



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAR 13 1986

Docket No. 50-373

Mr. Dennis L. Farrar
Director of Licensing
Commonwealth Edison Company
P.O. Box 767
Chicago, Illinois 60690

Dear Mr. Farrar:

Subject: Issuance of Amendment No. 36 to Facility Operating License
No. NPF-11 - La Salle County Station, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 36 to Facility Operating License No. NPF-11 for the La Salle County Station, Unit 1. This amendment is in response to your letter dated January 9, 1986.

The amendment revises the La Salle Unit 1 Technical Specifications to change the Main Steam Line Low Pressure instrument response time from 1 to 2 seconds on Table 3.3.2-3.

A copy of the related safety evaluation supporting Amendment No. 36 to Facility Operating License No. NPF-11 is enclosed.

Sincerely,

A handwritten signature in cursive script that reads "Elinor G. Adensam".

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosures:

1. Amendment No. 36 to NPF-11
2. Safety Evaluation

cc w/enclosure:
See next page

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Mr. Dennis L. Farrar
Commonwealth Edison Company

La Salle County Nuclear Power Station
Units 1 & 2

cc:

Philip P. Steptoe, Esquire
Suite 4200
One First National Plaza
Chicago, Illinois 60603

John W. McCaffrey
Chief, Public Utilities Division
160 North La Salle Street, Room 900
Chicago, Illinois 60601

Assistant Attorney General
188 West Randolph Street
Suite 2315
Chicago, Illinois 60601

Resident Inspector/LaSalle, NPS
U.S. Nuclear Regulatory Commission
Rural Route No. 1
P.O. Box 224
Marseilles, Illinois 61341

Chairman
La Salle County Board of Supervisors
La Salle County Courthouse
Ottawa, Illinois 61350

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager
Nuclear Facility Safety
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Rossevelt Road
Glen Ellyn, Illinois 60137

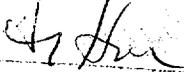
AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. NPF-11 - LA SALLE, UNIT 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated January 9, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 36, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: ~~MAR 13 1974~~

ENCLOSURE TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 3-18

INSERT

3/4 3-18

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>A. AUTOMATIC INITIATION</u>	
<u>1. PRIMARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level	
1) Low, Level 3	NA
2) Low Low, Level 2	$\leq 2.0^*/\leq 13^{(a)**}$
b. Drywell Pressure - High	$\leq 13^{(a)}$
c. Main Steam Line	
1) Radiation - High ^(b)	$\leq 1.0^*/\leq 13^{(a)**}$
2) Pressure - Low	$\leq 1.0^*/\leq 13^{(a)**}$
3) Flow - High	$\leq 0.5^*/\leq 13^{(a)**}$
d. Main Steam Line Tunnel Temperature - High	NA
e. Condenser Vacuum - Low	NA
f. Main Steam Line Tunnel Δ Temperature - High	NA
<u>2. SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Building Vent Exhaust Plenum Radiation - High ^(b)	$\leq 13^{(a)}$
b. Drywell Pressure - High	$\leq 13^{(a)}$
c. Reactor Vessel Water Level - Low, Level 2	$\leq 13^{(a)}$
d. Fuel Pool Vent Exhaust Radiation - High ^(b)	$\leq 13^{(a)}$
<u>3. REACTOR WATER CLEANUP SYSTEM ISOLATION</u>	
a. Δ Flow - High	$\leq 13^{(a)**}$
b. Heat Exchanger Area Temperature - High	NA
c. Heat Exchanger Area Ventilation Δ T-High	NA
d. SLCS Initiation	NA
e. Reactor Vessel Water Level - Low Low, Level 2	$\leq 13^{(a)}$
<u>4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>	
a. RCIC Steam Line Flow - High	$\leq 13^{(a)###}$
b. RCIC Steam Supply Pressure - Low	$\leq 13^{(a)}$
c. RCIC Turbine Exhaust Diaphragm Pressure - High	NA
d. RCIC Equipment Room Temperature - High	NA
e. RCIC Steam Line Tunnel Temperature - High	NA
f. RCIC Steam Line Tunnel Δ Temperature - High	NA
g. Drywell Pressure - High	NA
h. RCIC Equipment Room Δ Temperature - High	NA
<u>5. RHR SYSTEM STEAM CONDENSING MODE ISOLATION</u>	
a. RHR Equipment Area Δ Temperature - High	NA
b. RHR Area Cooler Temperature - High	NA
c. RHR Heat Exchanger Steam Supply Flow High	NA



