to the above deficiencies by letter dated November 5, 1990. The Technical Specification (TS) change request was submitted by letter dated April 1, 1991 and supplemental information, which was not outside the scope of the original submittal, was provided in letters dated September 13 and October 17, 1991.

2.0 DISCUSSION

CECo's response to each of the deficiencies noted in the SE proposed TS changes and the staff's evaluation are presented below.

- Item (1): CECO proposed a statement be included in Technical Specification 4.0.5 stating that the inservice inspection program for piping identified in NRC Generic Letter 88-01 shall be performed in accordance with the NRC staff positions on schedule, methods, personnel, and sample expansion included in generic letter 88-01 or in accordance with alternate measures approved by the NRC staff.

Since CECO proposed a statement under the inservice inspection surveillance requirements of the TS (Section 4.0.5) in its letter dated April 1, 1991, that is consistent with the guidance contained in the staff's Safety Evaluation issued on August 22, 1990, the staff finds this proposed change acceptable.

- Item (2): CECO proposed an additional limiting condition for operation (LCO) be added to Technical Specification 3.4.3.2 that specifies reactor coolant leakage shall be limited to a 2 gpm increase in unidentified leakage within any 24 hour period.

Since CECO proposed to add an LCO to TS Section 3.4.3.2 that is consistent with the guidance contained in the staff's Safety Evaluation issued on August 22, 1990, the staff finds this proposed change acceptable.

Item (3): This item recommends addition of a requirement to the Technical Specifications to monitor the primary containment sump flow rate at least once per 8 hours. CECO stated that LaSalle County Station "shiftly" surveillance interval as defined in Table 1.1 of the Technical Specifications is "at least once per 12 hours." At the present time there are no Technical Specification surveillance requirements at LaSalle which require an 8-hour time interval. As a result the station does not have any administrative controls in place to ensure that such a surveillance requirement is met. Technical Specification 4.4.3.2 currently contains a surveillance requirement to monitor the primary containment sump flow rate at least once per 12 hours. Normally the Technical Specification "shiftly" surveillance requirements are performed three times daily (once during each shift). Therefore, the surveillance requirement as currently written was considered by the station to be adequate to meet the intent of the recommendation in Generic Letter 88-01.

The staff reevaluated the frequency of leakage monitoring after discussions with the BWROG and concluded that measurements taken every 8 hours (compared to every 4 hours as originally specified in GL 88-01) are adequate and necessary. On this basis CECO was informed that its response to monitor the containment sump flow rate at least once per 12 hours was unacceptable.

In letters dated September 13 and October 17, 1991, CECO revised its April 1, 1991 TS submittal to state that monitoring of the primary containment sump flow rate shall be performed on average once per 8 hours not to exceed 12 hours. Since this revision is consistent with the staff's current position, it is acceptable. The proposed TS change also permits up to a maximum 4 additional hours to perform this surveillance. This provides LaSalle some flexibility in performing this surveillance which the staff also finds acceptable.

CECO has proposed to add an action statement under TS Section 3.4.3.2 which would allow 4 hours to identify the leakage source if the 2 gpm increase limit is exceeded in any 24-hour period or be in hot shutdown within the next 12 hours and in cold shutdown within the following 24 hours. Since this action statement is consistent with the other action statements in TS Section 3.4.3.2 related to reactor pressure coolant system leakage, the staff finds this acceptable.

Item (4): On June 30, 1989 CECO provided additional information concerning NRC Generic Letter 88-01 as requested in an NRC letter dated May 1, 1989. At that time it was identified that LaSalle Unit 1 had two Category E weldments.

Further inspections were performed during the third refueling outage for Unit 1 in Spring 1990 of Weld Numbers RR-1005-27A and RR-1001-10 using the General Electric automated "SMART" Ultrasonic Testing (UT) System. The indications which had previously been classified as Category E IGSCC were redefined to be root and internal diameter geometry. This redefinition is attributed to the additional machining of the weld crown on these two welds which resulted in a smoother scanning surface for the UT system. The "SMART" UT system was able to characterize the indications more accurately on the smoother surface. As a result Weld Numbers RR-1005-27A and RR-1001-10 have been reclassified from Category E to Category B.

The staff concurs with CECO that LaSalle Station, Unit 1 no longer has any weldments classified as Category E and, therefore, the additional leak detection monitoring requirements provided in Generic Letter 88-01 for plants with Category E weldments no longer apply. Also, as a result of the reclassification of these welds an LCO related to the operability of the sump monitoring instruments is not required.

Item (5): An evaluation by the LaSalle Station Technical Staff Inservice Inspection (ISI)/Inservice Testing (IST) Group determined that neither the Unit 1 or Unit 2 Reactor Water Cleanup System (RWCU) contains any Austenitic Stainless Steel. Therefore, the RWCU system does not fall within the scope of NRC Generic Letter 88-01 and no further action is required.

The staff concurs with CECO that since the RWCU systems at LaSalle Station do not contain any austenitic stainless steel, no further action is required.

In summary based on the review of CECO's responses to the staff's SE dated August 22, 1990, the staff concludes these responses are in accordance with the requirements of Generic Letter 88-01 and are, therefore, acceptable. The staff also concludes that the proposed TS changes contained in your April 1, September 13 and October 17, 1991, submittals are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (56 FR 27038). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Elliott

Dated: October 25, 1991