October 7, 1992

Docket Nos. 50-373
and 50-374

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

Dear Mr. Kovach:

DISTRIBUTION:
J. Roe
PDIII-2 r/f
J. Zwolinski
C. Moore
R. Elliott
D. Hagan
W. Jones
V. Ordaz
TDunning
PDIII-2 p/f
WMeinke
Docket File
NRC & Local PDRs
OPA
R. Barrett
B. Siegel
OGC
G. Hill (8)
C. Grimes
ACRS (10)
OC/LFMB
B. Clayton RIII

SUBJECT: CORRECTION TO AMENDMENTS (TAC NOS. M80638 AND M80639)

By letter dated September 1, 1992, the NRC transmitted to Commonwealth Edison Company Amendment No. 85 to Facility Operating License No. NPF-11 and Amendment No. 69 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2 respectively, related to the staff approval of the licensee's request to relocate the Radiological Effluent Technical Specifications (RETS) to the Offsite Dose Calculation Manual (ODCM) or the Process Control Program (PCP). It has come to our attention that the Safety Evaluation (SE) and the Technical Specification (TS) pages contained several errors. The SE and TS pages enclosed contain marginal lines indicating the area of change.

This correction is for clarification purposes only and does not revise the technical content of the affected TS pages the SE, or the conclusions stated in the SE.

Sincerely,

original signed by Chandu P. Patel for

Byron L. Siegel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation

cc w/enclosure:
See next page

OFC	LA:PDIII-2	PE:PDIII-2	PM:PDIII-2	D:PDIII-2	
NAME	CMOORE	RELLIOTT:jar	BSIEGEL	RBARRETT	
DATE	10/7/92	10/6/92	10/7/92	10/7/92	

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PDR ADOCK 05000373
P PDR

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cc:

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LINEAR HEAT GENERATION RATE

1.21 LINEAR HEAT GENERATION RATE (LHGR) shall be the heat generation per unit length of fuel rod. It is the integral of the heat flux over the heat transfer area associated with the unit length.

LOGIC SYSTEM FUNCTIONAL TEST

1.22 A LOGIC SYSTEM FUNCTIONAL TEST shall be a test of all logic components, i.e., all relays and contacts, all trip units, solid state logic elements, etc. of a logic circuit, from sensor through and including the actuated device to verify OPERABILITY. THE LOGIC SYSTEM FUNCTIONAL TEST may be performed by any series of sequential, overlapping or total system steps such that the entire logic system is tested.

MAXIMUM FRACTION OF LIMITING POWER DENSITY

1.23 The MAXIMUM FRACTION OF LIMITING POWER DENSITY (MFLPD) shall be the highest value of the FLPD which exists in the core.

MEMBER(S) OF THE PUBLIC

1.24 MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the licensee, its contractors, or vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational, or other purposes not associated with the plant.

MINIMUM CRITICAL POWER RATIO

1.25 The MINIMUM CRITICAL POWER RATIO (MCPR) shall be the smallest CPR which exists in the core.

OFFSITE DOSE CALCULATION MANUAL

1.26 The OFFSITE DOSE CALCULATION MANUAL (ODCM) shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring Alarm/Trip Setpoints, and in the conduct of the Environmental Radiological Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Technical Specification Section 6.2.F.4 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Operating and Semi-Annual Radioactive Effluent Release Reports required by Technical Specification Sections 6.6.A.3 and 6.6.A.4.

INSTRUMENTATION

EXPLOSIVE GAS MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.7.11 The explosive gas monitoring instrumentation channels shown in Table 3.3.7.11-1 shall be OPERABLE with their Alarm/Trip setpoints set to ensure that the limits of specification 3.11.2.1 are not exceeded.

APPLICABILITY: During operation of the main condenser air ejector.

ACTION:

a. With an explosive gas monitoring instrumentation channel Alarm/Trip setpoint less conservative than required by the above specification, declare the channel inoperable, and take the ACTION shown in Table 3.3.7.11-1.

b. With less than the minimum number of explosive gas monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.7.11-1. Restore the inoperable instrumentation channels to an OPERABLE status wihtin 30 days, or prepare and submit a Special Report to the Commission pursuant to Specification 6.6.C within the next 10 days outlining the cause of the malfunction and the plans for restoring the channel(s) to OPERABLE status.

c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.11 Each explosive gas monitoring instrumentation channel shall be demonstrated OPERABLE by performance of a CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION at the frequencies shown in Table 4.3.7.11-1.