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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. NPF-11

COMMONWEALTH EDISON COMPANY

LA SALLE COUNTY STATION, UNIT 1

DOCKET NO. 50-373

1.0 INTRODUCTION

By letter dated January 9, 1986, Commonwealth Edison Company (licensee) proposed an amendment that would change the La Salle County Station, Unit 1 Technical Specifications to revise the response time for the Main Steam Line Low Pressure trip function in the Technical Specifications Table 3.3.2-3 from 1 to 2 seconds.

2.0 EVALUATION

La Salle Unit 1 is currently shutdown performing refueling and is replacing environmentally non-qualified equipment with qualified equipment as required by License Condition 2.C.(11) during this first refueling outage. The Main Steam Line Low Pressure switches are scheduled to be changed from the non-qualified Barksdale model currently installed to environmentally qualified Static-O-Ring (SOR) pressure switches. These switches protect the reactor core by closing the Main Steam Isolation valves during a depressurization transient caused by a failure of the pressure regulator.

These same Barksdale switches were replaced with SOR switches in Unit 2 in June 1985. At that time, the licensee discovered that the new SOR pressure switch trip function could not consistently meet the less than or equal to 1 second response time required by the Technical Specifications in Table 3.3.2-3. The licensee took numerous steps to obtain conformance with the 1 second response time. These steps included testing at various ramp rates, discussions with the instrument supplier and General Electric Company (La Salle's vendor), and additional retesting and various configurations (without instrument snubbers and with modified snubbers). All of this effort was to no avail. The switches' average response time remains at or about 1.1 seconds.

The Final Safety Analysis Report of the Main Steam Isolation Valve (MSIV) closure transients includes: (1) the overpressure aspects from rapid closure from 3 seconds of MSIV closure at 105 percent steam flow conditions, and (2) the long-term MSIV closure where 105 percent steam flow is assumed to persist for 30 seconds due to loss of alternating current power and other adverse failures indicating acceptable results. The safety analysis of concern to this Technical Specification Table 3.3.2-3 A.1.C(2) apportioned the response time as follows:

Sensor - 1.0 second

MSIV - 13.0 seconds (actually Diesel Generator start interval dominates the MSIV actual closure time which is 3 to 5 seconds).

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The safety analysis for the Main Steam Line Low Pressure isolation via MSIVs was done at 825 psi pressure (analytical minimum limit to assure validity above the GEXL fuel correlation threshold). The allowable Technical Specification setpoint is 834 psig and the nominal setpoint is 854 psig. At the analytical limit of 825 psi, the 30 second MSIV closure time (total of sensor and valve and its auxiliaries) is acceptable. At this condition, the present Technical Specification utilizes 14 total seconds of this 30 second safety analysis interval. The revised Technical Specification extends this utilization to 15 total seconds (2 seconds for the sensor and the same 13 seconds for the valve) of the 30 second interval. This is an insignificant change in the pre-analyzed safety envelope because the protective isolation action (MSIV closure) occurs prior to the time when the system pressure decreases to the analytical limit (825 psi) even from the lowest allowable Technical Specification value (834 psig).

The purpose of the low pressure isolation is to protect the fuel by restricting reactor operation to the pressure regimes covered by the data base for the GEXL correlation. Because the analyses using the 30 second MSIV closure time are still applicable, the staff concludes that use of the GEXL correlation is still valid. We find the proposed Technical Specification change acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation and use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (51 FR 3712) on January 29, 1986. No public comments were received, and the state of Illinois did not have any comments.

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Bournia, BWR Project Directorate No. 3, DBL

Dated: ~~May~~ 13 1970