

April 24, 1990

Docket Nos. 50-373
and 50-374

Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
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Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. NPF-11 AND AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC. NOS. 71077 AND 71078)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. NPF-11 and Amendment No.57 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letter dated September 9, 1988.

The amendments revise the LaSalle County Station, Units 1 and 2. The inspection period for Technical Specification 3/4.7.9 regarding subsequent visual inspection periods when there are no inoperable snubbers of each type on any system per inspection period is being changed. These amendments provide flexibility for snubber inspections to accommodate the 18-month operating cycle.

A copy of the related Safety Evaluation supporting Amendment No. 73 to Facility Operating License No. NPF-11 and Amendment No. 57 to Facility Operating License No. NPF-18 is enclosed. Notice of Issuance of these amendments will be published in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/s/

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 73 to NPF-11
2. Amendment No. 57 to NPF-18
3. Safety Evaluation

Handwritten initials: C/P

cc w/enclosures:
See next page

OFFICIAL RECORD COPY
Document Name: AMENDMENT 71077/71078

PDIII-2:LA
LLuther
4/19/90

PDIII-2:PM
RPulsifer
4/19/90

NRR:EMEB
LMarsh
4/11/90

PDIII-2:PD
JCraig
4/15/90

OGC
4/12/90

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 24, 1990

Docket Nos. 50-373
and 50-374

Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE
NO. NPF-11 AND AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE
NO. NPF-18 - LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC. NOS.
71077 AND 71078)

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. NPF-11 and Amendment No. 57 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2. These amendments are in response to your letter dated September 9, 1988.

The amendments revise the LaSalle County Station, Units 1 and 2. The inspection period for Technical Specification 3/4.7.9 regarding subsequent visual inspection periods when there are no inoperable snubbers of each type on any system per inspection period is being changed. These amendments provide flexibility for snubber inspections to accommodate the 18-month operating cycle.

A copy of the related Safety Evaluation supporting Amendment No. 73 to Facility Operating License No. NPF-11 and Amendment No. 57 to Facility Operating License No. NPF-18 is enclosed. Notice of Issuance of these amendments will be published in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 73 to NPF-11
2. Amendment No. 57 to NPF-18
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Thomas J. Kovach
Commonwealth Edison Company

LaSalle County Nuclear Power Station
Units 1 & 2

cc:

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Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated September 9, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script that reads "Richard F. Dudley".

Richard F. Dudley, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance:

ENCLOSURE TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 7-27

INSERT

3/4 7-27

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All hydraulic and mechanical snubbers shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3. OPERATIONAL CONDITIONS 4 and 5 for snubbers located on systems required OPERABLE in those OPERATIONAL CONDITIONS.

ACTION:

With one or more snubbers inoperable, on any system, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Inspection Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these groups (inaccessible and accessible) may be inspected independently according to the schedule below. The first inservice visual inspection of each type of snubber shall be performed after 4 months but within 10 months of commencing POWER OPERATION and shall include all hydraulic and mechanical snubbers. If all snubbers of each type on any system are found OPERABLE during the first inservice visual inspection, the second inservice visual inspection of that system shall be performed at the first refueling outage. Otherwise, subsequent visual inspections of a given system shall be performed in accordance with the following schedule:

<u>No. Inoperable Snubbers of Each Type On Any System per Inspection Period</u>	<u>Subsequent Visual Inspection Period*#</u>
0	18 months - 50% + 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
8 or more	31 days ± 25%

*The inspection interval for each type of snubber on a given system shall not be lengthened more than one step at a time unless a generic problem has been identified and corrected; in that event the inspection interval may be lengthened one step the first time and two steps thereafter if no inoperable snubbers of that type are found on that system.

#The provisions of Specification 4.0.2 are not applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.57
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated September 9, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard F. Dudley, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance:

ENCLOSURE TO LICENSE AMENDMENT NO. 57

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 7-28

INSERT

3/4 7-28

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All hydraulic and mechanical snubbers shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3. OPERATIONAL CONDITIONS 4 and 5 for snubbers located on systems required OPERABLE in those OPERATIONAL CONDITIONS.

ACTION:

With one or more snubbers inoperable, on any system, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Inspection Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these groups (inaccessible and accessible) may be inspected independently according to the schedule below. The first inservice visual inspection of each type of snubber shall be performed after 4 months but within 10 months of commencing POWER OPERATION and shall include all hydraulic and mechanical snubbers. If all snubbers of each type on any system are found OPERABLE during the first inservice visual inspection, the second inservice visual inspection of that system shall be performed at the first refueling outage. Otherwise, subsequent visual inspections of a given system shall be performed in accordance with the following schedule:

<u>No. Inoperable Snubbers of Each Type On Any System per Inspection Period</u>	<u>Subsequent Visual Inspection Period*#</u>
0	18 months - 50% + 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
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8 or more	31 days ± 25%

*The inspection interval for each type of snubber on a given system shall not be lengthened more than one step at a time unless a generic problem has been identified and corrected; in that event the inspection interval may be lengthened one step the first time and two steps thereafter if no inoperable snubbers of that type are found on that system.

#The provisions of Specification 4.0.2 are not applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. NPF-11 AND
AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. NPF-18
COMMONWEALTH EDISON COMPANY
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter dated September 9, 1988 (C. M. Allen, CECO to U.S. NRC), Commonwealth Edison Company (CECO) (the licensee), proposed a Technical Specification change to the subsequent visual inspection period when no inoperable snubbers of each type on any system per inspection period are found in LaSalle County Station, Units 1 and 2. The inspection period is being changed to reflect the current operating cycle of 18 months.

2.0 BACKGROUND

Current specification 3/4.7.9 allows the subsequent visual inspection period for zero inoperable snubbers per inspection period to be 18 months $\pm 25\%$. If a particular refueling outage date is extended due to an unscheduled outage early in the cycle, the unscheduled outage could be too soon to do the snubbers visual inspection (18 months -25%). Yet the new refueling outage date could be too late (18 months $+25\%$) to do the inspection. The proposed amendment would allow an earlier inspection if the plant is shut down, thus avoiding an additional outage later to perform this inspection.

3.0 EVALUATION

This change would modify LaSalle County Station, Units 1 and 2, Technical Specification requirement under Section 4.7.9.b, "Subsequent Visual Inspection Period" for zero failures from 18 months ($\pm 25\%$) to 18 months ($-50\% +25\%$). This would allow greater flexibility in snubber surveillance. The 18 months -50% period is more conservative and is still within the interval of inspection where there have been two (2) failures. Nine months is sufficient operating time to challenge the snubbers. The maximum interval of 18 months $+25\%$ will not be affected.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes to requirements with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20. The staff had determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant

increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (53 FR 53089) on December 30, 1988, and consulted with the state of Illinois. No public comments were received, and the state of Illinois did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Pulsifer, NRR/PDIII-2

Dated: April 24, 1990