RADIOACTIVE EFFLUENTS

MAIN CONDENSER

LIMITING CONDITION FOR OPERATION

3.11.2.2 The release rate of the sum of the activities from the noble gases measured prior to the holdup line shall be limited to less than or equal to 3.4×10^5 microcuries/second.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

With the release rate of the sum of the activities of the noble gases prior to the holdup line exceeding 3.4×10^5 microcuries/second restore the release rate to within its limit within 72 hours or be in at least STARTUP with the main steam isolation valves closed within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.11.2.2.1 The radioactivity rate of noble gases prior to the holdup line shall be continuously monitored in accordance with the ODCM.

4.11.2.2.2 The release rate of the sum of the activities from noble gases prior to the holdup line shall be determined to be within the limits of Specification 3.11.2.2 at the following frequencies by performing an isotopic analysis of a representative sample of gases taken prior to the holdup line.

a. At least once per 31 days.

b. Within 4 hours following an increase, as indicated by the off gas pre-treatment Noble Gas Activity Monitor, of greater than 50%, after factoring out increases due to changes in THERMAL POWER level, in the nominal steady state fission gas release from the primary coolant.

INSTRUMENTATION

EXPLOSIVE GAS MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.7.11 The explosive gas monitoring instrumentation channels shown in Table 3.3.7.11-1 shall be OPERABLE with their Alarm/Trip setpoints set to ensure that the limits of specification 3.11.2.1 are not exceeded.

APPLICABILITY: During operation of the main condenser air ejector.

ACTION:

a. With an explosive gas monitoring instrumentation channel Alarm/Trip setpoint less conservative than required by the above specification, declare the channel inoperable, and take the ACTION shown in Table 3.3.7.11-1.

b. With less than the minimum number of explosive gas monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3.7.11-1. Restore the inoperable instrumentation channels to an OPERABLE status within 30 days, or prepare and submit a Special Report to the Commission pursuant to Specification 6.6.C. within the next 10 days outlining the cause of the malfunction and the plans for restoring the channel(s) to OPERABLE status.

c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.11 Each explosive gas monitoring instrumentation channel shall be demonstrated OPERABLE by performance of a CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION at the frequencies shown in Table 4.3.7.11-1.

RADIOACTIVE EFFLUENTS

MAIN CONDENSER

LIMITING CONDITION FOR OPERATION

3.11.2.2 The release rate of the sum of the activities from the noble gases measured prior to the holdup line shall be limited to less than or equal to 3.4×10^5 microcuries/second.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3.

ACTION:

With the release rate of the sum of the activities of the noble gases prior to the holdup line exceeding 3.4×10^5 microcuries/second restore the release rate to within its limit within 72 hours or be in at least STARTUP with the main steam isolation valves closed within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.11.2.2.1 The radioactivity rate of noble gases prior to the holdup line shall be continuously monitored in accordance with the ODCM.

4.11.2.2.2 The release rate of the sum of the activities from noble gases prior to the holdup line shall be determined to be within the limits of Specification 3.11.2.2 at the following frequencies by performing an isotopic analysis of a representative sample of gases taken prior to the holdup line.

a. At least once per 31 days.

b. Within 4 hours following an increase, as indicated by the off gas pre-treatment Noble Gas Activity Monitor, of greater than 50%, after factoring out increases due to changes in THERMAL POWER level, in the nominal steady state fission gas release from the primary coolant.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-11 AND

AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. NPF-18

COMMONWEALTH EDISON COMPANY

LASALLE COUNTY STATION, UNITS 1 AND 2

DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By application dated May 22, 1991, Commonwealth Edison Company (CECo, the licensee) requested changes to the Technical Specifications (TS) for the LaSalle County Station, Units 1 and 2. The proposed changes would incorporate programmatic controls for radiological effluents and radiological environmental monitoring in the Administrative Controls section of the TS consistent with the requirements of 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50. At the same time, the licensee proposed to transfer the procedural details of the Radiological Effluent Technical Specifications (RETS) from the TS to the Offsite Dose Calculation Manual (ODCM) or to the Process Control Program (PCP) for solid radioactive wastes as appropriate. With these changes, the specifications related to RETS reporting requirements were simplified. Finally, changes to the definitions of the ODCM and PCP were proposed consistent with these changes. Guidance on these proposed changes was provided to all power reactor licensees and applicants by Generic Letter (GL) 89-01 dated January 31, 1989.

2.0 EVALUATION

The licensee's proposed changes to the TS are in accordance with the guidance provided in GL 89-01 and are addressed below.

(1) The licensee has proposed to incorporate programmatic controls for radioactive effluents and radiological environmental monitoring in Specification 6.2.F, "Plant Operating Procedures and Programs," of the TS as noted in the guidance provided in GL 89-01. The programmatic controls ensure that programs are established, implemented, and maintained to ensure that operating procedures are provided to control radioactive effluents consistent with the requirements of 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50.

(2) The licensee has confirmed that the detailed procedural requirements addressing Limiting Conditions for Operation, their applicability, remedial actions, associated surveillance requirements, or reporting requirements for the following specifications have been prepared to

implement the relocation of these procedural details to the ODCM or PCP. These changes to the ODCM and PCP have been prepared in accordance with the new Administrative Controls in the TS on changes to the ODCM and PCP so that they will be implemented in the ODCM or PCP when this amendment is issued.

SPECIFICATION	TITLE
3/4.3.7.10	Radioactive Liquid Effluent Monitoring Instrumentation
3/4.3.7.11	Radioactive Gaseous Effluent Monitoring Instrumentation
3/4.11.1.1	Radioactive Effluents: Liquid Effluents Concentration
3/4.11.1.2	Radioactive Effluents: Dose
3/4.11.1.3	Radioactive Effluents: Liquid Waste Treatment System
3/4.11.2.1	Radioactive Effluents: Gaseous Effluents Dose Rate
3/4.11.2.2	Radioactive Effluents: Dose - Noble Gases
3/4.11.2.3	Radioactive Effluents: Dose - Radioiodines, Radioactive Material in Particulate Form, and Radionuclides Other Than Noble Gases
3/4.11.2.4	Radioactive Effluents: Gaseous Radwaste Treatment System
3/4.11.2.5	Radioactive Effluents: Ventilation Exhaust Treatment System
3/4.11.2.8	Radioactive Effluents: Venting or Purging
3/4.11.3	Radioactive Effluents: Solid Radioactive Waste
3/4.11.4	Radioactive Effluents: Total Dose
3/4.12.1	Radiological Environmental Monitoring: Monitoring Program
3/4.12.2	Radiological Environmental Monitoring: Land Use Census
3/4.12.3	Radiological Environmental Monitoring: Interlaboratory Comparison Program

6.6.A.3 Annual Radiological Environmental Operating Report

6.6.A.4 Semiannual Radioactive Effluent Release Report

These procedural details that have been removed from the TS are not required by the Commission's regulations to be included in the TS. They have been prepared for incorporation in the ODCM or PCP upon issuance of this license amendment and may be subsequently changed by the licensee without prior NRC approval. Changes to the ODCM and PCP are documented and will be retained for the duration of the operating license in accordance with Specification 6.5.B.18.

(3) The licensee has proposed replacing the existing specifications in the Administrative Controls section of the TS for the Annual Radiological Environmental Operating Report, Specification 6.6.A.3, for the Semiannual Radioactive Effluent Release Report, Specification 6.6.A.4, for the Process Control Program, Specification 6.7, and for the Offsite Dose Calculation Manual, Specification 6.8, with the updated specifications that were provided in GL 89-01.

The following specifications that are included under the heading of Radioactive Effluents have been retained in the TS. This is in accordance with the guidance of GL 89-01.

SPECIFICATION	TITLE
3/4.3.7.11	Explosive Gas Monitoring Instrumentation
3/4.11.1.1	Radioactive Effluents: Liquid Holdup Tanks
3/4.11.2.1	Radioactive Effluents: Explosive Gas Mixture
3/4.11.2.2	Radioactive Effluents: Main Condenser
6.9	Major Changes to Radioactive Waste Treatment Systems

On the basis of the above, the staff finds that the changes included in the proposed TS amendment requests are consistent with the guidance provided in GL 89-01. Because the control of radioactive effluents continues to be limited in accordance with operating procedures that must satisfy the regulatory requirements of 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50, the NRC staff concludes that there is no impact on plant safety as a consequence. Accordingly, the staff finds the proposed changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.